



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-237

DRESDEN NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 112
License No. DPR-19

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated July 31, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B. of Provisional Operating License No. DPR-19 is hereby amended to read as follows:

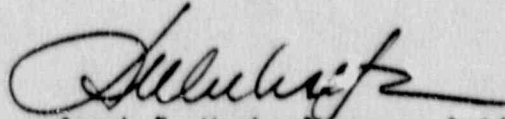
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B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 112, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Jacob F. Wechselberger, Acting Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 9, 1990

ATTACHMENT TO LICENSE AMENDM. NO. 112

PROVISIONAL OPERATING LICENSE NO. DPR-19

DOCKET NO. 50-237

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3/4.7-6

3/4.7-7

3/4.7-8

3/4.7-9

INSERT

3/4.7-6

3/4.7-7

3/4.7-8

3/4.7-9

3/4.7-9a

3.7 LIMITING CONDITION FOR OPERATION
(Cont'd.)

4.7 SURVEILLANCE REQUIREMENTS
(Cont'd.)

- (1) An overall integrated leakage rate for Type A tests of:
 - (a) L_{am} less than or equal to 75 percent of L_a . The RBCCW pathway described in Section 4.7.A.2.e.(4) is excluded until the end of the Cycle 12 refueling outage.
 - (b) L_{tm} less than or equal to 75 percent of L_t .
- (2) (a) A combined leakage rate of less than or equal to 60 percent of L for all testable penetrations and isolation valves subject to Type B and C tests except for main steam isolation valves. The RBCCW pathway described in Section 4.7.A.2.e.(4) is also excluded until the end of the cycle 12 refueling outage.

3.7 LIMITING CONDITION FOR OPERATION
(Cont'd.)

(b) A leakage rate of less than or equal to 3.75 percent of L_0 for any one air lock when pressurized to 10 psig.

(c) 11.5 SCF per hour for any main steam isolation valve at a test pressure of 25 psig.

4.7 SURVEILLANCE REQUIREMENTS
(Cont'd.)

c. If two consecutive Type A tests fail to meet either 75 percent of L_0 or 75 percent of L_1 , a Type A test shall be performed at each shutdown for refueling or approximately every 18 months until two consecutive Type A tests meet the above requirements, at which time the normal test schedule may be resumed.

d. The accuracy of each Type A test shall be verified by a supplemental test which:

3.7 LIMITING CONDITION FOR OPERATION
(Cont'd.)

4.7 SURVEILLANCE REQUIREMENTS
(Cont'd.)

- (1) Confirms the accuracy of the test by verifying that the difference between the supplemental data and the Type A test data is within 25 percent of L_a or 25 percent of L_t .
 - (2) Has a duration sufficient to establish accurately the change in leakage rate between the Type A test and the supplemental test.
 - (3) Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 9 scfm.
- e. Type B and C tests shall be conducted at P_a , at intervals no greater than 24 months except for tests involving:

3.7 LIMITING CONDITION FOR OPERATION
(Cont'd.)

4.7 SURVEILLANCE REQUIREMENTS
(Cont'd.)

- (1) Main steam line isolation valves which shall be tested at a pressure of 25 psig each operating cycle.
- (2) Bolted double-gasketed seals which shall be tested at a pressure of 48 psig whenever the seal is closed after being opened and each operating cycle.
- (3) Air locks which shall be tested at 10 psig each operating cycle.
- (4) The Reactor Building Closed Cooling Water (RBCCW) inlet pathway to the primary containment (penetration X-123) will not be tested until the end of the Cycle 12 refueling outage.

f. Continuous Leak Rate Monitor

- (1) When the primary containment is inerted, the containment

3.7 LIMITING CONDITION FOR OPERATION
(Cont'd.)

4.7 SURVEILLANCE REQUIREMENTS
(Cont'd.)

shall be continuously monitored for gross leakage by review of the inerting system make-up requirements.

- (2) This monitoring system may be taken out of service for the purpose of maintenance or testing but shall be returned to service as soon as practical.

- g. The interior surfaces of the drywell shall be visually inspected each operating cycle for evidence of deterioration.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-249

DRESDEN NUCLEAR POWER STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 108
License No. DPR-25

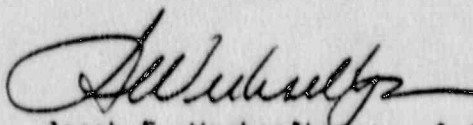
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated July 31, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B. of Facility Operating License No. DPR-25 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 108, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Jacob F. Wechselberger, Acting Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 9, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 108

FACILITY OPERATING LICENSE NO. DPR-25

DOCKET NO. 50-249

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3/4.7-6

3/4.7-7

3/4.7-8

3/4.7-9

INSERT

3/4.7-6

3/4.7-7

3/4.7-8

3/4.7-9

3/4.7-9a

3.7 LIMITING CONDITION FOR OPERATION
(Cont'd.)

4.7 SURVEILLANCE REQUIREMENTS
(Cont'd.)

(1) An overall
integrated
leakage rate
for Type A
tests of:

(a) L_{am} less
than or
equal to 75
percent of
 L_t . The RBCCW
pathway de-
scribed in Sec-
tion 4.7.A.2.e(4)
is excluded until
the end of the
Cycle 12 refuel-
ing outage.

(b) L_{tm} less
than or
equal to 75
percent of
 L_t .

(2) (a) A combined
leakage rate
of less than
or equal to
60 percent
of L_t for
all testable
penetrations
and isola-
tion valves
subject to
Type B and C
tests except
for main
steam isola-
tion valves.
The RBCCW path-
way described
in Section
4.7.A.2.e.(4)
is also excluded
until the end of
Cycle 12 refueling
outage.

3.7 LIMITING CONDITION FOR OPERATION
(Cont'd.)

(b) A leakage rate of less than or equal to 3.75 percent of L_a for any one air lock when pressurized to 10 psig.

(c) 11.5 SCF per hour for any main steam isolation valve at a test pressure of 25 psig.

4.7 SURVEILLANCE REQUIREMENTS
(Cont'd.)

c. If two consecutive Type A tests fail to meet either 75 percent of L_a or 75 percent of L_t , a Type A test shall be performed at each shutdown for refueling or approximately every 18 months until two consecutive Type A tests meet the above requirements, at which time the normal test schedule may be resumed.

d. The accuracy of each Type A test shall be verified by a supplemental test which:

(1) Confirms the accuracy of the test by verifying that the difference

3.7 LIMITING CONDITION FOR OPERATION
(Cont'd.)

4.7 SURVEILLANCE REQUIREMENTS
(Cont'd.)

between the supplemental data and the Type A test data is within 25 percent of L_a or 25 percent of L_t .

- (2) Has a duration sufficient to establish accurately the change in leakage rate between the Type A test and the supplemental test.
 - (3) Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 9 scfm.
- e. Type B and C tests shall be conducted at P_a , at intervals^a no greater than 24 months except for tests involving:
- (1) Main steam line isolation valves which shall be tested at a pressure of 25 psig each operating cycle.

3.7 LIMITING CONDITION FOR OPERATION
(Cont'd.)

4.7 SURVEILLANCE REQUIREMENTS
(Cont'd.)

(2) Bolted double-gasketed seals which shall be tested at a pressure of 48 psig whenever the seal is closed after being opened and each operating cycle.

(3) Air locks which shall be tested at 10 psig each operating cycle.

(4) The reactor Building Closed Cooling Water (RBCCW) inlet pathway to the primary containment (penetration X-123) will not be tested until the end of the Cycle 12 refueling outage.

f. Continuous Leak Rate Monitor

(1) When the primary containment is inerted, the containment shall be continuously monitored for gross leakage by review of the inerting system make-up requirements.

3.7 LIMITING CONDITION FOR OPERATION
(Cont'd.)

4.7 SURVEILLANCE REQUIREMENTS
(Cont'd.)

(2) This monitoring system may be taken out of service for the purpose of maintenance or testing but shall be returned to service as soon as practical.

g. The interior surfaces of the drywell shall be visually inspected each operating cycle for evidence of deterioration.