Procedures Evaluation Checklist for Maintenance, Test and Calibration Procedures Used in Nucle Power Plants

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HPT Inc.

Sandia National Laboratories

Prepared for U.S. Nuclear Regulatory Commission

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ABSTRACT

This report describes a checklist to be used by the United States Nuclear Regulatory Commission (NRC) inspectors during their evaluation of maintenance, test, and calibration procedures. The objective of the checklist is to aid inspectors in identifying procedural characteristics that can lead to human performance deficiencies. A companion document, Development of a Checklist for Evaluating Maintenance, Test, and Calibration Procedures Used in Nuclear Power Plants, NUREG/CR-1368, SAND80-7053, describes how the checklist was developed.

Revision 1 of the checklist, presented herein, is the result of a oneyear field test by NRC inspectors in all five NRC regions. It incorporates improvements that were suggested by inspectors based on their experience with the checklist in performing evaluations of licensee procedures.

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CHECKLIST FOR EVALUATING MAINTENANCE, TEST, AND CALIBRATION PROCEDURES USED IN NUCLEAR POWER PLANTS

INTRODUCTION

This document describes the checklist to be used by United States Nuclear Regulatory Commission (NRC) inspectors for evaluating maintenance, test, and calibration procedures and provides guidelines for its application. The checklist is intended to aid inspectors in identifying procedural deficiencies that can lead to errors in performance. The procedures evaluation checklist is the product of an extensive study of nuclear power plant operations and procedures sponsored by the Office of Inspection and Enforcement. A detailed description of the study is provided in a companion document, Development of a Checklist for Evaluating Maintenance, Test, and Calibration Procedures Used in Nuclear Power Plants, NUREG/CR-1368, SAND80-7053.

As part of the study, abstracts of licensee event reports (LERs) submitted by all plants during the four-year period 1975-1978 were reviewed. The purpose of the review was to identify the specific kinds of procedures-related personnel error that have been associated with the performance of maintenance, test, and calibration activities. A total of 751 LERs was attributable to procedural deficiencies. An analysis identified eight categories of performance errors resulting from procedural deficiencies. They are summarized in Table 1. Each of the procedures evaluation criteria contained in the checklist deals with procedural characteristics related to one or more categories of performance error. The use of a procedure that is deficient with respect to these criteria can lead to errors in performance.

The checklist was distributed to the NRC regional offices for a field test by NRC inspectors who used it in evaluating licensee maintenance, test, and calibration procedures. As a result of inspector experiences with the checklist in field situations, the regional offices suggested that a number of changes be made in its content and method of application. Most of their suggestions have been incorporated in Revision 1 of the checklist described herein, resulting in an improved checklist and a more efficient procedure evaluation process. The checklist begins on page 3. An explanation of the checklist items begins on page 8. A suggested method of applying the checklist is described beginning on page 18.

Table 1. Performance Errors Associated with Maintenance, Test, and Calibration Activities

Category

Definition

Non-Compliance

Unauthorized Action. A procedure was available but not used or followed. An action not in the procedure was performed or a step was performed out of the sequence specified in the procedure.

Omission. A procedure was used but steps were not performed.

Incorrect Action. A procedure was used but an error was committed in the performance of a step.

Misalignment

Valves, switches, jumpers, relays, fuses, breakers, or solenoids were incorrectly positioned before, during, or after a procedure.

Preparation of Inaccurate Procedures

Personnel prepared inaccurate procedures. As a consequence, users performed incorrect actions.

Preparation of Incomplete Procedures

Personnel prepared incomplete procedures. Information needed by users was missing from procedures. As a consequence, users omitted important actions.

Failure to Revise Procedures Personnel were not informed of necessary actions following completion of a procedure, e.g., a test following repair.

Communication

An extra-procedural communication, e.g., phone, intercom, was misunderstood.

Comprehension

The user misinterpreted the instructions in a procedure.

CHECKLIST FOR EVALUATING

MAINTENANCE, TEST, AND CALIBRATION PROCEDURES

Procedure Title/No.__

Revi	ision_	Revi	ewer by		Date
poss	sesses ck <u>No</u> .	the characte	ristic, check Not Applicable	e following charact Yes; if it lacks t e) if the character	he characteristic,
on 1	the pe	rformance of haracteristic	personnel usi rated A, per	relative impact of ng the procedures. formance error is m formance error is l	If a procedure ost likely; if it
Per	form [ocument Revie	w Evaluation	for Items #1—#26.	
		Item			Rating Yes No N/A
1.	cedur	e? (Also, if enance or tes	the procedur	pose of the pro- e is for scheduled should state the y.)	c
2.		on a cover pa	ge or first p		D
			title and nu	mber	
		2.2 Revision		-6.1-1	
			er (if applic		
3.			signature and ovide the foll	owing identifica-	D
		3.1 Procedure	number and/o	r title	
		3.2 Revision	number		
		3.3 Unit numb	er (if applic	able)	
		3.4 Page numb	per		
4.			orary procedur opiration date	re, is it clearly	D
5.		able by marking		re clearly iden- eof; Final	D

	<u>I tem</u>	Rating	Yes	No	N/A
6.	Does the procedure provide a statement of purpose or brief description which clearly specifies the function it performs? The description should appear in an introductory section preceding the instructions.	D			
7.	Does the procedure provide the following job planning information in an introductory section preceding the instructions?				
	7.1 Other actions or procedures which must be completed prior to use.	В	_		
	7.2 Plant, system, or equipment conditions which must exist prior to use.	В			
	7.3 Precautions which must be observed in the performance of the procedure. For example, are applicable radiological requirements and precautions specified	? C			
	7.4 The specific equipment (by part number and/or unique nomenclature) to which the procedure is applicable.	e D			
	7.5 Special tools and test equipment required to perform the procedure (by part number and/or unique nomenclature).	В			
	7.6 Other documents, e.g., procedures, drawings, schematics, required to perform procedure.	С			
8.	If critical coordination of activities of two or more persons is required to perform the procedure, does the procedure provide a means for coordinating their activities? For example, is critical communication between persons located remotely from each other specified?	С			
9.	Does the procedure provide adequate quality control hold points?	С			
10.	Does the procedure provide for verification and signoff of actions?	В			
11.	If the answer to Item $\#10$ is \underline{Yes} , are the verifications predominantly performed by persons other than those performing the action?	С			
12.	If the procedure refers to a skill of the craft task, i.e., is general rather than specific, go directly to Item #30 to evaluate the procedure.				
13.	Are the instructions written in short, concise, identifiable steps as cpposed to multi-step paragraphs?	В			

	<u>Item</u>	Rating	Yes	No	N/A
14.	Complexity Index (CI): Evaluate the complexity of the instructions by determining the average number of actions (verbs) called out per step. Base estimate on a sample of 20% of the steps or, if the sample size is less than 10, use all steps.				
	Is the average number of actions per step 1.5 or less?	В			
15.	Specificity Index (SI): Evaluate the level of specificity of a procedure by determining the percent of steps in a selected sample that meet all of the following criteria.				
	15.1 The action to be taken is specifically identified (open, close, torque, etc.).				
	15.2 Limits (if applicable) are expressed quantitatively (2 turns, 100 inch lbs., etc.).				
	15.3 The equipment or parts are identified completely (HPCI-MO-17, etc.).				
	Base the estimate on a sample of 20% of the steps in a given procedure or a minimum of 10 steps.				
	Do at least 90% of the steps evaluated meet all the above criteria?	В			
16.	If precautions or explanations are applicable to the performance of specific steps or series of steps, are they placed immediately ahead of the step(s) to which they apply?	С			
17.	Are graphs, charts, and tables adequate for readability and interpolation or extraction of values?	С			
18.	Do worksheets provide enough space to record data and perform necessary calculations? For example, in calibration procedures are there provisions for an "as found" column and an "as left" column?	С			
19.	Does the procedure (or related data sheets or work- sheets) provide for the independent verification and signoff of computations?	С			
20.	Are acceptance criteria and limits stated in quantitative terms?	С			
21.	Are units expressed as ranges rather than point values whenever possible?	С			
22.	Are the acceptance criteria and limits compatible with limits specified in requirements documents?	Δ			

	<u>I tem</u>	Rating	Yes	No	N/A
23.	If computations are required by the procedure, are they based on technically accurate, complete, and up-to-date formulas?	А			
24.	If items (valves, breakers, relays, solenoids, jumpers, fuses, switches) require alignment to perform the procedure, do the alignment instructions in the procedure meet all of the following criteria?	В			
	24.1 Each item requiring alignment is individually specified. (Note—It is not acceptable to refer personnel to previous steps).				
	24.2 Each item is identified with a unique number or nomenclature.				
	24.3 The position in which the item is to be placed is specified.				
	24.4 The position in which the item is placed is verified and checked off or signed off.				
25.	If any of the above alignment instructions are for system restoration, is the verification performed by someone other than the person performing the alignment?	В			
26.	If any follow-on action, test, or procedure must be performed upon the completion of this procedure, does the procedure or a related document (e.g., work order) instruct the user regarding what follow-on action is required and whom to notify?	С	_		
27.	Does the procedure provide instructions for reasonable contingencies? For example, if equipment is operating outside the range specified by the procedure, is the person instructed what action to take?	С			
Per	form a Walk-Through of Procedure for Items #28 and #29				
28.	Are equipment numbers and/or nomenclature used in the procedure the same as those which are displayed on the equipment?	В			
29.	Are the units of measurement used in the procedure the same as those displayed on equipment?	В			

Observe Licensee Simulate a Performance of Procedure or Observe Performance of Procedure for Items #30 and #31

	Item	Rating	Yes	No	N/A
30.	Determine whether the amount and kind of information (level of detail) provided by the procedure are adequate for the intended users.				
	Are the following criteria met?:				
	30.1 Can the procedure be performed in the sequence it is written?	А			
	30.2 Can the user locate and identify all equipment referred to in the instructions?	Α			
	30.3 Where general rather than specific instructions are provided, can the user explain in detail how to perform the general instructions?	Α			
	30.4 Can the user perform the procedure without obtaining additional information from persons or documents not specified by the procedure?	В			_
	30.5 Can the user perform the procedure without obtaining direct assistance from persons not specified by the procedure?	В			
31.	Is the sensitivity of the test instruments and tools being used adequate and are they in proper calibration?	В	_		
Act	ion:				
Dis	position:		-		
				-	

DISCUSSION OF THE CHECKLIST

Item Format

The procedures evaluation criteria are expressed in question form so that they can be answered by Yes or No. They are constructed so that a Yes answer indicates that the procedure possesses a desirable characteristic. A No answer indicates a procedural deficiency. In some cases, it will not be possible to evaluate a procedure on a characteristic because it is not applicable to the procedure. For example, a procedure cannot be evaluated regarding the adequacy of graphs, charts, or tables if it does not contain any of these items.

In this case, check Not Applicable rather than leave the item unanswered.

Explanation of Checklist Items

The checklist items are listed below. In cases where the relationship between a procedural characteristic and the quality of human performance might not be apparent, an explanation is provided

 Does the title describe the purpose of the procedure? (Also, if the procedure is for scheduled maintenance or tests, the title should state the frequency, e.g., weekly, monthly.)

Explanation. The title is the first information read by the user and therefore should be descriptive of the contents of the procedure. A clear, unique title will make it easier to identify a given procedure among other similar procedures. The use of titles that are descriptive of the purpose of the procedure will reduce the probability of incorrect procedure selection.

- 2. Does the procedure provide the following information on a cover page or first page?
 - 2.1 Procedure title and number
 - 2.2 Revision number
 - 2.3 Unit number (if applicable)
 - 2.4 Approval signature and date

Explanation. Performance errors have occurred because users have selected incorrect or out-of-date procedures. Descriptive titles and revision information will aid in the selection of correct and current procedures. If this is a multiple-unit site, separate, identifiable procedures for each unit will reduce the probability of performing procedures on the wrong unit.

- 3. Does each page provide the following identification information?
 - 3.1 Procedure number and/or title

3.2 Revision number

3.3 Unit number (if applicable)

3.4 Page number

Explanation. Page identification information facilitates effective document control and allows users to check that their procedures are complete and that the pages are in the correct order.

4. If this is a temporary procedure, is it clearly marked with the expiration date?

Explanation. None.

5. Is the last page of the procedure clearly identifiable by marking, e.g., Fage of ; Final Page?

Explanation. The last page of a procedure is most vulnerable to becoming detached and lost. It should be made obvious to the user if the last page is missing.

6. Does the procedure provide a statement of purpose or brief description which clearly specifies the function it performs? The description should appear in an introductory section preceding the instructions.

Explanation. None.

- 7. Does the procedure provide the following job planning information in an introductory section preceding the instructions?
 - 7.1 Other actions or procedures which must be completed prior to use.
 - 7.2 Plant, system, or equipment conditions which must exist prior to use.
 - 7.3 Precautions which must be observed in the performance of the procedure. For example, are applicable radiological requirements and precautions specified?
 - 7.4 The specific equipment (by part number and/or unique nomenclature) to which the procedure is applicable.
 - 7.5 Special tools and test equipment required to perform the procedure (by part number and/or unique nomenclature).
 - 7.6 Other documents, e.g., procedures, drawings, schematics, required to perform procedure.

Explanation. The provision of job planning information will enable personnel to make adequate preparations for performing the procedure

and reduce the probability of taking on-the-job "shortcuts" or innovations due to incomplete job planning.

8. If critical coordination of activities of two or more persons is required to perform the procedure, does the procedure provide a means for coordinating their activities? For example, is critical communication between persons located remotely from each other specified?

Explanation. Many procedures require close coordination of actions among several persons located remotely from each other. However, by reading the procedure, it is frequently not possible to determine which person is being instructed by a particular step. Also, the procedure might not specify critical communications to ensure that the step has been initiated or completed. To the extent that complex, multi-man activities are not specified, errors in communications and omissions of actions can result.

9. Does the procedure provide adequate quality control hold points?

Explanation. Maintenance procedures should provide for inspecting equipment at appropriate points during the repair or service process to verify that the procedure is being performed correctly. For example, an assembly procedure might specify that a physical check of bearing installation be performed before a cover or other parts are installed that would prevent access to the bearing.

10. Does the procedure provide for verification and signoff of actions?

Explanation. See Item #11.

11. If the answer to Item #10 if Yes, are the verifications predominantly performed by persons other than those performing the action?

Explanation. Verification is the primary method for ensuring compliance with procedures. Self-verification by checking, initialling or signing steps serves as an aid or reminder to the procedure user to perform the step. However, it is too easily subject to abuse to serve as a compliance control. If it is important to ensure compliance with an action because of the consequences of a performance error, verification by someone other than the person performing the action is in order. It is required if an error might otherwise remain undetected.

12. If the procedure refers to a skill of the craft task, i.e., is general rather than specific, go directly to Item #30 to evaluate the procedure.

Explanation. Many procedures that are intended for use by skilled craftsmen lack detail. The performance of Item #30 enables the

procedure evaluator to check the adequacy of a procedure with respect to the qualifications of the personnel permitted to use it.

13. Are the instructions written in short, concise, identifiable steps as opposed to multi-step paragraphs?

Explanation. Studies have shown that the speed of reading and comprehension of written instructions are improved if the instructions are presented in short, concise sentences. Ideally, an instruction should consist of an action verb and the object of the action—plus action limits and object identifiers, and, if necessary, object locators. Additional information (such as that contained in explanations and descriptions) that is intended to aid the user to accomplish the action more effectively should ordinarily be presented in the form of a note preceding the action instruction.

14. Complexity Index (CI): Evaluate the complexity of the instructions by determining the average number of actions (verbs) called out per step. Base estimate on a sample of 20% of the steps or, if the sample size is less than 10, use all steps.

Is the average number of actions per step 1.5 or less?

Explanation. The complexity index (CI) of a procedure is defined as the average number of actions stated in the instructional steps or paragraphs. The average is computed from a random sample of steps or paragraphs in a procedure.

 ${\tt CI = \frac{Number\ of\ Actions\ in\ a\ Sample\ of\ Steps\ or\ Paragraphs}{Number\ of\ Steps\ or\ Paragraphs\ Sampled}}$

The number of actions is simply the number of verbs in a step or paragraph. For example, the instruction "Turn switch XXX to position No. 2, observe value on pressure gauge XX, and record value" has three actions. The more actions that are expressed, the less likely they will be recalled accurately, particularly if they are unrelated actions. Ideally, a step should contain only one action unless the actions are related, in which case up to three actions in a step are acceptable. Related actions are a group of actions required to produce a single result. The example illustrates three related actions. Their single object is to obtain a value.

- 15. Specificity Index (SI): Evaluate the level of specificity of a procedure by determining the percent of steps in a selected sample that meet all of the following criteria.
 - 15.1 The action to be taken is specifically identified (open, close, torque, etc.).

15.2 Limits (if applicable) are expressed quantitatively (2 turns, 100 inch lbs., etc.).

15.3 The equipment or parts are identified completely (HPCI-MO-17, LPV-6, etc.).

Base the estimate (a sample of 20% of the steps in a given procedure or a minimum of 10 steps.

Do at least 90% of the steps evaluated meet all the above criteria?

Explanation. The above criteria list the basic characteristics of a specific (versus general) instruction. Fewer errors of interpretation or omission result from instructions with high specificity.

16. If precautions or explanations are applicable to the performance of specific steps or series of steps, are they placed immediately ahead of the step(s) to which they apply?

Explanation. None.

17. Are graphs, charts, and tables adequate for readability and interpolation or extraction of values?

Explanation. Misinterpretation of graphs, charts, and tables has resulted in performance errors. It is often traceable to poor readability of these materials—which, in turn, is attributable to 1) inadequate reproduction or 2) inadequate original construction. The following guidelines are provided to evaluate readability.

Reproduction—In some cases, copies are so many generations removed from the original or master copy that lines in graphs, charts, and tables have deteriorated or disappeared, making it difficult to track or interpolate values. Letters and numbers can undergo similar deterioration. Also, materials have sometimes been reduced in size so that readability is impaired. Letters and numbers should be at least 1/8 in. in height, unbroken, and unfilled. All lines in the reproductions should be as visible as they are in the original or master copies. First, compare the reproductions to the originals or master copies. Then evaluate the readability of the reproductions under the conditions of illumination in which personnel use them.

Original construction—Letters and numbers should be typed rather than handwritten. Lines on graph paper should be reproducible on licensee reproduction equipment. On graphs, units of measurement used in plotted values should be compatible with divisions on graph paper. That is, if plotted values progress in units of five (e.g., 5, 10, 15, etc.) it is better to separate the values by five lines than by four lines. To facilitate accuracy of locating values in charts and

tables look for such aids as 1) partitioning tables with lines, 2) arranging values in subgroups, e.g., inserting spaces between subgroups of five values, and 3) placing connecting lines between values or between nomenclature and values.

18. Do worksheets provide enough space to record data and perform necessary calculations? For example, in calibration procedures are there provisions for an "as found" column and an "as left" column?

Explanation. None.

19. Does the procedure (or related data sheets or worksheets) provide for the independent verification and signoff of computations?

Explanation. None.

20. Are acceptance criteria and limits stated in quantitative terms?

Explanation. None.

21. Are units expressed as ranges rather than point values whenever possible?

Explanation. When equipment does not permit the setting of point values, or when a range of values is acceptable, the acceptance criteria should be expressed in terms of ranges. However, they should be expressed in a form to avoid errors of addition, subtraction, or conversion. Example:

Preferable	Not Preferable
	125 ⁺ 6 gpm
(Best) 125 gpm (119 - 131)	125 + 4.8% gpm (Worst)

22. Are the acceptance criteria and limits compatible with limits specified in requirements documents?

Explanation. Acceptance criteria and limits can be found in Technical Specifications as well as other documents, e.g., equipment technical manuals.

23. If computations are required by the procedure, are they based on technically accurate, complete, and up-to-date forumulas?

Explanation. None.

- 24. If items (valves, breakers, relays, solenoids, jumpers, fuses, switches) require alignment to perform the procedure, do the alignment instructions in the procedure meet all of the following criteria?
 - 24.1 Each item requiring alignment is individually specified.

 (Note—It is not acceptable to refer personnel to previous steps).

24.2 Each item is identified with a unique number or nomenclature.

24.3 The position in which the item is to be placed is specified.

24.4 The position in which the item is placed is verified and checked off or signed off.

Explanation. Two of the primary factors associated with misalignment are lack of specificity of instructions and lack of physical verification of position. The criteria listed above are aimed at improving specificity and verification. In some procedures it was found that instructions were adequate for initial alignment but shortchanged realignment by simply directing personnel to "Reposition valves listed in Step 5". In this instance, personnel were not provided a means within the procedure for verifying valve positions. The instruction should have relisted the valves, specified their new positions, and provided for checkoff or signoff for each valve.

25. If any of the above alignment instructions are for system restoration, is the verification performed by someone other than the person performing the alignment?

Explanation. It was found that up to three-fourths of undetected alignment errors occur during restoration. Independent physical verification of position is less likely to be performed during this process. The requirement for independent verification is aimed at reducing this error. The independent verification should involve physically checking the positions—not be confined to simply checking log entries and tags.

26. If any follow-on action, test, or procedure must be performed upon the completion of this procedure, does the procedure or a related document (e.g., work order) instruct the user regarding what follow-on action is required and whom to notify?

Explanation. None.

27. Does the procedure provide instructions for reasonable contingencies? For example, if equipment is operating outside the range specified by the procedure, is the person instructed what action to take?

Explanation. Many procedures are written as though all acceptance criteria will be met. They do not address the exceptions. Personnel should be instructed within the procedure what actions to take in the event criteria are not met.

Perform a Walk-Through of Procedure for Items #28 and #29

28. Are equipment numbers and/or nomenclature used in the procedure the same as those which are displayed on the equipment?

Explanation. None.

29. Are the units of measurement used in the procedure the same as those displayed on equipment?

Explanation. None.

Observe Licensee Simulate a Performance of Procedure or Observe Performance of Procedure for Items #30 and #31

30. Determine whether the amount and kind of information (level of detail) provided by the procedure are adequate for the intended users.

Are the following criteria met?

- 30.1 Can the procedure be performed in the sequence it is written?
- 30.2 Can the user locate and identify all equipment referred to in the instructions?
- 30.3 Where general rather than specific instructions are provided, can the user explain in detail how to perform the general instructions?
- 30.4 Can the user perform the procedure without obtaining additional information from persons or documents not specified by the procedure?
- 30.5 Can the user perform the procedure without obtaining direct assistance from persons not specified by the procedure?

Explanation. NRC inspectors are required to evaluate whether or not procedures are adequate for use by qualified personnel. Because of lack of definitions of adequacy and personnel qualifications, this assessment cannot be made definitively. There is considerable room for different interpretations and disagreement between inspectors and licensees. The above observations permit an objective assessment of procedural adequacy.

31. Is the sensivity of the test instruments and tools being used adequate and are they in proper calibration?

Explanation. Personnel may mistakenly use "equivalent" tools and instruments that do not meet sensitivity or calibration requirements.

Item Ratings

The ratings A. B. C. or D indicate the impact of an item on the quality of human performance. If a procedure is deficient with respect to the characteristic referred to by the item, a performance deviation is more likely to occur than if the procedure possesses the characteristic. The absence of some procedural characteristics is more likely to result in performance deviations than the absence of others. It is therefore necessary to develop a method of rating the checklist items to indicate to the evaluator the relative importance of the characteristic stated in the item. The rating considerations are shown in Table 2. These ratings, integrated with the inspector's own knowledge of the consequences of error associated with the performance of a specific procedure or action, should enable the inspector to assess the importance of correcting a particular procedural deficiency. In general, it should be considered mandatory to correct a deficiency rated A or B. Correction of a deficiency rated C may or may not be considered mandatory, depending upon the inspector's judgment regarding the consequences of error and situational stress factors associated with use of the procedure. A rating of D would not ordinarily be regarded as a mandatory change. Howev correction is desirable if the intent is to reduce the frequency formance error to the minimum rate attainable by means of procedures.

Table 2. Rating Scale for Procedural Deficiencies

	Probability of	Performance Deviati	on Under:
Rating	Low Stress*	Moderate Stress*	High Stress*
A	Moderate	High	High
В	Moderate	Moderate	High
C	Low	Moderate	Moderate
D	Low	Low	Moderate

^{*}These terms are defined with descriptions and examples in Swain, A.D. and Guttmann, H.E., <u>Handbook of Human Reliability Analysis with Emphasis on Nuclear Power Plant Applications</u>, NUREG/CR-1278, SAND80-0200, United States Nuclear Regulatory Commission, Washington, D.C., May 1980.

Description of Ratings

- A—Errors are likely to occur during low stress (normal) conditions and will be frequently made under moderate and high stress conditions.
- B---Errors are likely to occur during low and moderate stress conditions and will occur frequently under high stress.
- C—Errors are not very likely under low stress but could occur readily under moderate and high stress.
- D—Errors are not very likely to occur under low and moderate stress but could readily occur during high stress.

Evaluation Methods

The checklist employs three methods for evaluating procedures. The Document Review method is used to evaluate a procedure on Items #1 through #27. The Walkthrough Method is used for a procedure on Items #28 and #29 and the Observation Method applies to Items #30 and #31. The methods are described below.

Document Review. This method consists of collecting a sample of the procedures of interest and their related documents and then examining their contents and interrelationships. Typically, related documents will consist of 1) all drawings, procedures, schematics, etc. specifically referred to by the procedure, 2) technical specifications and other basic requirements documents which reasonably might affect the content of the procedures, and 3) corporation policies and station directives dealing with procedures contents, development, and implementation. These documents together comprise the information system affecting the performance of a maintenance, test, or calibration activity. If an inspector cannot evaluate a characteristic from the available documents alone when a document review has been specified, it can be assumed that the information system is deficient with respect to completeness or with respect to organization. Either deficiency will affect the quality of procedural content adversely. At the least, an information system must be auditable.

Walk-through. Some evaluations such as determining the correspondence between equipment nomenclature or identification numbers used in a procedure and the nomenclature or numbers actually displayed on equipment, can be performed only by walking through the facility with the procedure in hand and comparing the two. During the walk-through it might be desired to make selected human factors observations of the work environment, the facility layout, and the equipment, all of which bear upon the effectiveness and safety of personnel performance. For example, the inspector might wish to assess the readability of

legends and displays from the perspective of the person performing the procedure.

Observation. Unlike the preceding methods, the performance of this evaluation requires the direct support of licensee personnel. The objective of this method is to judge whether the amount and kind of information provided by the procedure is complete with respect to the information needs of the user. That is, the inspector seeks to evaluate the adequacy of the "level of detail" of the procedure. This attribute of a procedure is the most difficult of all procedural characteristics to evaluate. Judgments of adequacy of level of detail are based on assumptions about the qualifications of the personnel for whom the procedure is provided. Such assumptions are often tenuous at best because, unlike operators, documentation detailing the qualifications of personnel who perform these procedures—particularly maintenance procedures, is inadequate or non-existent.

To reduce the probability of human error in the performance of an activity, a procedure must be designed to be usable for the least qualified person permitted to use the procedure. This requirement implies that the procedure must provide all of the information needed by persons representative of that skill level and, furthermore, must express the information in understandable language (vocabulary, sentence structure). Partial evidence of the completeness of a procedure can be obtained by observing a person who is representative of the minimum skill level perform a walk-through of the procedure, simulating the actions specified in the instructions.

Typically, a procedure is composed of general and specific instructions. The user should be able to explain in detail how to perform a general instruction. The user should be able to perform the entire procedure without seeking information from other personnel, unless they are specified by the procedures, and without referring to documents that are not specified by the procedures. If either of these criteria is not met, the procedure is incomplete.

APPLICATION OF THE CHECKLIST

The checklist can be applied to serve either or both of two distinctly different purposes. They are:

- To identify deficiencies in a sample of procedures with the objective of correcting the deficiencies in that specific sample of procedures, and/or
- To identify deficiencies in a sample of procedures with the objective of correcting the process that produced the deficient procedures.

In the first case, the inspector is basically performing the function of an editor, and, as a result, the impact of the inspection process will be confined for the most part to the procedures being evaluated. The rate at which existing procedures are modified and new procedures are prepared far outpaces the rate at which they can be evaluated by NRC inspectors. Therefore, if only changes in identified deficiencies are sought, the objective of improving the effectiveness and safety efficiency of the inspection process will not be attained.

In the second case, the impact of the inspector on the quality of licensee procedures will be maximized. Here, the procedures are viewed primarily as indicators of the quality of the procedures development process. Although the inspector should also seek to have the specific procedures corrected, the main objective of the inspection is to determine the quality of the licensee's procedures. If the samples of procedures exhibit particular deficiencies in common with each other, a change in the process used by the licensee to prepare (or revise) procedures, or a change in the licensee's specification governing the format or content of the procedures is in order.

Based on inspector experience in evaluating procedures with the checklist, the following method of application is suggested.

- To determine deficiencies that are common to a set of procedures, sample a small number of procedures that is representative of the licensee's maintenance, test, or calibration procedures. Evaluate the procedures in detail with the checklist to determine whether they have deficiencies in common with each other. After they have been identified, it may be unnecessary to review subsequent procedures on these generic characteristics.
- To identify deficiencies specific to a particular procedure perform a review for the following characteristics in the order listed below. Complete the review of a procedure on one characteristic before proceding to the next characteristic.
 - a. Proper procedure and page identification information
 - b. Statement of purpose
 - c. Job planning information
 - Verification provisions (e.g., quality control hold points, independent verification)
 - e. Clarity and specificity of instructions
 - f. Adequacy of cautionary information
 - g. Quantitative acceptance criteria
 - h. Instructions for post-maintenance and system restoration

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RIDGE NATIONAL LABORATORY

UNION CARBIDE **Quarterly Management Report**

ORNL Programs for the **NRC Office of Nuclear Regulatory Research**

A. L. Lotts

September 1982

Prepared for the U.S. Nuclear Regulatory Commission Office of Nuclear Regulatory Research Under Interagency Agreements DOE 40-543-75, 40-550-75, 40-551-75, and 40-552-75

UNION CARBIDE CORPORATION FOR THE UNITED STATES DEPARTMENT OF ENERGY

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PROGRAM DIRECTOR'S SUMMARY

Technical Highlights

The highlights of technical progress for each division during the past quarter are as follows:

Division of Engineering Technology

An analysis of risks associated with recycle of decommissioned materials from nuclear facilities was performed under the project assessing the Technology and Costs of Termination Surveys Associated with Decommissioning of Nuclear Facilities. The conclusion of this study was that the fatalities associated with use of recycled materials are more than balanced by the avoided fatalities associated with the savings in energy production. The CONDOS computer code was used to estimate direct radiation exposure from soil on decommissioned nuclear fuel cycle sites. Soil on sites contaminated to 10 mrem/yr levels as established in NUREG/CR-2241 gave the following direct exposure levels: PWR, 0.67 μ R/h; mixed oxide fuel fabrication facility, 0.07 μ R/h; UO2 fuel fabrication facility, 0.3 μ R/h; UF6 production, 0.3 μ R/h; reprocessing plant, 0.3 μ R/h; and spent fuel storage, 0.5 μ R/h.

In the Additional Requirements for Materials project, further slow-bend testing of precracked Charpy specimens from Intermediate Test Vessels 7B and 8 (from the HSST Program) has begun to define the heat-affected zone fracture toughness behavior below -100°C. Specimen testing was completed under Subtask 7 (Effect of Poor Practice During Half-Bead Weld Repair). A report on this work was completed and is in review, entitled "Effects of Off-Specification Procedures on the Mechanical Properties of Half-Bead Weld Repairs." Under Subtask 8 (Underclad Cracking) initial metallographic examination of the clad A-508 class 2 section did not reveal underclad cracks. However, a series of samples varying in depth around the overlap region of two weld passes is in progress and will provide more definitive information.

Material studies for the Heavy Section Steel Technology (HSST) project (Task 3, Thermal Shock) to evaluate the effects of irradiation on thermal conductivity were completed on irradiated samples machined from previously tested Charpy specimens of HSST Plate 02. The results clearly show that neutron irradiation does not degrade the thermal conductivity of this steel under the conditions of interest. Fracture-mechanics design calculations performed for TSE-7 and -8 indicate increases in the

KI due to the cladding of 35% at the inner surface and 15% at a depth corresponding to the cladding thickness. The difference between the KI value at the inner surface of the unclad cylinder and at the interface of the clad cylinder was only 2%. These results mean that a difference in crack behavior between TSE-7 (no cladding) and TSE-8 (clad) will be a clear indication of the effect of cladding on crack propagation.

In other tasks of the HSST project, major test parameters were determined for the first two tests (PTSE-1 and -2) of the Pressurized Thermal Shock (PTS) task. These parameters are (1) initiation of a shallow flaw in 6-8 minutes at a stress intensity level of about 110 MPa(m) 1/2 with a projected arrest on the upper shelf at a/w = 0.4 and (2) initiation of a shallow flaw (a/wabout 0.05) in two minutes and arrest at a/w = 0.15. Decreasing thermal stresses then induce warm prestressing (WPS) from about 3 to 6 minutes. WPS is overcome about 6 minutes into the transient by the application of pressure. In the task on cladding evaluation, one unclad plate, three plates clad with T308/J09 stainless steel and one plate clad with T312 stainless steel have been tested. Two of the T308/309 clad plates and the T312 plate failed prematurely during the loading cycle, apparently due to the remaining stainless steel in the flaw region, which was intended to be completely removed prior to electron-beam welding. One T308/309 clad plate which was loaded to yield and hydrogen charged fractured upon pop-in of the electron-beam weld. Results to date suggest that stainless steel cladding has a limited capacity for arresting a running flaw on the surface.

Under the project on Improved Eddy Current In-Service Inspection for Steam Generator Tubing, a sample 1.65-mm mean radius pancake coil was constructed and trained on new fretting standards that were recently machined. This coil was demonstrated at the EPRI Steam Generator Workshop in Charlotte, North Carolina.

Division of Accident Evaluation

Three tests (C-3, HI-1, C-4) were performed in the Fission Product Pelease from Fuel project. Tests C-3 and C-4 were control tests to calibrate the system under known releases. Test HI-1 was a release test with the fuel specimen held at 1400°C for 30 minutes. The dominant released materials were 85Kr, 134Cs, 137Cs, and some 125Sb. The fractional release values agreed, in general, with those previously reported. The laser-Raman facility located at the Oak Ridge Gaseous Diffusion Plant was used to detect saturated vapor above CsI crystals in the temperature range 650-800°C (vapor pressure 4x10-4 to 5x10-3 atm). Good spectra were obtained at these concentrations which should typify the released cerium species in the high temperature release

tests. A status report prepared on the studies to evaluate the feasibility of the laser-Raman technique for specie identification in our release tests concluded that the feasibility could not yet be determined.

By inference from spectrophotometric data on HOCl and HOBr, which have known absorption bands at 240 and 260 nm, respectively, HOI would be expected to absorb in the 280 nm region. In the Iodine and Tellurium Chemistry project, spectrophotometric examinations were made in kinetic experiments at progressively higher iodine concentrations (5×10^{-5} up to 10^{-4} M I₂) over a pH range of 9 to 11. These have revealed a weak shoulder at ~270 nm. The net absorbance in this region follows second order decay kinetics in support of expected HOI behavior.

In the LWR Core-Melt studies as part of the LWR Aerosol Release and Transport project, a series of experiments was started in which simulant fission product materials are added to UO2, sintered, and zircally clad for heating inductively to various temperatures up to melting. The first experiment used a sintered (1400°C) mixture of UO2 containing barium oxide (BaO), strontium oxide (SrO), and cerium oxide (CeO2). Preliminary results show low release fractions for barium and strontium (~4%) with a much smaller value for cerium. The apparent enhanced release of barium and strontium relative to cerium could be explained as resulting from a partial chemical reduction of the oxides of barium and strontium by the presence of metallic zirconfum in the melt. Release would then be as metallic elements with subsequent reoxidation. Two additional tests were conducted in this series with mixtures containing appropriate amounts of molybdenum, ruthenium, and tellurium as metal powders blended with UO2 and steel powder all sintered at 1200-1400°C and loaded into Zircaloy tubes. Almost none of the fission product additives were transported to the filters. Practically all of the aerosol material that was found on the filters came from the steel additive and from the Zircaloy tube.

In the aerosol tasks of the same project, a trial test section (a 10-inch diameter SS pipe, 8-feet high) was assembled with the plasma torch aerosol generator located at the bottom. Four Fe₂O₃ aerosol/steam experiments have been completed in the NSPP (Nos. 501-504) at increasing concentration levels (1 to 10 g/M³). Although analysis of the data is incomplete, the Fe₂O₃ aerosol (at comparable concentration levels) appeared to be removed faster than U₃O₈ at early times and more slowly at later times. Two smaller scale aerosol generation tests were also conducted using powdered concrete as the feed material to the plasma torch. One concrete sample was made up of limestone aggregate and the other was basalt. The aerosols produced were composed mainly of silicon dioxide and calcium silicate in highly agglomerated crystalline forms.

The Severe Accident Sequence Analysis project has completed and issued for review the draft report "SBLOCA Outside Containment at Browns Ferry Unit One - Accident Sequence Analysis." The report "Iodine and Noble Gas Distribution and Release Following Station Blackout at Browns Ferry Unit One" was also completed. The first volume recommends changes in operational procedures, operator training, and plant design. The second volume develops methodology for fission transport studies, and concludes that the pressure suppression pool is effective in reducing fission product release, and predicts that about 0.5% of the initial core iodine inventory would be released to the atmosphere.

The Advanced Two-Phase Instrumentation project is developing an ultrasonic sensor to measure simultaneously the level, temperature, and density of the fluid in which it is immersed. The use of a high voltage pulser for driving the magnetic-coil transducer has resulted in an order-of-magnitude increase in the signal strength. A new electronic package under design will make multiple zone measurement faster and more flexible.

Development of the ORECA code capability for HTGR severe accident sequence analysis continued with the formation of equations describing heat transfer between the upper core (reflector) nodes and the upper plenum area of the PCRV and between the core side reflector block and the PCRV sidewall area. A version of the ORECA (3-D Core model) for simulating the dynamics of the 2240 MWt steam cycle/cogeneration HTGR plant design was also completed and checked out.

The final Multirod Burst Test (B-6) bundle was sectioned, polished and photographed, and strain values were determined. The bundle exhibited uniform strains in the 15-25% range with localized burst strains in the 25-50% range. As expected for the test temperature (930°C), rod-to-rod interaction was not a significant factor. A draft of a topical report, entitled "Variations in Zircaloy-4 Cladding Deformation in Replicate LOCA Simulation Tests" was prepared and is undergoing internal peer review. The report presents results of five single rod heated shroud replicate tests that were conducted to study statistical variations in cladding deformation parameters.

Division of Risk Analysis

In the Analysis of Reliability Data From Nuclear Power Plants project, an interim letter report titled "The Reliability Characteristics of Selected Pumps in Four Nuclear Plants" was submitted to NRC for comment. The report documented failure rate calculations and presented repair information.

In the project on Acceptable Level of Risk Criteria for Nuclear Power Plants, a report was completed and submitted for review entitled "Health and Safety Standards: Theoretical Rationale and Application to Safety Goals for Nuclear Power." The report is in two parts. Part 1 is "Standard Setting Standards: A General Theory of Standards." Part 2 is "Safety Goals for Nuclear Power: An Application of the General Theory of Standards."

The Evaluation of Pressurized Thermal Shock project completed a study to investigate the importance of counter-current flow limiting behavior in the steam generator when injecting feedwater to the auxiliary header. The impact upon the cooling supplied by the auxiliary feedwater was determined to be minimal regardless of the injection point. Event trees for the more important classes of overcooling sequences were pruned (on the bases of engineering judgement and initial estimates of probability associated with certain top-line events) to yould a reduced number of possible end-states and were replotted for distribution to PTS program participants. Three specific overcooling scenarios were selected from the pruned event trees and were completely specified (initial conditions, timing of operator actions, etc.) for thermal-hydraulic calculation using the large systems codes, following approval by NRC.

Two major reports on common-cause risk analysis for 1 cods were issued under the Flood Risk Analysis Methodology project, NUREG/CR-2677, ORNL/TM-8313, "ESP and NOAH - Computer Programs for Flood Risk Analysis of Nuclear Power Plants," and NUREG/CR-2678, ORNL/TM-8314, "Flood Risk Analysis Methodology Development Project Final Report."

The LWR Accident Sequence Precursor Study project issued a major report on precursors to potential severe core damage accidents for the period 1969-1979 (NUREG/CR-2497). An interim report was drafted on the study of precursors to pressure vessel thermal shock.

Division of Facility Operations

In the project on Continuous On-Line Reactor Surveillance System Evaluations, the in-plant demonstration system performed well except when removed from service briefly for software modifications. Data were also recorded during a loss-of-feedwater-flow test at LOFT to assess the ability of noise analysis methods to detect and diagnose abnormal operating conditions in PWRs.

The Noise Diagnostic Methods for Safety Assessments project completed all flux adjoint calculations for the beginning and end of fuel cycle core conditions to help complete the assessment of ex-core neutron detector sensitivity to fuel element vibrations. At the midyear review on April 15, the RRG adviced post-ponement of the study of functional redundancy until a later date. The funds allocated for this study are being used to perform additional analysis and interpretation of noise data from LOFT and Sequoyah and to explore the use of expert system technology for automated anomaly diagnosis.

Three reports have been written for the Maintenance Error Model project. The report on the job analysis of the Maintenance Mechanic position (NUREG/CR-2570) has been completed, reviewed, and is now being prepared for publication. The report on the front-end analysis, literature review and overall program plan (NUREG/CR-2669) has been completed in draft, and is now ready for review. The report on the job analysis of the Maintenance and I&C Supervisor position (NUREG/CR-2668) has been completed and is now being reviewed prior to publication. At the mid-year review meeting on April 5, a modified program plan was presented concerning the development of a human reliability model for NPP maintenance personnel. The modified program plan proposed an initial model release (following debugging and sensitivity testing) 20 months after the start of development. This is a reduction of approximately 10 months in the originally proposed schedule. In addition, the modified program plan including model validation and transfer to users was reduced in total by about 14 months. The development phase of this program will be initiated as soon as funding is received.

As part of the project on Operational Aids for Nuclear Power Plant Operators, during site visits the subcontractor secured exclusive access and use of the simulator and experienced operators for a total of 24 operating hours. Still photographs and video tape recordings were made of operating sequences, including one emergency sequence (low coolant leading to a scram.) A report was issued entitled The Allocation of Functions in Man-Machine Systems: A Perspective and Literature Review (NUREG/CR-2623). This report reviews the literature relevant to allocation of functions and presents a procedure for the allocation process applicable to nuclear power plant control rooms.

In the Safety Implications of Control Systems project, work began on simulating the ICS on the analog computer, with the steam generator-feedwater control unit selected as the starting point. The model is being implemented wherever sufficient approximate information exists. The Rod Control and the Unit Load Development sections of the Oconee Integrated Control System were added to the analog simulation. The simulation provides the capability of changing the rate at which the load demand is allowed

to vary. Adaptation of the WIGGLE neutronics code was completed, and preparations were made to begin coupling the code to the core thermalhydraulics subroutings. FORTRAN routines and subroutines were written and debugged for thermodynamic and transport water properties, heat transfer logic and correlations, and the computation of one- and two-phase friction pressure loss coefficients. Steam generator modeling was focused on the two-phase phenomena in a single flow channel, with particular attention to transitions between single and two-phase regimes. Test runs were made of realistic steam generator flow. Models of four pumps - hotwell, booster, mainfeedwater, and the pump between flash tank and main feedwater line - were added to ORTURB.

The Safety Related Operator Actions project completed data collection during "Week 1" of the PWR requalification program, recording 83 data runs. BWR data collection for "Week 2" of requalification training was completed for five operator teams. Selection of critical task elements (CTEs) to be used in analysis of response times and error probability is nearly complete. Demographic data on operators has been reduced for statistical analysis. Data collection for the ten events in the BWR task analysis was also completed. Two reports were issued, Specification and Verification of Nuclear Power Plant Training Simulator Response Characteristics, Part II. Conclusions and Recommendations (NUREG/CR-2353) and Nuclear Power Plant Control Room Task Analysis: Pilot Study for Pressurized Water Reactors (NUREG/CR-2598).

Division of Health, Siting, and Waste Management

Camera-ready copy is being prepared for the report, "Nuclear Power Plant Operating and Maintenance Costs and Estimating Guidelines." This is part of Task 1 of the project CONCEPT/OMCOST Code Development. As part of Task 2 of this project, a draft of a literature review on cost-size scaling for nuclear and coal power generation units has been completed. The review documents that the cost-size scaling exponent for nuclear units is 0.4-0.6 and 0.7-0.9 for coal units.

The project, Forecasting Electricity Demand by State, has documented the Integrated Forecasting System in An Integrated System for Forecasting Electric Energy and Load for States and Utility Service Areas (ORNL/TM-7947).

In the project studying Internal Dose for Specific Occupational Exposure Conditions, committed dose equivalent (H50) per unit uptake and the annual limit on intake (ALI) were computed for inhalation of radioiodines in the vapor form assuming an instantaneous uptake by the transfer compartment of the total inhaled vapor activity (100%). The results show that for the shorter-lived radioiodines, the vapor form becomes increasingly more restrictive due to the prompt uptake from the lung.

The project on Pathogenic Microorganisms in Closed Cycle Cooling Systems compared air sampling devices to determine their efficacy for detecting Legionnaires' Disease Bacteria (LDB) at the point of discharge from cooling towers. No LDB could be detected in either the impinger or high-volume air samplers placed adjacent to the tower plume. Air samples with detectable LDB did not produce infection on intraperitoneal inoculation of guinea pigs, whereas obvious lung disease did occur on intranasal injection. Histologic and cultural analysis of diseased lung tissue did not indicate the presence of LDB but did show the presence of a variety of other bacteria.

The draft document prepared for the project on Uncertainties in Assessment of Long-Term Collective Dose and Health Effects from Geologic Disposal of High Level Waste entitled An Analysis of Uncertainties in Long-Term Dose and Health Effects Resulting from Geologic Disposal of High-Level Radioactive Waste, NUREG/CR-2506, ORNL-5838, has completed internal technical review. The reviews have indicated the need for extensive revisions. A letter report was transmitted to NRC giving technical responses to selected public comments on the proposed rule, 10 CFR Part 60. A set of notes on developmment of a system performance standard for high-level waste was sent to NRC to serve as a basis for discussions on June 28. The notes discussed (1) the applicability of 10 CFR 61, 10 CFR 40, and ICRP 26 to a high-level waste standard, (2) comments on Draft 20 of the EPA standard (40 CFR 191), (3) special features of high-level waste disposal that could influence performance standards, and (4) topics that must be dealt with in a standard and illustrations of alternatives. Several papers were published in technical journals.

Problem Areas

Seventy milestones were scheduled to be completed this quarter for the Office of Nuclear Regulatory Research. Of these, fifty-one were completed on schedule for a completion rate of 73%. Those that necessitated rescheduling were primarily due to decreased funding levels, delayed receipt of FY 1982 funds, or changes in work scope. Minor problem areas are delineated below.

Ten out of twelve milestones scheduled for the NRC Division of Engineering Technology were achieved on schedule for a completion rate of 83%. Two milestones in the project on Eddy-Current In-Service Inspection for Steam Generators were delayed due to shortages in personnel and personnel commitments to other programs. In Task 4 (Intermediate Vessel Test) of the HSST project, the small Balseals, used in instrumentation penetrations, continue to be a minor problem. A set survived 86 pressure cycles to 200 MPa at 150°C. A further modification of the instrumentation penetration fitting was made to reduce the likelihood of failure.

For the Division of Accident Evaluation, on April 19, NRC informed us that analysis and correlation assessment efforts were to be deleted from the Correlation and Evaluation of Bundle Heat Transfer Models project in the areas of film boiling, forced convection to steam, and void fraction distribution, and that three fourths of the program's funding for FY 1983 would be withdrawn. Sixteen of nineteen milestones were completed on schedule for a completion rate of 84% for all projects. Milestones have been delayed in two projects. In the Post-Accident Iodine and Tellurium Chemistry project, the level of work required along with unexpected technical difficulties has led to a delay in two milestones. Problems in the Aerosol Release and Transport Program such as a decreased level of funding and difficulties in obtaining a suitable power supply have caused rescheduling of several milestones. Efforts are being made to resolve these difficulties.

Seven out of fourteen milestones scheduled for the Division of Risk Analysis this quarter were completed on schedule for a completion rate of 50%. Most of the delayed milestones were because of delays in receipt of expected FY 1982 funding. In the project on Acceptable Level of Risk Criteria for Nuclear Power Plants, reports pertaining to the quantitative safety goal have been rescheduled because of late arrival of FY 1982 funds. A potential problem area in the Common Cause Failure Analysis Procedure project is the delivery of plant/PRA information necessary to initiate the required plant analyses. In the project on Utilization of Risk Analysis and Risk Criteria, Rand's subsubcontractor, Kuljian Corp., has failed to meet contractural agreements by being at least three months overdue on delivery of a draft report. This problem is being confronted by Rand and will be resolved shortly. This project also has had to reschedule four milestones because of delays in FY 1982 funding. In the project on Definition of Scenarios and Controlling Parameters for Major Accidents Involving UF6 at NRC-Licensed Fuel Cycle Facilities, relevant documents obtained so far do not provide sufficent accident scenario descriptions. Development of scenario descriptions may be difficult if sufficient details are not obtained through planned site visits.

Four milestones were scheduled to be completed this quarter for the NRC Division of Facility Operations and they were all completed as scheduled for a completion rate of 100%. There are several potential problem areas that may affect future milestones mainly due to a lack of utility cooperation in providing plant design and operating data. TVA has not agreed to extend the demonstration of the ORNL automated noise surveillance system at Sequoyah-1 because the space is needed for installation of the technical support center. NRC has sent a formal request to TVA requesting extension of the demonstration beyond the first fuel cycle. Limited access to detailed design data is delaying the completion and testing of models to be used in the program to study the Safety Implications of Control Systems. While continuing to pursue avenues for obtaining utility cooperation, ORNL and NRC staff are discussing alternative ways to perform this task if

the desired data is not obtained. This same project also experienced subcontractor problems which slowed the failure modes portion, but the problem has been resolved. The problem of obtaining BWR operator participation in assessing Safety-Related Operator Actions has also been resolved and data collection during BWR requalification programs has resumed.

There were twenty-one milestones scheduled for the Division of Health, Siting, and Waste Management this quarter and sixteen were completed on time for a completion rate of 63%. In the CONCEPT/OMCOST Code Development project two subtasks were rescheduled by agreement with NRC to allow incorporation of the results of the January 1982 cost study being performed by United Engineers & Constructors for DOE.

New Projects, Initiatives, and Proposals

In the Division of Engineering Technology, two projects are being reported on for the first time, ASME Code Section III-Technical Support and Containment Leak Rate Testing. A proposal was submitted as an outgrowth of work in the first project on recommendations for future research on the problem of preloading of bolted connections.

Several new projects, proposals, and extensions of present tasks were begun or funded in the Division of Accident Evaluation. Reported for the first time this quarter is the project on Near-Term TRAP-MELT Verification. Several rew proposals for FY 1983 were funded including Fast Reactor Aerosol Release and Transport, Clad Ballooning Evaluation, Fission Product Deposition Onto Aerosols, and Computerized Research Information.

Two new projects were started this quarter in the Division of Risk Analysis on Analysis of Proposed New IAEA Basis for Transportation Regulatory System and Definition of Scenarios and Controlling Parameters for Major Accidents Involving UF6 at NRC-Licensed Fuel Cycle Facilities. One additional new project has been funded for FY 1983, Accident Sequence Precursor Methodology. For the project on Evaluation of Pressurized Thermal Shock, the task on Probabilistic Risk Assessment was expanded to a level of ~4.5 manmonths beginning about June 1. A proposal was also submitted relative to this project on the development of a method for incorporating the uncertainties of calculated temperature—and pressure—vs—time histories into vessel failure prediction codes (Probabilistic Methodology for Pressure Vessel Failure Prediction). One other new proposal was submitted for FY 1983 for a Modal Study — Structural Testing and Technical Analysis.

New projects initiated during the third quarter in the Division of Facility Operations were: Pressure Sensor/Sensing Line System Evaluation Research and Occupational Radiological Monitoring at Uranium Mills. New proposals included Internal Dose for Specific Occupational Exposure Conditions and High Sensitivity Radionuclide Analysis for Internal Dose Assessment. As part of the Human Factors effort, new projects include Human Factors in Incident Alert Notification and Organizational Interface in Reactor Emergency Preparedness. As a partial outgrowth of the Safety Related Operator Actions Program, a proposal was submitted for FY 1983 to conduct Training Simulator Experiments.

Financial Status

The ORNL projects supported by the Office of Nuclear Regulatory Research are under satisfactory financial control as shown in the attached figures for each NRC division (ET, AE, RA, FO, and HSWM). The financial plan amounts are based on funds carried over from FY-81, the funds received in FY-82, and funds expected to be carried over to FY-83. The cumulative planned amounts include expected funds for FY-82 in addition to financial plan amounts. The funds shown in the Cost/Budget Reports reflect all authorizations through June 30, 1982. Programs which are not properly funded are discussed below.

The overall summary Cost/Budget Report shows that \$4765K was carried over from FY-81, \$17558K was received as of June 30, 1982, and \$2596K is planned carryover for FY-83. Therefore, the financial plan as of June 30, 1982 was \$19702K. With the expected funds of \$577K, the total funding allocation to ORNL in FY-82 from RES is expected to be \$20279K, which is lower than the \$20403K authorized in FY-81. This lower funding level and pressure to accelerate many tasks has caused many programs to have insufficient funds available to use as planned carryover for FY-83 to ensure program continuation if FY-83 funding authorizations are delayed. Immediate resolution of this problem is required.

Funds were received this quarter for two new projects in the Division of Engineering Technology: ASME Code Section III - Technical Support and Containment Leak Rate Testing. Remaining funds in the closed-out project on Evaluation of Performance of Greased Prestressing Tendons are being used for preparation of the final report.

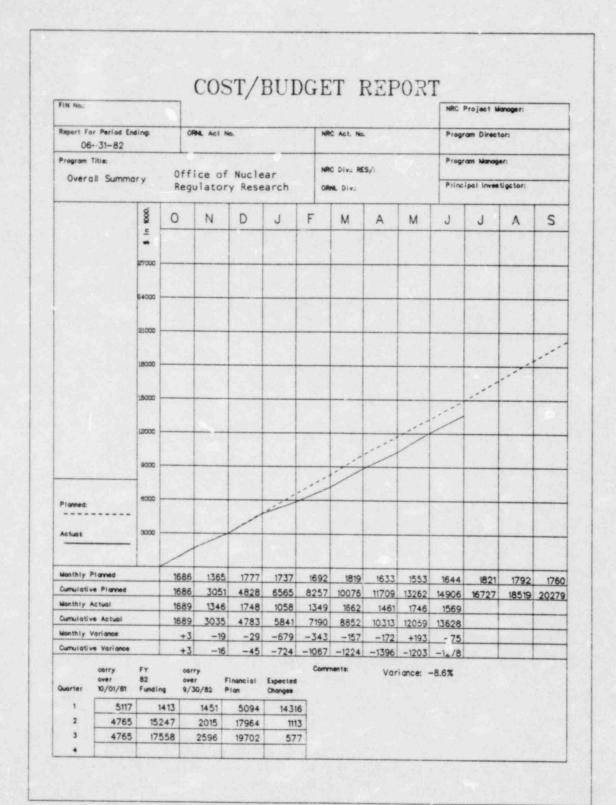
The new project in the Division of Accident Evaluation on Trap-Melt Verification tests received initial funding this quarter.

FY 1982 funds were received this quarter for two new projects in the Division of Risk Analysis: Analysis of Proposed New IAEA Basis for Transportation Regulatory System and Definition of Scenarios and Evaluation of Methodologies. In addition, expected funds for FY 1982 which had been delayed were received for the projects on Common Cause Screening Failure Analysis Procedures and Mathematical and Statistical

Problems in Risk Analysis. The delayed receipt of expected FY 1982 funding is still causing problems in the project on Utilization of Risk Analysis and Risk Criteria. While the project on LWR Systems Survey on PRA is not yet complete pending issuance of a revised report, the spending target was achieved in February and no funds have been expended for the last four months.

Two new projects in the Division of Facility Operations received their FY 1982 funding this quarter: Occupational Radiological Monitoring at Uranium Mills and Pressure Sensor/Sensing Line System Evaluation Research. The project on Maintenance Error Model finally was funded and is progressing.

In the Division of Health, Siting, and Waste Management, no new funds were received this quarter for either new or existing projects. Some additional FY 1982 funding is still expected for the projects on Evaluation of Atmospheric Dispersion Models and Forecasting Electricity Demand by States. The Methods in Dosimetry for Nuclear Regulations project was completed this quarter with no additional funds expected during FY 1982.



DIVISION OF ENGINEERING TECHNOLOGY

COST/BUDGET REPORT NRC Project Manager: FIN No.: NRC Act. No. Program Director: Report For Period Ending OFM. Act No. 06-31-82 Program Nanager: Program Title: NRC DIV. RES/: ET Office of Nuclear Summary Principal Investigator: Regulatory Research ORNE Div.: S F M A M A D N -2900 2100 1400 Actuat Monthly Plemed 328 327 599 568 544 653 505 Cumulative Planned 328 1822 2366 4645 5164 5684 6199 Monthly Actual 328 328 599 327 393 50 4000 Cumulative Actual 328 656 1582 1975 2489 -139 Monthly Variance 0 +1 0 -241 -151 Cumulative Variance -391 -530 -680 -644 -645 0 +1 +1 -240 over Finan 9/30/82 Plan over 82 10/01/81 Funding 135 744 1266 657 5589 5572 588 5830 199 2 846 5977 768 6055 846

COST/BUDGET REPORT NRC Project Manager A. Taboada 80103 Report For Period Ending: Program Director: 06-31-82 41 88 54 30 3 10 19 02 01 2 Lotts/Homan Program Wanager: Program Title: NRC DIV.: RES/: ET R. K. Nanstad Principal Investigator: Additional Requirements For Materials ORNE DIV. M&C R. W. McClung 0 D A S N A = (Sc & Tech) man this 2.6 man 1.7 Manpower (Other Direct) Salaries (So & Tech) 16730 133031 Material & Services 8227 ADP Support 13 -407 Subcontracts 797 2638 Travel 43993 5792 Overhead 23925 187499 Total Costs Actuat Monthly Planned 26 27 20 20 20 20 20 20 20 19 19 Cumulative Planned 40 67 87 107 127 147 167 187 207 226 245 Monthly Actual 26 27 26 12 21 18 20 24 Cumulative Actual 67 93 105 126 144 164 188 Monthly Variance +6 -8 +1 0 +4 Cumulative Variance 0 0 +6 Comments: Variance: +0.5% carry cver 9/30/82 Financial Expected over 10/01/81 Funding Plan Changes 113 28 38 103 142 113 198 66 245 0 113 198 66 245 0

TITLE: ADDITIONAL REQUIREMENTS FOR MATERIALS FIN No.: B0103
PROJECT MANAGER: R. K. NANSTAD

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Submit final report on ferrite content in austenitic weld metal		1							
Radiography, ultrasonics, materials, and welder qualification review (as required)									
Complete report on the effect of poor practice during half-bead weld repair		1							
Underclad cracking a. Complete literature survey report b. Submit final report on test results			A .						
HAZ transformations a. Completion of draft report b. Submit final report									
Effect of aging at 500-600°F on toughness		8	₹	S					
of stainless steel weld metal a. Complete literature search									
b. Complete long-term againg tropos ",	_	_	-						

of stainless steel weld metal
a. Complete literature search
b. Complete long-term aging (10,000 h)
of materials
c. Complete testing and submit final report
12. Effect of irradiation on Charpy V-notch
upper-shelf energy - literature review

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11.

13. Review of materials for Code applications (as required)

UCN-14127 (3 9-80)

COST/BUDGET REPORT NRC Project Manager: E. T. Baker B0474 Program Director: Report For Period Ending: NRC Act. No. 60 19 21 00 Lotts/Homan 41 88 55 05 1 06-31-82 Program Manager: Program Title: MRC DIV. RES/: ET G. T. Yahr ASME Crise Section III—Technical Support OPM DIVE ETD S. E. Moore S A Α M 1.1 N = Year to Manpower man (Sc & Tech) months Manpower (Other Direct) Sciaries (Sc & Tech) 14354 56949 Material & Services 3666 147 438 1999 1999 Subcontracts 2511 -46 4809 19119 Overhead 21866 84687 Total Costs Planned Actuat Monthly Planned 27 18 Cumulative Planned 0 0 0 0 0 27 45 63 87 110 133 157 Monthly Actual 0 0 0 0 27 17 18 22 Cumulative Actual 0 0 44 84 Monthly Variance 0 Cumulative Variance 0 0 0 0 0 -3 Comments Variance: -3.4% carry carry over Financial 9/30/82 Plan 10/01/81 Funding Changes 0 0 2 0 0 0 0 0 0 157 0 200 43

ASME CODE SECTION III — TECHNICAL SUPPORT

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TASK 1: DYNAMIC ALLOWABLE STRESSES		I														
 Issue report on combined experimental/ analytical investigation of allowable stress criteria for dynamic loads (Univ. of Akron/PVRC) 									1	T						
TASK 2: PIPING SUPPORT REACTIONS		1							8/	82			1			16.5
a. Complete draft report that identifies the sources of uncertainties associated with piping support restraint loads and quantifies them to the extent feasible (E. C. Rodabaugh Associates, Inc.)								7	0	8/8	271 171	ayed : done	so tha	: Tasi	: 3 c	uld
 Provide recommendations for future research to NRC with cost and time schedule 		1	1					+	Y	8,	82					
c. Publish report that identifies the sources of uncertainties associated with piping support restraint loads and quantifies them to the extent feasible								+	4	4						
TASK 3: FATIGUE EVALUATION FOR CLASS 2 AND 3 PIPING COMPONENTS								B	/82							
a. Complete draft report that describes the basis for the Code Class 2 and 3 fatigue evaluation method, shows how it correlates with Class 1 method, and recommends Code changes that will bring the two methods into better agreement (E. C. Rodabaugh Associates, Inc.)								1	Z		То	be do	ne bef	ore T	ask 2	

ASME CODE SECTION III — TECHNICAL SUPPORT (80474)

the Code Class 2 and 3 fatigue evaluation method, shows how it correlates with Class 1 method, and recommends Code changes that will bring the two methods into better agreement c. Present recommendations to appropriate Code committee TASK 5: PRELOADING OF BOLTED CONNECTIONS a. Provide recommendations for future research and cost/time schedule TASK 7: EVALUATION OF SECTION III ACCEPTANCE STANDARDS AND FATIGUE CURVES USING A FRACTURE MECHANICS APPROACH	4	84	85	86	87	88	FY 88
PIPING COMPONENTS (cont'd) b. Publish report that describes the basis for the Code Class 2 and 3 fatigue evaluation method, shows how it correlates with Class 1 method, and recommends Code changes that will bring the two methods into better agreement c. Present recommendations to appropriate Code committee TASK 5: PRELOADING OF BOLTED CONNECTIONS a. Provide recommendations for future research and cost/time schedule TASK 7: EVALUATION OF SECTION III ACCEPTANCE STANDARDS AND FATIGUE CURVES USING A FRACTURE MECHANICS APPROACH a. Review Section III fatigue rules, fatigue				1			-
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a. Provide recommendations for future research and cost/time schedule TASK 7: EVALUATION OF SECTION III ACCEPTANCE STANDARDS AND FATIGUE CURVES USING A FRACTURE MECHANICS APPROACH a. Review Section III fatigue rules, fatigue	24	4					
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TASK 7: EVALUATION OF SECTION III ACCEPTANCE STANDARDS AND FATIGUE CURVES USING A FRACTURE MECHANICS APPROACH a. Review Section III fatigue rules, fatigue							
a. Review Section III fatigue rules, fatigue							
ing histograms							
b. Submit coposal to NRC to evaluate the con- servatism of Section 1.1 fatigue criteria							

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.ITLE: Containment Leak Rate Testing ACTIVITY NO.: 41 89 55 13

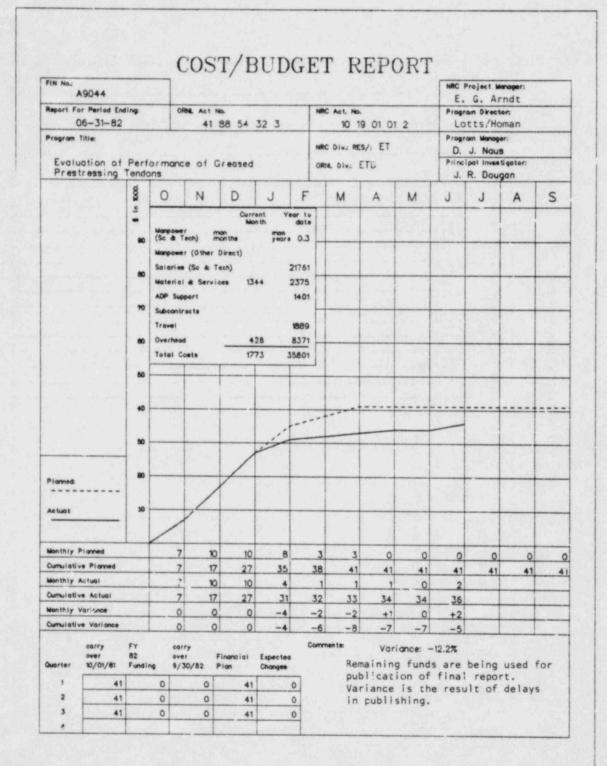
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1.	Task	A:	Review Existing Containment Leak Rate Testing Requirements.																
			a. Complete review of relevant reports.			7	7												
			 Complete witnessing of typical tests. 			٠	D												
			 Complete interviews with interested parties. 			1	4												
2.	Task	B:	Review Proposed Appendix J Revisions.	1												1			
			a. Complete review of proposed revision to Appendix J.				7	7						-					
			 Complete commentary on proposed revision to Appendix J. 				-	4				1							
			 Complete recommendations on specific aspects of the proposed revision to Appendix J. 				,	7				1							
3.	Task	C:	Review ANSI/ANS 56.8-1981 for compatability with Appendix J.									1				1			
			a. Complete review with respect to potential conflicts with Appendix J.				-	7					-						
			b. Complete determination of items in Appendix J which should be				+	V											

2

TITLE: Containment Leak Rate Testing ACTIVITY NO.: 41 89 55 13

4. Task D: Provide Value-Impact Analysis for Appendix J proposed revision. a. Complete development of an initial value-impact statement. b. Complete revision of value-impact statement. 5. Task E: Provide Final Report. a. Complete development of draft final report.	Task D: Prov Appe a. b. Task E: Prov	Value—Impact Analysis for J proposed revision. lete development of an ial value-impact statement. lete revision of value-ict statement.					
a. Complete development of an initial value-impact statement. b. Complete revision of value-impact statement. Task E: Provide Final Report. a. Complete development of draft final report.	>	lete development of an ial value-impact statement. Hete revision of value- ict statement.					
D. Complete revision of value- impact statement. Task E: Provide Final Report. a. Complete development of draft final report.	>	olete revision of value-	Α 9				
Task E: Provide Final Report. a. Complete development of draft final report.	>		٩				
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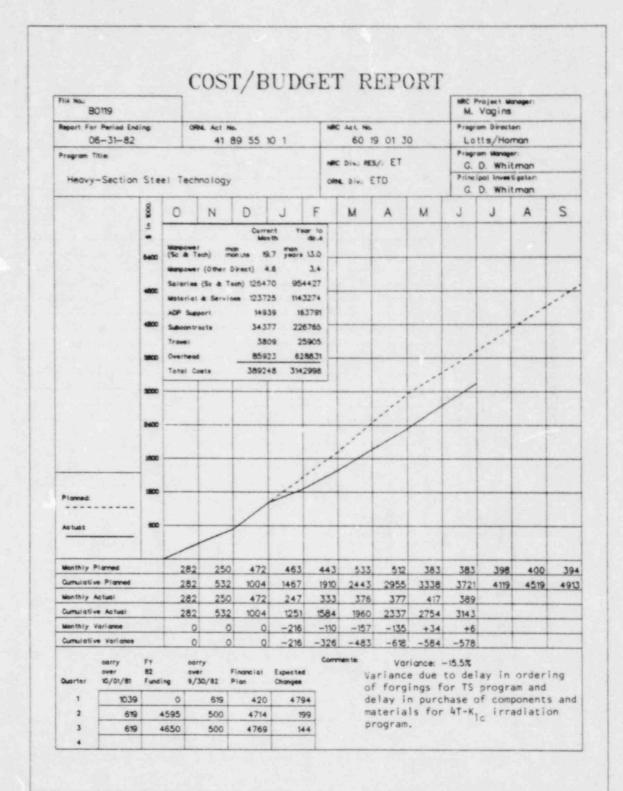


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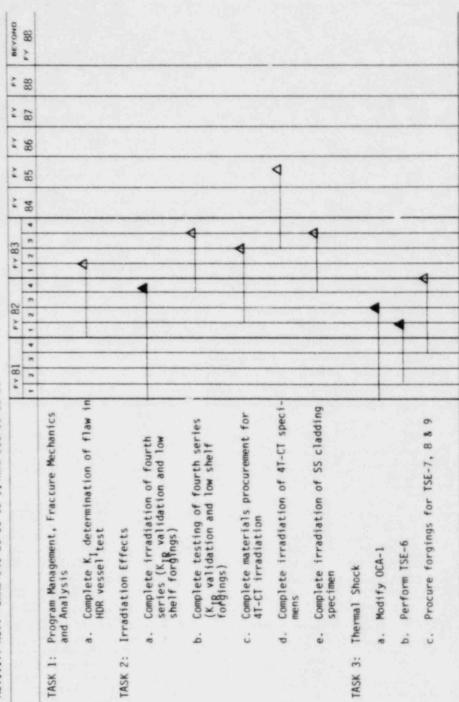
IIILE: Evaluation of Performance of Greased Prestressing Tendons in Nuclear Power Plant Structures ACTIVITY NO. 41 88 54 32 3

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	1 2 3 4 1 2 3 4 1 2 3	•					2
. Subtask C: Complete draft final report.	1						
Publish final report.	7						
				I			

UCN-14127 (3 9-80)



TITLE: Heavy-Section Steel Technology Program ACTIVITY NO.: ORNL #41 89 55 10 1; NRC #60 19 01 30



UCN-14117 (3 9-80)

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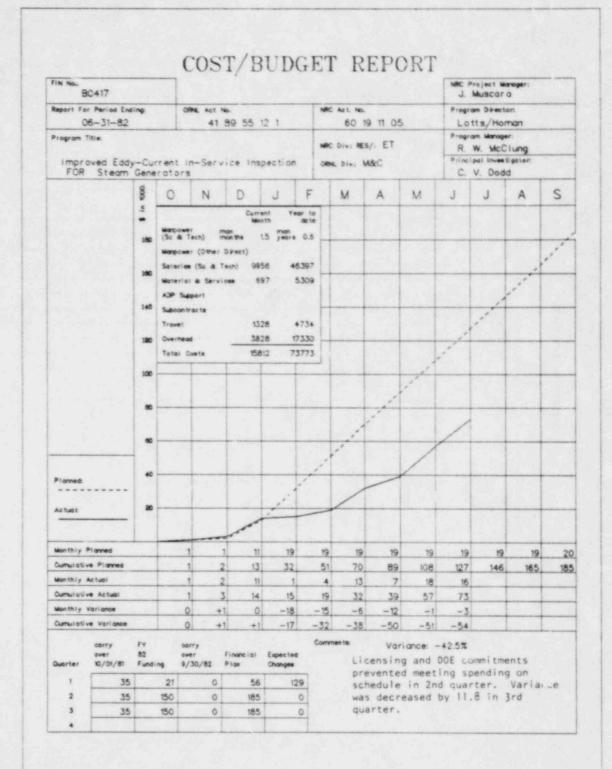
TITLE: Heavy-Section Steel Technology Program

		FY	8	1	1	4 8	2		F 1	v 1	83		FY	FY	6.4	FY	FY	BEYONE
		1	2	3 4		2	3	4	1	2	3	4	84	85	86	87	88	FY 88
TASK 4:	Intermediate Vessel Testing		1															
	a. Complete fabrication of ITV-8A	1	1	1	4											H		134
	b. Complete ITV-8A	H	1	1		-	-	Δ									h	
TASK 5:	Pressurized Thermal Shock		1							7			+					
	 Complete feasibility study and system design 		1	+	+	H	-	1				Ī						
	b. Complete facility construction		1	1			E	П	-	-	4	4						
	c. Complete PTS-1		1	1				П				5	7					
	d. Complete PTS-2		1		1	ŀ	ı						V		6			
ASK 6:	Fracture Methodology	1	1	1											F .		1	
	 Issue draft report on small specimen fracture testing in cleavage regime 		1		1	-			4			i						
	b. Complete testing and report on ${\rm PCC}_{_{\boldsymbol{V}}}$ Round Robin			1	1	+	-					4	4					
ISK 7:	Crack Arrest		1															
	a. Complete Round Robin Testing	1	+	+	+	-	-	H	4				Δ					
ISK 8:	Cladding Evaluations		1	1	-													
	a. Complete first test series		1		1	1			4			- 1	100		1	1		

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TITLE: Heavy-Section Steel Technology Program

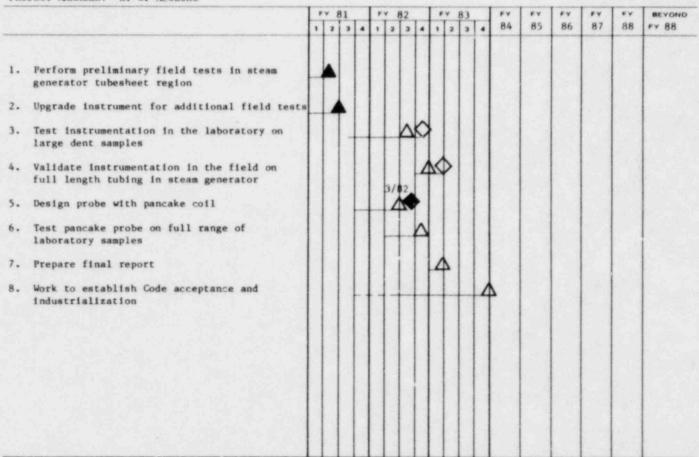
	1 2 3 4	82	1 2 3 4	84	85 86	87	88	rv 88
TASK 9: Environmentally Assisted Crack Growth								
a. Complete orientation effects at high R ratio				A				



TITLE: EDDY-CURRENT IN-SERVICE INSPECTION FOR STEAM GENERATORS

FIN No.: BO417

PROJECT MANAGER: R. W. McCLUNG



UCN 14127 (3 9-80)

8

COST/BUDGET REPORT FIN NO. NRC Project Manager C. Z. Serpan B0415 Program Director: Report For Period Ending: NRC Act. No. ORNE Act No. 06-31-82 41 89 55 12 0 60 19 21 Lotts/ Homan Program Manager Program Title: NAC DIV. RES/E ET F. B. K. Kam Principal investigator Light Water Reactor Pressure Vessel (LWR-PV) knadiation DRM. Div: OP Kam/Maerker 1000 F S 0 D A N A M Current Month 1.5 years Manpower (Other Direct) 0.2 0.2 Spieries (Sc & Tech) 10443 Material & Services 23062 3728 12453 -1282 11274 587 6355 11485 93952 Total Costs 48027 414149 Actuat Monthly Planned 24 39 79 45 45 45 49 49 49 49 49 47 Cumulative Planned 24 63 142 187 232 375 277 326 424 473 522 569 Monthly Actual 24 39 79 34 39 48 55 48 48 Cumulative Actual 24 63 142 176 215 263 318 366 414 Monthly Variance 0 -6 +3 +6 Cumulative Variance -17 -14 -8 -9 -10 Comments: carry Variance: -2.4% over 10/01/81 over 9/30/82 82 Financial Expected Funding Pian Changes 22 77 0 99 473 2 22 569 22 569 0 3 22 569 22 569 0

Title: Light Water Reactor Pressure Vessel (LWR-PV) Irradiation Program (B0415)

		rv 81		FY 82		> 4	83	*	4.4	**	44	**	BEYOND
		~	*	-	•	2 -	-	* 84	85				2
BTASK	BETASK A. LWR-PV BENCHMARK FACILITIES												
A-1.	Dosimetry Benchmark Facility (PCA)					-							
ė.	Final neutron flux characterization experiments for "Blind Test" completed.												
A-2.	Metallurgical Benchmark Facility (PSF)												
	Complete fabrication and check out of Second Simulated Surveillance Capsule (SSC-2).												
Ъ,	Start irradiation of SSC-2.					-						Ĺ	
ů	Complete irradiation of SSC-2, simulated pressure vessel capsule (SPVC), and void box capsule (VBC).												
d.	Remove, decapsulate, and ship dosimetry capsules and metallurical specimens from SSC-2, SPVC, and VBC.				4								
A-3.	Surveillance Dosimetry Measurement Facility								3				
eg.	Characterization				Þ								
ن	B6W surveillance capsule mockup experi- ment and four vendors and two service laboratories certification tests at ORR*				b								
*	*Supplemental work statement for FY 1982					-							

UCN 14127 (3 9-80)

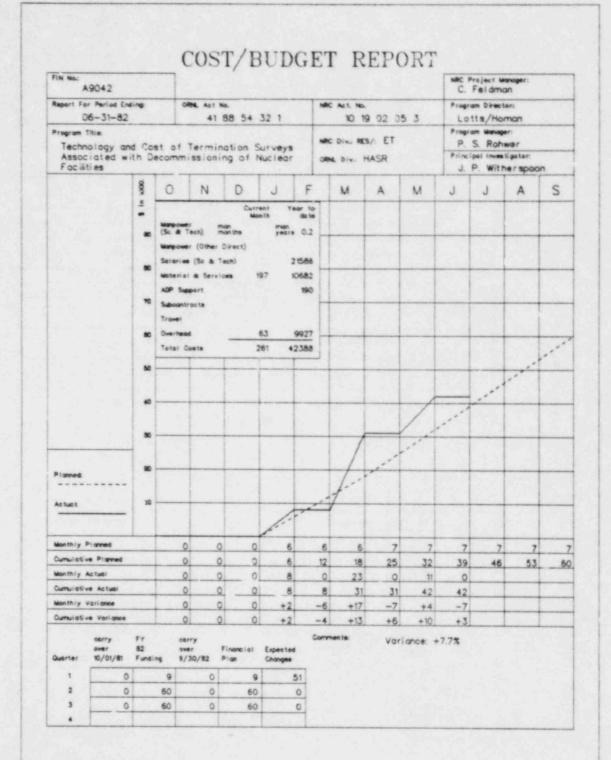
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SUBTASK	B. INSTRUMENTED IRRADIATION CAPSULES (IIC) AND PROCESS CONTROL SYSTEM (PCS)																		
a.	Operation and maintenance of PCS until end of irradiation.							*				-							
SUBTASK	C. TRANSPORT THEORY, DOSIMETRY, ASTM STANDARDS, AND DAMAGE CORRELATION																		
C-1.	Transport Theory																		
a.	Complete neutron transport calculations of PSF perturbation experiment for Westinghouse/HEDL surveillance capsule.		A LE IN		*														
b.	Complete neutron transport theory cal- culations for PSF capsules.				*												H		
c.	Documentation of calculational and experimental results for neutron exposure parameters in PSF.							•											
đ.	Coupled neutron-gamma calculations for PCA.								V										
e.	Calculate source distribution for VENUS "core source benchmark."										H	1							
f.	Calculate source distribution and neutron fluxes for two-year metallurgical experiment and "Blind Test," and B&W surveillance capsule mockup experiment and four vendors and two service laboratories certification test.												4						

SS 43		The second control of the second	FY 81	-	FY 82	>	. 83			*	*	-	*	BEYOND
Calculate neutron fluxes outside VENUS coupled neutron-gamma calculations for VENUS. Dosimetry Analysis Analysis and documentation of PCA "Blind Test" results. Analysis and documentation of PSF experiments and "Blind Test" - SSC-1 and SSC-2. Analysis and documentation of PSF experiments and "Blind Test" - PV capsule and void box capsule. ASTM Standards Complete draft of Neutron Transport Methods Guide to E10.05. Complete draft of Meutron Spectrum Ad- justment Methods Guide to ASTM E10.05. Complete draft of ocument for assess- ment and propagation of uncertainties for 17 LMR-PV Surveillance Standards to ASTM E10.05. Review above drafts as necessary to ob- tain acceptance as Standards. Input to other ASTM LMR-PV Standards to obtain acceptance as Standards.			*		*	-	2		780	82				
Dosimetry Analysis Analysis and documentation of PCA "Biind Test" results. Analysis and documentation of PSF experiments and "Blind Test" - SSC-1 and SSC-2. Analysis and documentation of PSF experiments and "Blind Test" - PV capsule and void box capsule. ASTM Standards Complete draft of Neutron Transport Methods Guide to E10.05. Complete draft of Neutron Spectrum Ad- justment Methods Guide to ASTM E10.05. Complete draft of document for assess- ment and propagation of uncertainties for 17 LWR-PV Surveillance Standards to ASTM E10.05. Review above drafts as necessary to ob- tain acceptance as Standards. Input to other ASTM LWR-FV Standards to obtain acceptance as Standards.	66	Calculate neutron fluxes outside VENUS core to determine azimuthal lead factors coupled neutron-gamma calculations for VENUS.							Þ					
Analysis and documentation of PCA "Blind Test" results. Analysis and documentation of PSF experiments and "Blind Test" - SSC-1 and SSC-2. Analysis and documentation of PSF experiments and "Blind Test" - PV capsule and void box capsule. ASTM Standards Complete draft of Neutron Transport Methods Guide to E10.05. Complete draft of Neutron Spectrum Adjustment Methods Guide to ASTM E10.05. Complete draft of document for assessment and propagation of uncertainties for 17 LMR-PV Surveillance Standards to ASTM E10.05. Review above drafts as necessary to obtain acceptance as Standards. Input to other ASTM LMR-PV Standards to obtain acceptance as Standards.	2.													
Analysis and documentation of PSF experiments and "Bild Test" - SSC-1 and SSC-2. Analysis and documentation of PSF experiments and "Bild Test" - PV capsule and void box capsule. ASTM Standards Complete draft of Neutron Transport Methods Guide to E10.05. Complete draft of Neutron Spectrum Ad- justment Methods Guide to ASTM E10.05. Complete draft of document for assessment and propagation of uncertainties for 17 LMR-PV Surveillance Standards to ASTM E10.05. Review above drafts as necessary to obtain acceptance as Standards. Input to other ASTM LMR-FV Standards to obtain acceptance as Standards.	100	Analysis and documentation of PCA "Blind Test" results.												
Analysis and documentation of PSP experiments and "Blind Test" - PV capsule and void box capsule. ASTM Standards Complete draft of Neutron Transport Methods Guide to E10.05. Complete draft of Neutron Spectrum Adjustment Methods Guide to ASTM E10.05. Complete draft of document for assessment and propagation of uncertainties for 17 LWR-PV Surveillance Standards to ASTM E10.05. Review above drafts as necessary to obtain acceptance as Standards. Input to other ASTM LWR-FV Standards to obtain acceptance as Standards.	· 0							D						
ASTM Standards Complete draft of Neutron Transport Methods Guide to E10.05. Complete draft of Neutron Spectrum Adjustment Methods Guide to ASTM E10.05. Complete draft of document for assessment and propagation of uncertainties for 17 LWR-PV Surveillance Standards to ASTM E10.05. Review above drafts as necessary to obtain acceptance as Standards. Input to other ASTM LWR-FV Standards to obtain acceptance as Standards.	·								D	7.5				11
Complete draft of Neutron Transport Methods Guide to E10.05. Complete draft of Neutron Spectrum Ad- justment Methods Guide to ASTM E10.05. Complete draft of document for assessment and propagation of uncertainties for 17 LWR-PV Surveillance Standards to ASTM E10.05. Review above drafts as necessary to ob- tain acceptance as Standards. Input to other ASTM LWR-FV Standards to obtain acceptance as Standards.				_										
Complete draft of Neutron Spectrum Ad- justment Methods Guide to ASTM E10.05. Complete draft of document for assessment and propagation of uncertainties for 17 LWR-PV Surveillance Standards to ASTM E10.05. Review above drafts as necessary to obtain acceptance as Standards. Input to other ASTM LWR-FV Standards to obtain acceptance as Standards.	ė		•											
Complete draft of document for assess— ment and propagation of uncertainties for 17 LWR-PV Surveillance Standards to ASTM E10.05. Review above drafts as necessary to ob- tain acceptance as Standards. Input to other ASTM LWR-FV Standards to obtain acceptance as Standards.	ė													
Review above drafts as necessary to ob- tain acceptance as Standards. Input to other ASTM LWR-FV Standards to obtain acceptance as Standards.	i													
Input to other ASTM LWR-FV Standards to obtain acceptance as Standards.	P					4								
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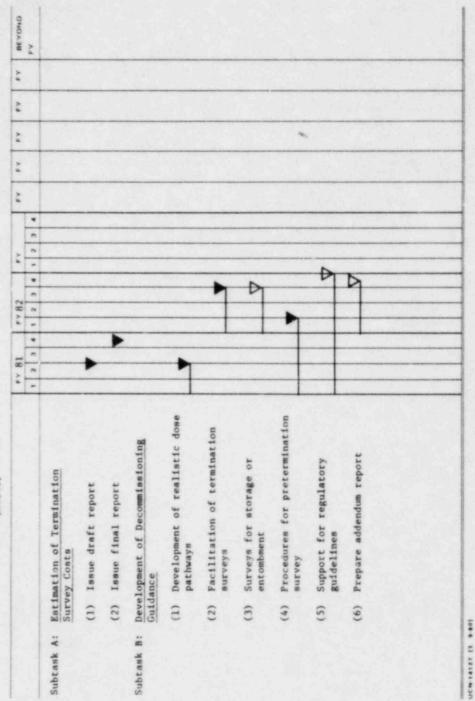
UCN-14127 (3 9-80)

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		8	-	6 2	-	1 2 3	* 84	-				**
C-4-	Damage Correlation											
ń	Statistical analysis of PSF metallurgical test data. Method estimates dependency of ART _{NDT} and shift of upper-shelf energy						Q				i, ti	
	on fluence, irradiation temperature, chemical composition, and other parameters.											
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COST/BUDGET REPORT NRC Project Manager: E. O. Woolridge 80724 Program Directors Report For Period Ending ORNE Act No. 10 19 02 01 3 Lotts/Homan 41 88 54 30 5 06-31-82 Program Manager: Program Title: NRC DIV. RES/: ET R. K. Nanstad Principal Investigator: Review of Materials for Code Applications ORNE DIV. M&C R. K. Nonstad 8000 S D F М A 0 A Manpower man (Sc & Tech) mon the years 0.1 Marpower (Other Direct) Salaries (Sc & Tech) 11794 Material & Services ADP Support Travel Total Costs 16575 40 Planned As lust Monthly Planned 0 0 0 Cumulative Planned 16 16 16 16 16 0 8 16 16 16 0 0 Monthly Actual 0 4 5 17 Cumulative Actual 12 17 17 17 0 8 0 Monthly Variance 0 0 +5 0 0 0 0 -4 Cumulative Variance 0 0 0 0 +1 +1 +1 -4 Variance: +6.3% carry Financial over Finan 9/30/82 Plan 10/01/81 16 0 0 16 0 2 0 16 0 0 16 3 16 0 16 0 0



TITLE: Technology and Costs of Termination Surveys Associated with Decommissioning of Nuclear Facilities (A9042)



DIVISION OF ACCIDENT EVALUATION

COST/BUDGET REPORT NRC Project Manager Report For Period Ending: ORNE ACT No. NRC Act. No. Program Director: 06-31-82 Program Manager: Program Tities NRC DIV. RES/: AE Office of Nuclear Summary Principal Investigator: Regulatory Research ORNE Div. 1000 0 0 F S N A A 5300 4900 700 Actust Monthly Planned 569 464 544 498 459 450 473 476 559 577 Cumulative Planned 851 1315 1959 2457 2916 3366 3839 4315 4874 5443 6020 8609 Monthly Actual 484 544 316 453 481 403 445 379 Cumulative Actual 852 380 2296 2749 3230 3633 4078 4457 Monthly Variance 0 -182 -6 +31 -70 -31 -180 Cumulative Variance -161 -167 -136 -206 -237 -417 +1 +21 +21 Comments: coury Variance: -8.6% over Financial Expected 9/30/82 Plan Changes 10/01/81 Funding Changes 1863 613 269 2222 3855 1920 4366 348 5938 99 1920 5049 348 6576 33

COST/BUDGET REPORT NRC Project Manager: Y. Y. Hau B0413 Report For Period Ending: Program Directors ORNE Act No. NRC Act. No. 06-31-82 41 89 55 11 8 60 19 10 01 Lotts/Kress Program Manager: Program Title NRC DIV. RES/: AE M. B. Herskovitz Advanced Instrumentation for Reflood Studies OFFIL DIV. I&C J. O. Hylton 1000 S 0 N D A (Sc & Tech) months 3.2 man 3.4 Waspower (Other Direct) 0.9 15 Salaries (Sc & Tech) 15156 210581 Material & Services 27286 281277 ADP Support 326 Subcontracts 3226 23852 Travel 6178 39510 15625 161149 Total Coats 68475 716700 -----1000 Planned: 600 Actuat Monthly Planned 191 110 127 Cumulative Planned 191 301 428 570 650 730 810 890 995 1205 1310 Monthly Actual 191 127 110 -17 54 29 63 91 68 Cumulative Actual 191 301 557 428 411 465 494 648 716 Monthly Variance 0 -159 0 0 -26 -51 -17 +11 -37 Cumulative Variance 0 0 0 -159 -185 -236 -253 -242 -279 carry Variance: -28.0% Financial Expected 82 10/01/81 9/30/82 Plan Variance due to delay in Changes procurement of instrumentation. 310 0 310 900 2 310 900 1210 0 3 310 1000 0 1310 0

80413 ADVANCED INSTRUMENTATION FOR PWR REFLOOD STUDIES ACTIVITY NO.: 60 19 10 01

Subtask 1.1 Support (a) Complete Software (b) Participate in Shakedown Test (c) Analyze Data of Shakedown Test (d) Provide Field Support (as required)	KL-II Instrumentation	1 2 3 4	3 4 85	86	87	88	BEYOND
in Shakedown Test a of Shakedown Test Id Support (as required)							
Participate in Shakedown Test Analyze Data of Shakedown Test Provide Field Support (as required)	1.1 Support			ij,			
Participate in Shakedown Test Analyze Data of Shakedown Test Provide Field Support (as required)	Complete Software						
Analyze Data of Shakedown Test Provide Field Support (as required)	Participate in Shakedown Test						
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ADVANCED INSTRUMENTATION FOR PWR REFLAOD STUDIES ACTIVITY NO.: 60 19 10 01

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11.	Task	Task 2 UPTF Instrumentation											
	Subt	Subtask 2.1 DP Measurement											
	(a)	Complete Design DP Measurement Block	4				-						
	(P)	Fabrication		1	D	1							
	(c)	DP Electronics Fabrication		+	4	4							
	(P)	Purge System Fabrication		+	8	_							
	(e)	System Installed in FRC			_	I	P						
	(£)	Documentation		+	4								
	Subt	Subtask 2.2 Tie Plate Drag Body Measurement								ı			
	(a)	Design (Type 1 & 2)	9	_									
	(9)	End Box at ORNL	-										
	(c)	Fabrication of Sensors	I	+	P								
	(p)	Design of Electronics	1				-						
	(e)	Electronics Fabrication	1	1	4								
	(£)	System Installed in FRG				1	*						
	(g)	Mass Flow Software	1	1			1	_					
	(P)	Documentation	1	+	4			1					

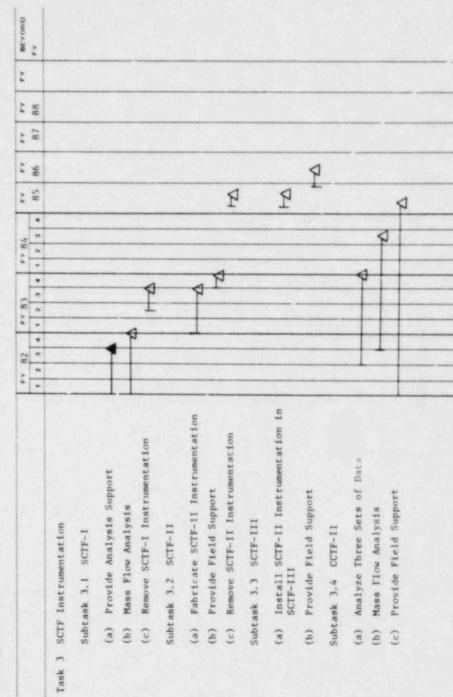
UCN 14127 (3 9-80)

ADVANCED INSTRUMENTATION FOR PWR REFLOOD STUDIES
ACTIVITY NO.: 60 19 10 01 B0413

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Pabrication Electronics Fab (Design to 2.2(d) Above) System Installed in FRG Documentation Mass Flow Software	Fabrication Electronics Fab (Design to 2.2(d) Above) System Installed in FRG Documentation Mass Flow Software	Jedsk 2.3 meanings mangin transcript												
Fabrication Electronics Fab (Design to 2.2(d) Above) System Installed in FRG Documentation Mass Flow Softwarz	Fabrication Electronics Fab (Design to 2.2(d) Above) System Installed in FRG Documentation Mass Flow Software	(a) Design	<u> </u>	9										
System Installed in FRG Documentation Mass Flow Software	System Installed in FRG Documentation Mass Flow Software	(b) Fabrication		_	+	+	4							
System Installed in FRG Documentation Mass Flow Softwarz	System Installed in FRG Documentation Mass Flow Software	(c) Electronics Fab (Design to 2.2(d) A	(bove)	_	7	f	4		_					
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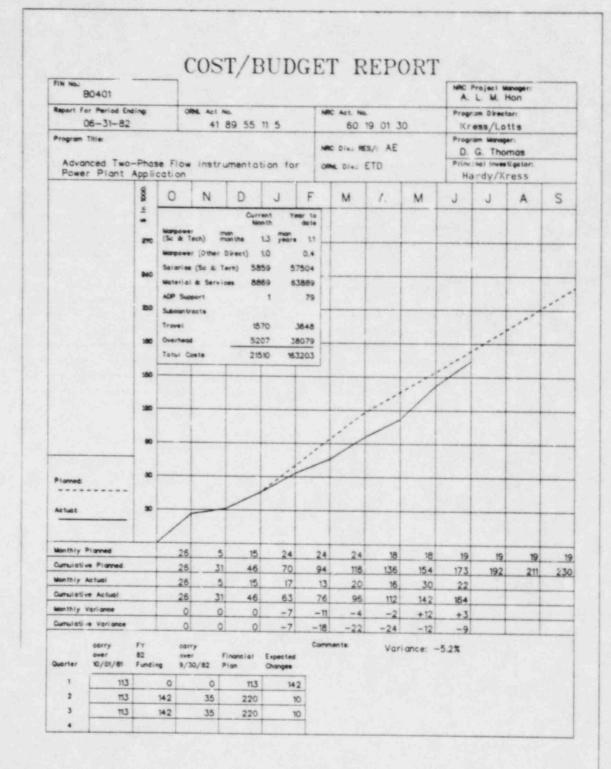
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UCM 14127 (3 9-40)

ADVANCED INSTRUMENTATION FOR PWR REFLOOD STUDIES ACTIVITY NO.: 60 19 10 01 B0413

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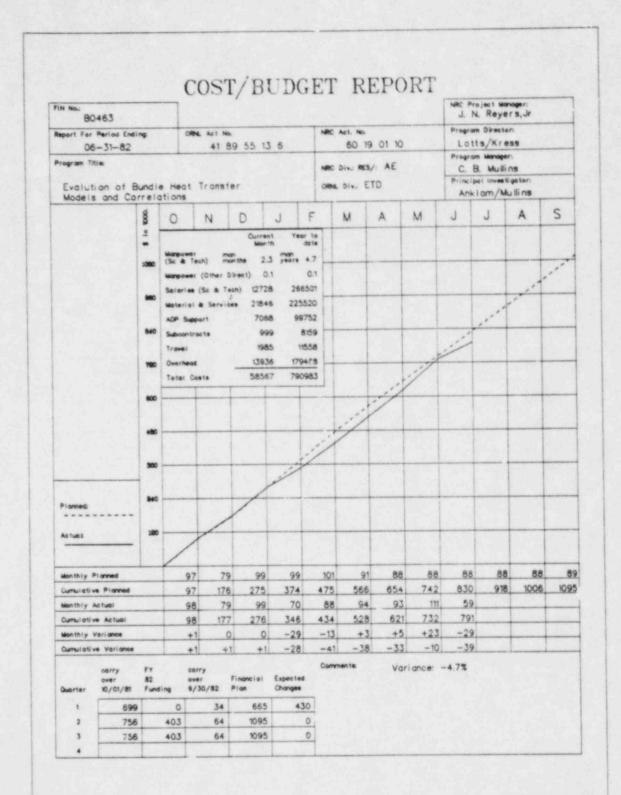
Title: Advanced Two Phase Flow Instrumentation for Power Plant Applications Activity No.: 40 89 55 11 5 80 401

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Complete report on INEL Tests of Westinghouse DP system.	4									
Turbine Meter Calibration.	1	V		_						
Final report on HJTC studies.	1	4								
Optimum specification for ultrasonic probe.		4								
Ultrasonic probe fabricated, tested, calibrated.		4								
Final report on ultrasonic probe development.		4								

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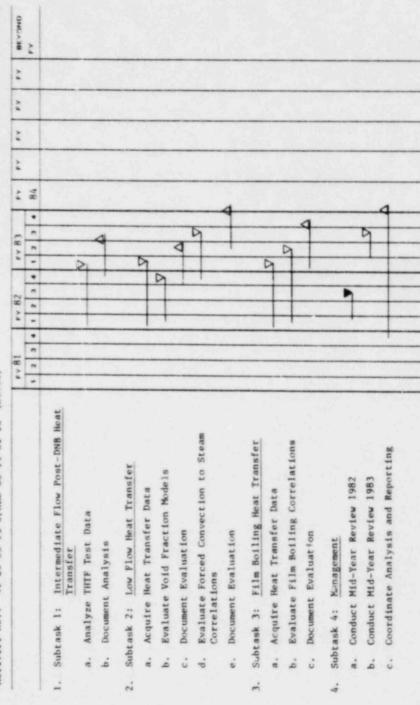
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UCM 14127 [3 9 80]



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TITLE: Evaluation of Bundle Heat Transfer Models and Correlations ACTIVITY NO.: 41 89 55 13 6/NRC 60 19 01 10 (80463)

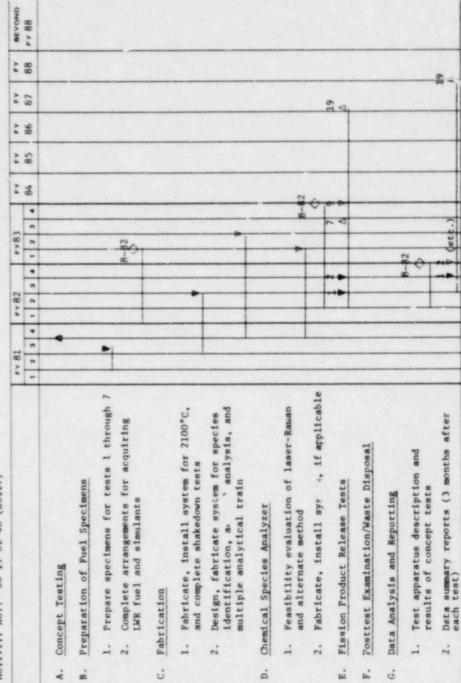


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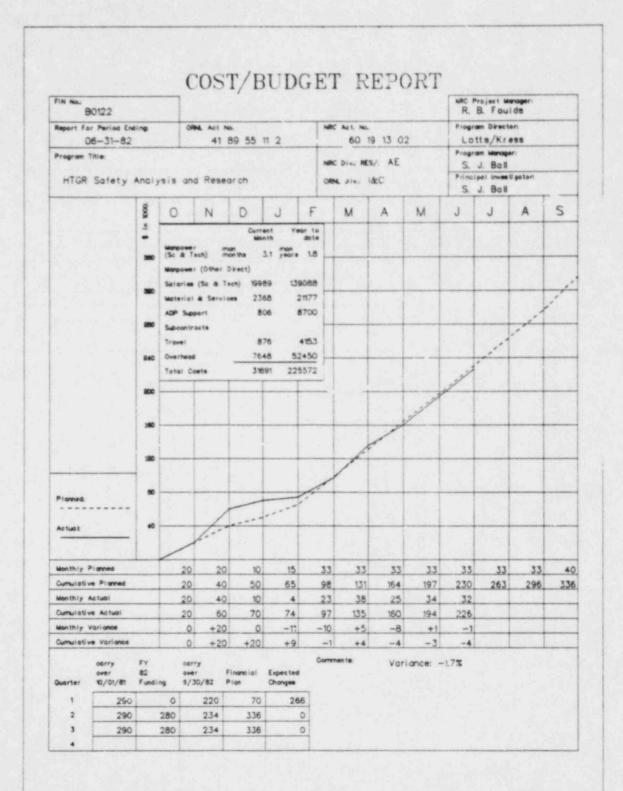
COST/BUDGET REPORT FIN No. R. R. Sherry 80127 Report For Period Ending Program Directors ORNE Act No. 06-31-82 41 89 55 10 8 60 19 13 03 Kress/Lotts Program Mignager: Program Title: NRC DIV. RES/: AE R. P. Wichner Principal Investigator: M. F. Osborne Fission Product Release from LWR Fuel ORNE DIV. CTD F 0 N D A M A S Current Manpower man (Sc & Tech) months 4.1 years 2.9 Manpower (Other Direct) 1.6 18 Salaries (Sc & Tech) 19291 162840 9362 Moterial & Services 208699 5819 Travel 819 Overhead 9154 113100 Total Costs 37809 491281 Planned: Actuet Monthly Planned 51 51 51 51 51 69 69 68 68 Cumulative Planned 83 145 200 251 302 404 455 524 593 661 729 Monthly Actual 83 55 56 49 54 47 47 38 Cumulative Actual 83 145 200 256 ٥٥5 359 406 453 491 Monthly Variance 0 +5 +3 -31 -4 Cumulative Variance 0 0 0 +5 +3 +6 +2 -2 Comments: Variance: -6.3% corry Financial Expected 10/01/81 Funding 9/30/82 Plan Changes 29 143 0 172 487 0 2 29 628 31 3 29 700 0 729 0

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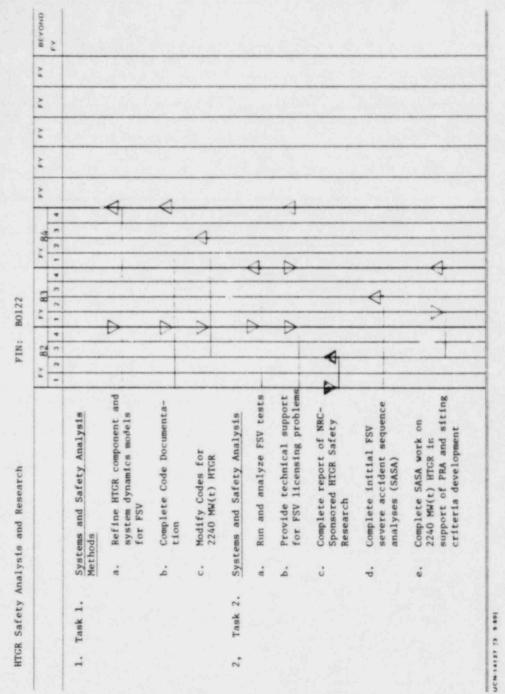
TITLE: Fission Product Release from LWR Fuel ACTIVITY NO.: 60 19 01 40 (80127)

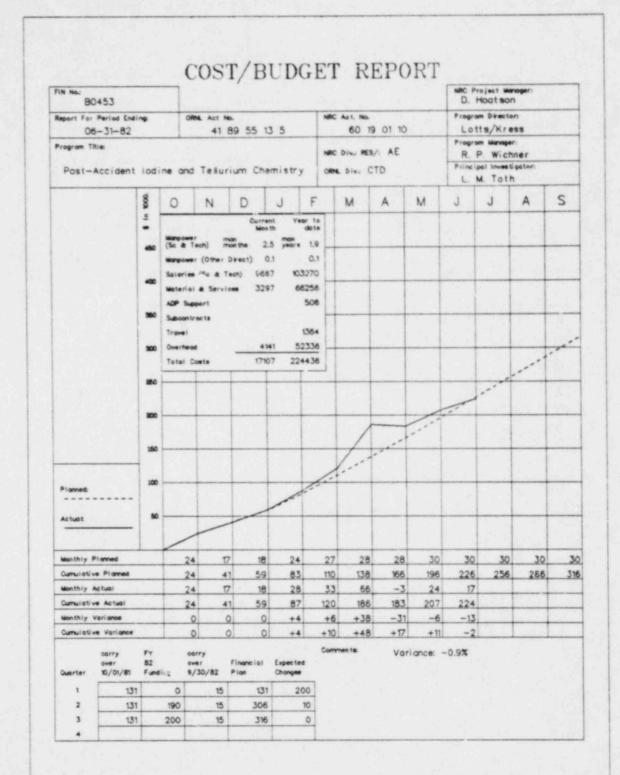


UCM 14127 (3 9-40)



MILESTONE BAR CHART





TITLE: Post-Accident lodine and Tellurium Chemistry

ACTIVITY NO.: 60 19 01 10 (B0453)

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Chemistry Evaluations and Transport Rates		1						ı
1. lodine chemistry evaluation	+	1	_	Lack of staff to do work	staff	co do	work	
2, Tellurium chemistry evaluation		1		in previous fiscal year.	lous fi	Scal	year.	
3. Organic todide-summary of planned experimental approach			D-					
4. Evaluation of transport rates of Ru, Sr, Cs, plus other fission products								
 Summary report on organic todide formation rates 					D			
Iodine Chemistry Tests								
 Complete low temperature (65°C) sump fluid tests 	>							
2. Draft report for tests in B.1					_			
3. Complete tests for sump liquid condi- tions; pH range 6 to 10; concentration range 0.01, 0.1, 1.0, and 10 times nominal; temperatures 65°C, 150°C; boron concentration 0 to 2500 ppm		0		Work to identify HOI was necessary and prolonged efforts.	ident ry and	1fy HO prolo	I was	
4. Draft report on sump fluid tests		1						
5. High temperature (300°C) ceil design and calibration		D		Cancelled: Experiments at	ed: Ex	perime eded.	ents	L.
6. Primary liquid conditions test - PWR. Temperature 150°C; reducing and oxidizing conditions; lodine concentration 0.5, 1, 2 times nominal; range of boron levels			- D			e de cel		

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UCN-14127 [3 9-80]

TITLE: Post-Accident Iodine and Tellurium Chemistry

ACTIVITY NO.: 60 19 01 10

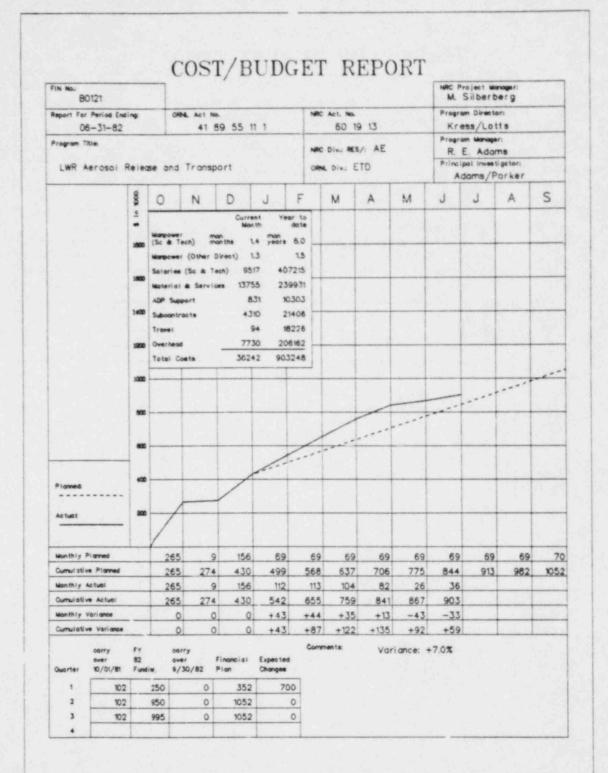
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		1 2	3	4	1	2	3 4	1	2	3		84	85	86	87	88	FY 88
B. Ic	odine Chemistry Tests (Cont'd)					1				^							1
7.	Draft report for tests B.6			1		-					1						170
8.	Primary liquid conditions test — BWR. Temperature 286°C; iodine concentration 0.01, 0.1, 1, 10 times nominal; range of oxidizing conditions										7	7					
9.	Draft report for tests B.9							1			4	7					
10.	Draft final report covering all tests in Subtask B						1	l				Δ					
. Te	llurium Chemistry Tests									V	,						
1.	Tests for sump fluid conditions (see B.3)							H	+	Y	۱						
2.	Draft report; tellurium tests C.1		1			1		-			4	4					
3,	High temperature (300°C) tellurium tests (a portion of test matrix B.7 and B.9)					1	1	1				∇					
4.	Draft final report; tests C.1, C.3					1						Δ					
). Or	ganic Iodine Tests			Н													
1.	Tests on formation by gas phase reaction with methane and CO ₂ ; using I ₂ and gaseous iodide						-	L			7	7					
2.	Draft summary report, tests D.1			П		1	1				1	7					
	Tests on formation by gas phase reaction (12 and iodide) with condensed organic			П		1						Δ					

TITLE: Post-Accident Iodine and Tellurium Chemistry ACTIVITY No.: 60 19 01 10

D.

regant lodine Tests (Cont'd) Draft summary report, tests D.3 Tests on formation in liquid with subsequent vaporization Draft summary report, tests D.5 Draft final report; organic iodide formation mechanism	ts D.3 uld with ts D.5 tc 1odide		1 2 3	* 1 2 3	* 1. 2 3 4	* 84	85	86	87 8	88 rv 88	88
Tests on formation in liquid with subsequent vaporization. Draft summary report; tests D.5 Draft final report; organic iodide formation mechanism.	Tests on formation in liquid with subsequent vaporization. Draft summary report, tests D.5 Draft final report; organic lodide formation mechanism.	Organic Todine Tests (Cont'd)									
Tests on formation in liquid with subsequent vaporization. Draft summary report, tests D.5 Draft final report; organic iodide formation mechanism.	Tests on formation in liquid with subsequent vaporization. Draft summary report, tests D.5 Draft final report; organic iodide formation mechanism.	4. Draft summary report, tests D.3				-					
Draft final report, tests D.5 Draft final report; organic iodide formation mechanism	Draft final report, tests D.5 Draft final report; organic iodide formation mechanism	5. Tests on formation in liquid with subsequent vaporization									
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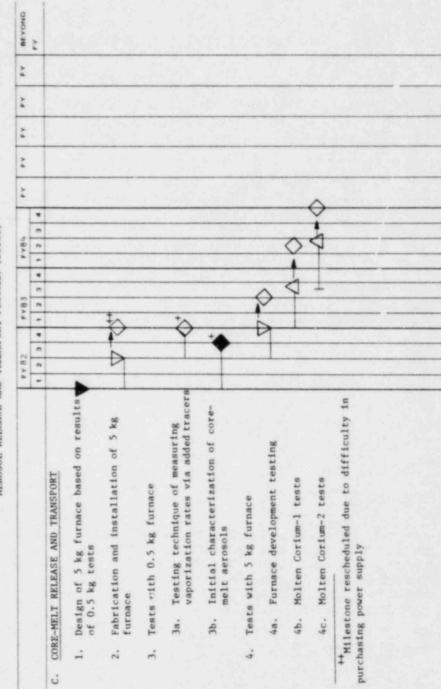


AEFOSOL RELEASE AND TRANSPORT PROGRAM (80121)

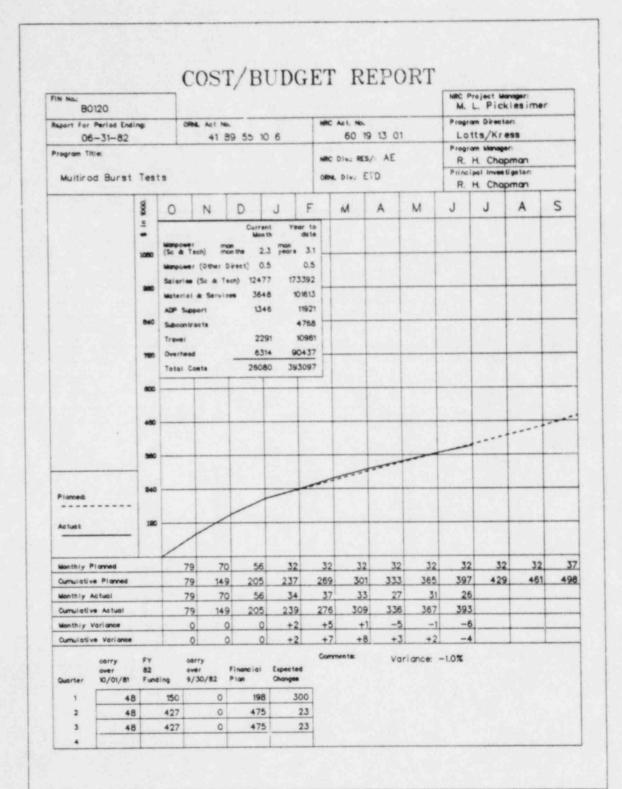
A. MANAGEMENT AND ANALYSES 1. LMFBR - FAST task 2. Test plan for LWR experiments in NSPP (containment aerosols) 3. Test plan for Core-melt release task 4. Preliminary proposal for TRAP-NELT validation experiments 5. Report on NSPP LMFBR mixed aerosols 1. Initial LWR aerosol tests in steam environment 2. Development of "core-melt & concrete" aerosol generator 3. Core-melt aerosols in steam environment 4. NSPP follow-on plans All previous milestones associated with the LMFBR portion of Bol21 are deleted due to NRC direction to delay LWFBR portion until FY83 and decreased level of funding.				# ¥82		F × 8 3	33	-	F VB1	_	^*	*	*	*	>	BEYOND
11. 1. 2. 2. 2. 4. 4. 4. 4. 4. 4. 4. 4. 4. 4. 4. 4. 4.				****	-	-	0	-	2 3			1	+	1	1	
1. 2. 2. 3. 3. 4. All FBR pc devel	A.		AAGEMENT AND ANALYSES							_	-					
2. 4. 4. 11. 12. 2. 2. 4. All devel devel		1.						_		-	-		-			
3. 4. 4. 1. 1. 2. 2. 2. 4. 4. 4. 4. 4. 4. 4. 4. 4. 4. 4. 4. 4.		2.	Test plan for LWR experiments in NSPP (containment aerosols)													
AA11 * AA11 Crease		3,		1	P					_		-				
NSPE 1. 1. 2. 2. 2. 4. 4. All FBR pcrectic devel		4	Preliminary proposal for TRAP-MELT validation experiments							-						
NSPE 1. 1. 2. 2. 3. 4. 4. All FBR pcrectic devel		5.		+	•						-					
1. Initial LWR aerosol tests in steam environment aerosol generator 3. Core-melt aerosols in steam environment 4. NSPP follow-on plans All previous milestones associated with the LWFBR portion of B0121 are deleted due to NRC direction to delay LWFBR portion until FY83 and to develop a separate 189. Milestones rescheduled due to impact of decreased level of funding.	8		PP: LWR CONTAINMENT AEROSOL STUDIES													
2. Development of "core-melt & concrete" 3. Core-melt aerosols in steam environment 4. NSPP follow-on plans *All previous milestones associated with the LMFBR portion of 80121 are deleted due to NRC direction to delay LMFBR portion until FY83 and to develop a separate 189. †Milestones rescheduled due to impact of decreased level of funding.		1.	Initial LWR aerosol tests in steam environment		4	_										
3. Core-melt aerosols in steam environment 4. NSPP follow-on plans * All previous milestones associated with the direction of B0121 are deleted due to NRC direction to delay LMFBR portion until FY83 and the develop a separate 189. **Milestones rescheduled due to impact of decreased level of funding.		2.			1	*					_					
* NSPP follow-on plans * All previous milestones associated with the direction of B0121 are deleted due to NRC direction to delay LMFBR portion until FY83 and to develop a separate 189. **Milestones rescheduled due to impact of decreased level of funding.		3.			+	4	+	^			-					
*All previous milestones associated with the LMFBR portion of B0121 are deleted due to NRC direction to delay LMFBR portion until FY83 and to develop a separate 189. †Milestones rescheduled due to impact of decreased level of funding.		4	NSPP follow-on plans		+	>		_		-	-					
*Milestones rescheduled due to impact of decreased level of funding.	195	* Al (FBR	l previous milestones associated with the portion of 80121 are deleted due to NRC ion to delay LMFBR portion until FY83 and elop a separate 189.													
	de	+ Mi	lestones rescheduled due to impact of sed level of funding.													

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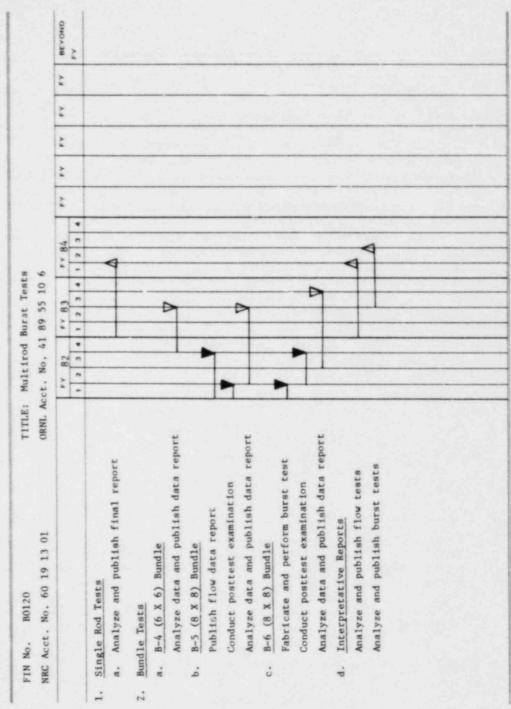
AEROSOL RELEASE AND TRANSPORT PROGRAM (B0121)

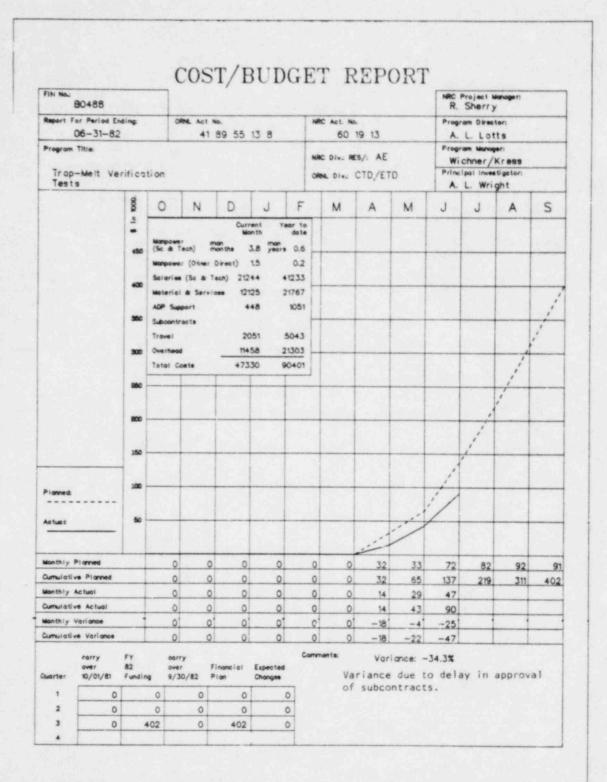


UCN-14127 (3 9-80)



A.





TITLE: TRAP-MELT Veriffication Tests

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		FY 82 FY 83	FY 84	4.6	**	**	**	*	BEYOND
SU	SUBTASK/MILESTONE	1 2 3 4, 2 3 4 1 2 3 4 85	2 3 4	85					2
1	1. Task 1: Management and Analysis								
	a. Complete preliminary work plan (draft report)	4							
	b. Complete overall test plan (draft report)	4							
	c. Complete report on assessment of test results relevant to TRAP-MELT verification			4					
2.	2. Task 2: Aerosol Transport Tests								
	a. Start experiments	4	_					Á	
	b. Complete experiments		4						
	c. Complete data record report			4					
3	3. Task 3: Fission Product Transport Leats							H	

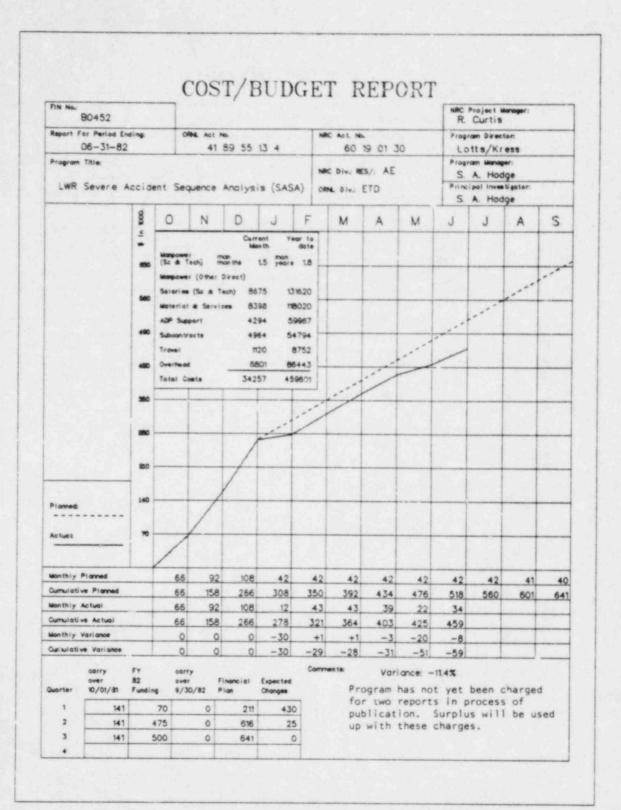
- a. Start experiments
- Complete experiments

Complete data record report

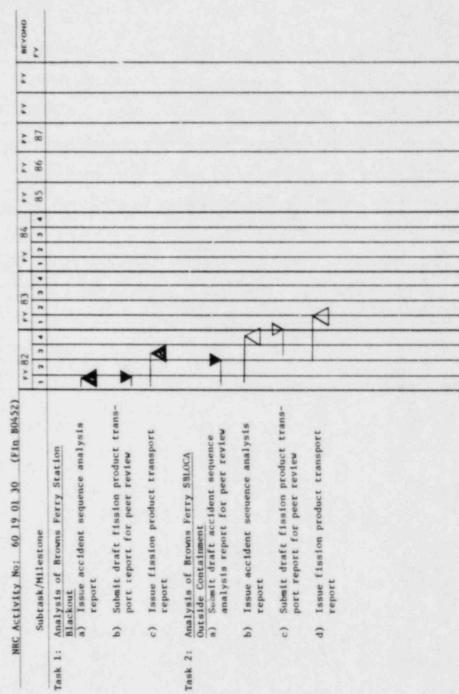
- Task 4: Aerosol Resuspension Tests
- a. Start experiments
- Complete experiments
- c. Complete data record report
- Conduct tests/provide analysis in support Continuing Activity of MARVIKEN experiments Task 5: MARVIKEN Test Support
- Coordinate/support participation of other laboratories in MARVIKEN test

Continuing Activity

UCM 14127 (3 9-80)



TITLE: Severe Accident Sequence Analysis (SASA)



UCN 14127 (3 9-80)

TITLE: Severe Accident Sequence Analysis (SASA)

a) Submit draft accident sequence analysis report for peer review b) Issue accident sequence analysis report c) Submit draft fission product transport report for peer review d) Issue fission product transport report c) Submit draft fission product transport report for peer review d) Issue fission product transport report seport d) Issue fission Pool Response During Severe Accidents a) Issue draft report b) Issue final report	a) Submit draft accident sequence analysis report for peer review b) Issue accident sequence analysis report c) Submit draft fission product transport report for peer review d) Issue fission product transport report report a) Issue fission Pool Response During Severe Accidents a) Issue draft report b) Issue draft report		Subtask/Milestone	1 2 3 4 1 2 3 4 1 2 3 4 85	85 86	87		2
a) b) c) d) d) d) every property of free points and	a) beve d) c) b)	5						
b) a) beve	b) b) beve			P				
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a) Issue draft report b) Issue final report	a) Issue draft report b) Issue final report	4						3 48
			a) Issue draft report	>				
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UCN 14127 (5 9-89)

TITLE: Severe Accident Sequence Analysis (SASA)

		Subtask/Milestone	-	3 4	1 2 3 4 1 2 3 4	1 2 3	4 65	86	87	*	2	2
Task 5:		Analysis of Limerick Station Blackout										
	9	Submit draft accident sequence analysis report for peer review			A							
	9	Issue accident sequence analysis report			A							
	c	Submit draft fission product transport analysis report for peer review			1	<u>A</u>						
	(p)	Issue fission product transport report				4						
Task 6:		Analysis of Limerick Scram Discharge Volume Break										
	a)	Submit draft accident sequence analysis report for peer review			7							
	9	Issue accident sequence analysis report			1	1						
	c)	Submit draft fission product transport analysis report for peer review.				4						
	P	Issue fission product transport					_					

UCR 14127 (3 \$-80)

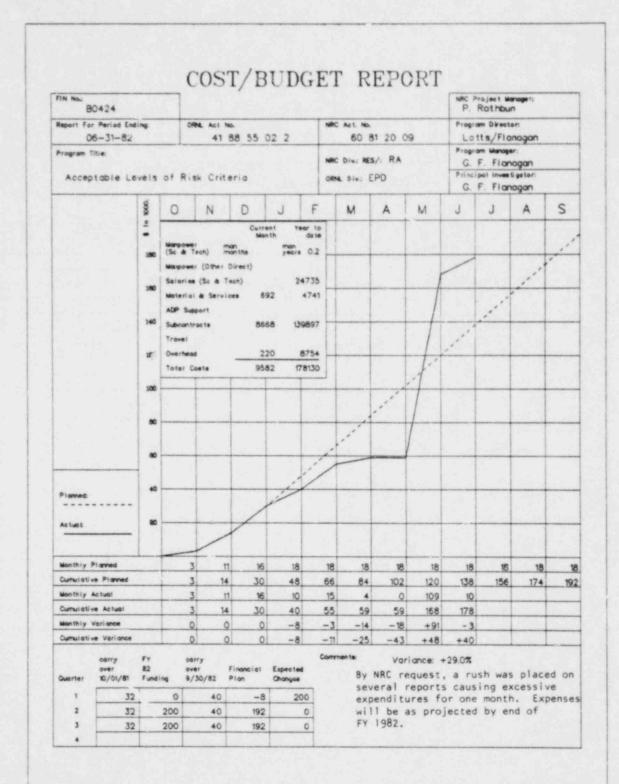
TITLE: Severe Accident Sequence Analysis (SASA)

Subtask/Milestone	1 2 3 4		83		3 4	85	86	2 60	2	*	BEYOND FY
malysis of Limerick Loss of DHR equences											
submit draft accident sequence analysis report for peer review			1	_							
) Issue accident sequence analysis report			_	A							
Submit draft fission product transport report for peer review					7	_					
Issue fission product transport report						1					
ccident Sequence Analysis for ddditional Browns Ferry Sequences nd Reports							7	1			
Analysis of Additional Limerick Accident Sequences and Reports						1	7	1			
	Subtask/Milestone Analysis of Limerick Loss of DHR Sequences a) Submit draft accident sequence analysis report for peer review b) Issue accident sequence analysis report c) Submit draft fission product transport report for peer review transport report for peer review Accident Sequence Analysis for Additional Browns Ferry Sequences and Reports Analysis of Additional Limerick Accident Sequences and Reports	nce lysis port	nce lysis port	nce lysis horr	nce lysis lysis ovr t	nce lysis lysis borr	ace ace lysis lysis borr				

UCN:14127 (3 9-80)

DIVISION OF RISK ANALYSIS

COST/BUDGET REPORT NRC Project Manager: Report For Period Ending. ORNE Act No. NRC Act. No. Program Director: 06-31-82 Program Managers Program Title: NRC DIN. RES/: RA Office of Nuclear Summar y Principal Investigators Regulatory Research ORNE DIV. S F A N D J Promed Actuat Monthly Planned Cumulative Planned 1822 2057 2277 Monthly Actual Cumulative Actual 1295 1554 Monthly Variance -75 -89 +7 +20 -21 +108 Cumulative Variance -75 -157 -137 -158 -164 -50 Variance: -3.1% over Financial Expected 9/30/82 Plan Changes over 82 10/01/81 Funding



ITLE: Acceptable Level of Risk Criteria for Nuclear Power Plants

ACT	ACTIVITY NO: 41 88 55 02 2	B0424	-		1	1	1	
		1 2 3 4 1 2 3	1 2 3	* 84	85	86 87	87 88	86 va
Y.	NRC Workshops on Risk Criteria							
	1. Review and prepare assessments of work-shop results and recommendations	>						
	Evaluation of the NRC Safety Goal Proposal							
	1. Evaluate and respond to comments on proposed NRC goals.		A					
	Complete reports on issues pertaining to quantitative safety goals							
	1. Issue final oraft of "Approaches to Acceptable Risk: A Critical Guide" NUREG/CR-1614	•						
	2. Issue report on "Standard Setting Standards"	Naza Naza	25.4	Delay	Delay in FY-82 funding.	.82 fur	ding.	
	3. Issue report on "Perceived Risk and Quanitative Safety Goals for Nuclear Power"	D	*	Delay	Delay in FY-82 funding.	82 fur	od ing.	
	4. Issue report on "The Public vs. the Experts: Perceived vs. Actual Disagreements about the risks of Nuclear Power"	♦	N ₀₀	Delay in	fi A	FY-82 funding.	· 8 ug pu	
	5. Issue report on "Evaluation of Nortality Risks from an Organizational Perspective"	D		Delay	Delay in FY-82 funding.	-82 fu	nd fing.	

UCN-14127 (3 9-40)

TITLE: Acceptable Level of Risk Criteria for Nuclear Power Plants

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6. Issue report on "Issues in Evaluating Alternative Standards"	1	1	7	7	6/8	82		De	lay fr	FY-82	func	ing.	
7. Issue report on "Understanding How Sa is Safe Enough"	fe	+	+	V	19	>		De	lay in	FY-82	func	ing.	
8. Issue report on "Risk Aversion and Safety Goals for Nuclear Power"	'H	+	+	7	13	82		De	lay in	FY-82	func	ing.	
 Issue report on "Judgmental Issues in Probabilistic Risk Analysis" 		H	+		H	_ \	1	-			1		
10. Issue report on "Incorporating Social Values into Nuclear Safety Goals"		H	+	+	H	+	V						
11. Review and prepare assessments of wor shop results and recommendations	k-	H	*										
12. Evaluate and respond to comments on proposed NRC safety goals			H	+	12	7					H		
Examine the relevance of research on probabilistic judgment for improving probabilistic risk assessment.			-	+	H	+	1	7					
Consideration of public attitudes in formulating quantitative safety goals													
 Examine issues and ramifications associated with incorporating public attitudes 	-	\parallel	+		H	1							

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2.	Evaluate specific models for incorporating public attitudes in risk criteria					A							
3.	Conduct pilot survey to sample public views on issues associated with risk				-	7			1				
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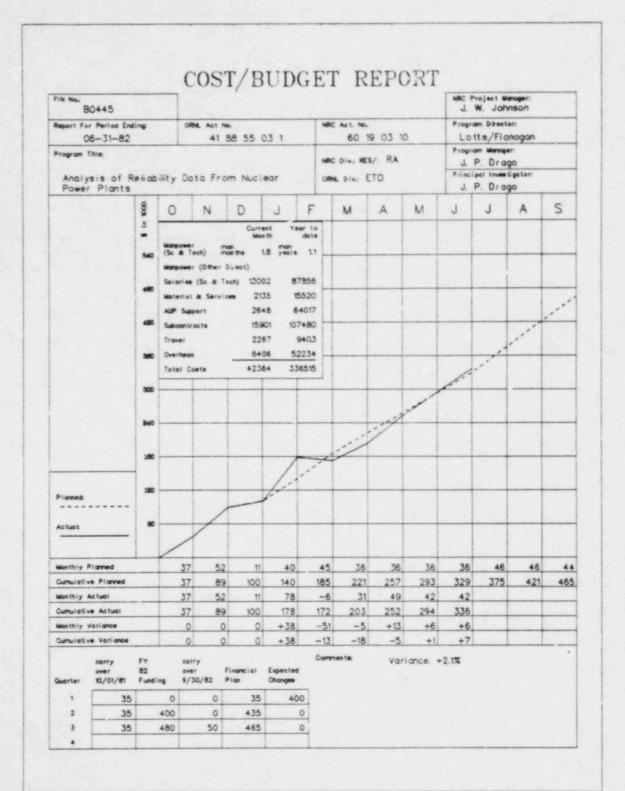
Analysis of Proposed New IAEA Basis for Transportation Regulatory System (80810)

	FY 82	FY 83	FY 84	20 00 300	FY 84 100 100 100 100 100 100 100	BEYOND
A restaurant of the second	1 2 3 4	1 2 3 4		100 11 00	100,1100	FY 89
Critical review of report by Goldfinch and Macdonald	1					
Evaluate other comments on same report	Î					
Participate in IAEA meeting in London	Î					

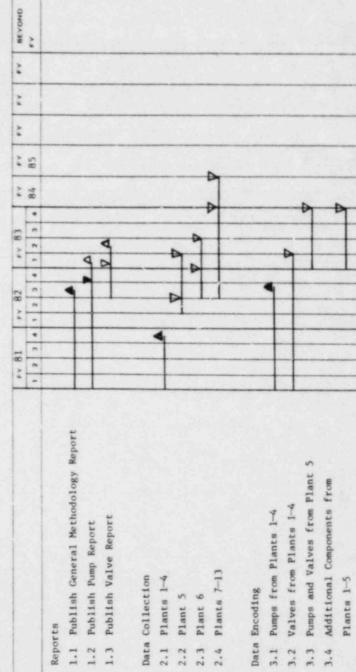
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UCN (4127 (3 9-80)



TitlE: Analysis of Reliability Data from Nuclear Power Plants ACTIVITY NO: 41 88 55 03 01 80445



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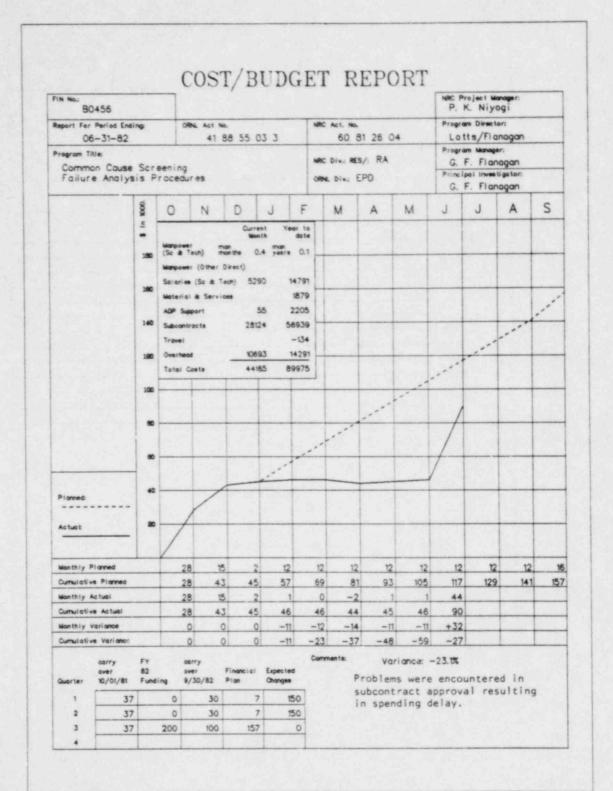
UCM 14127 (3 5-89)

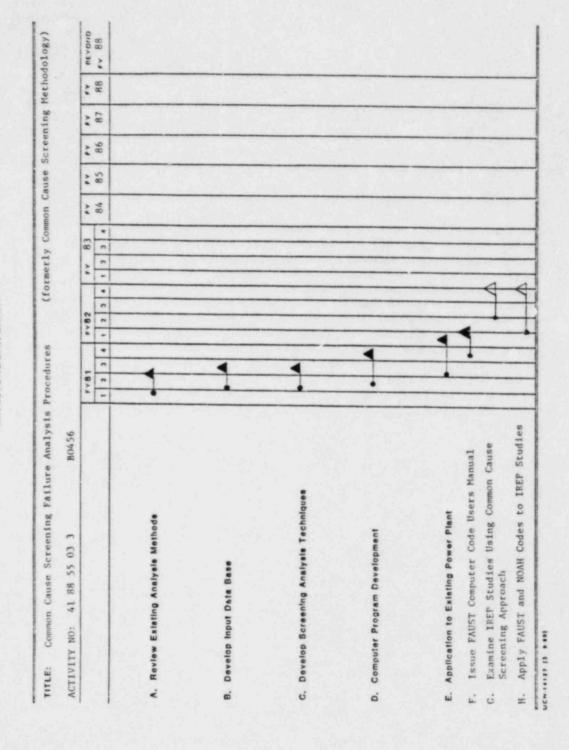
COST/BUDGET REPORT NRC Project Manager W. E. Vesely B0459 Report For Period Ending Program Directors ORNE. Act No. 06-31-82 41 88 55 03 7 60 81 19 08 Lutts/Flanagan Program Manager: Program Title: NRC DIV. RES/I RA G. F. Flanogan Principal Investigator Common Cause Evaluations in Applied Risk Analysis OPPL DING EFO G. F. Flanagan 1000 S 0 N D M A A 5 Nonpower man months Margower (Other Direct) Sqiquies (Sc & Tech) 1615 Material & Services 120 ADP Support 38107 759 902 41507 40 Planned Actuo: Monthly Planned 0 0 0 Cumulative Planned 28 36 12 41 41 41 41 41 41 41 41 41 Monthly Actual 12 8 0 16 5 0 0 Cumulative Actual 12 28 36 36 41 41 42 42 42 Monthly Varianc 0 0 0 -5 +5 0 0 Cumulative Variance 0 0 0 -5 0 0 +1 +1 Comments: Variance: +2.4% ourry over 9/30/82 carry over 82 10/01/81 Funding Plan Changes 41 0 41 0 0 2 41 0 41 0 0 3 41 0 0 41 0

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A. Identify key variables B. Use check lists to develop rankings C. Develop alternate approaches E. Final report E. Final report C. Delay in report reviews.	ACTIVITY NO: 41 88 55 03 7	80459
Use check lists to develop rankings Develop alternate approaches Apply proposed approaches Final report Final report		2 3 4 1 2 3 4 1 2 3 4 84 85 86 87 88
Develop alternate approaches Apply proposed approaches Final report Final report Final separt Apply proposed approaches Final report Final separt Fi	Identify key	14
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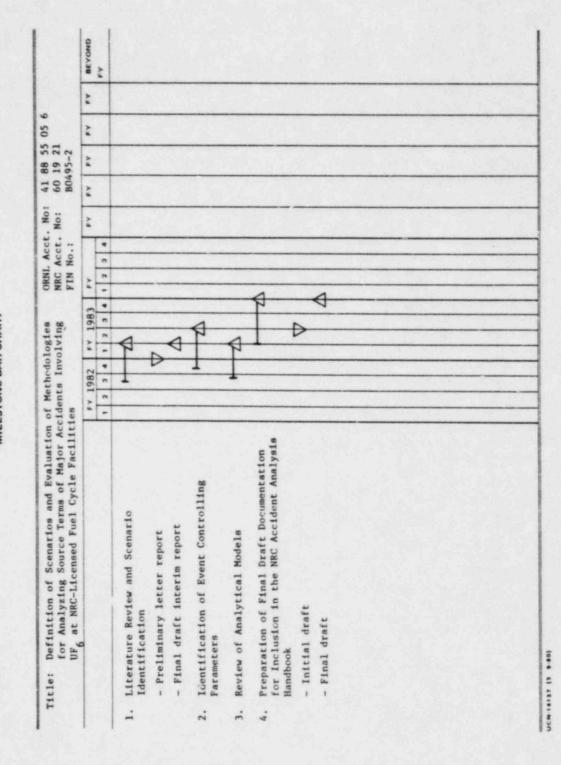
UCH-14127 (3 9 86)

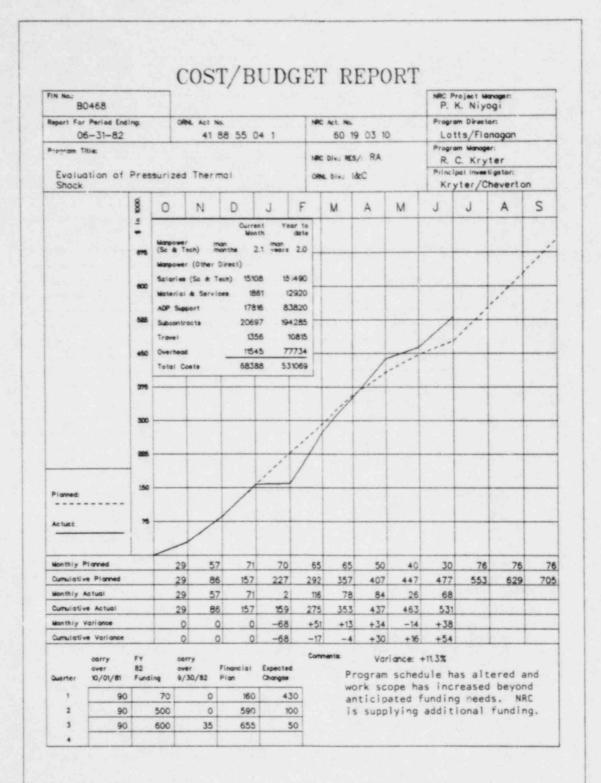




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COST/BUDGET REPORT NRC Project Managers S. Brernstein BO495 Report For Period Ending: Program Director: ORNE Act No. 06-31-82 41 88 55 05 6 60 19 21 Lotts/Flangan Program Manager: Program Title: NRC DIV.: RES/: RA Simon-Tov Principal investigato Definition of Scenarios and Evaluation of Methodologies ORNE DIV. ENG. Just/Huxtable 1000 0 N D M A M A S 5 Year to Manpower man (Sc & Tech) months Manpower (Other Direct) Saigries (Sc & Tech) Material & Services 12820 12820 ADP Support Subcontracts Trovel 4095 4095 Overhead Total Costs 16916 16916 Actuat Nonthly Planned 25 Cumulative Planned 0 0 17 82 107 Monthly Actual 0 17 Cumulative Actual 0 0 17 0 0 Monthly Variance 0 0 Cumulative Variance 0 0 0 0 0 Variance: 0.0% Financial Expected over 82 10/01/81 Funding over Finan 9/30/82 Plan 0 0 2 0 0 0 0 3 0 48 107 0

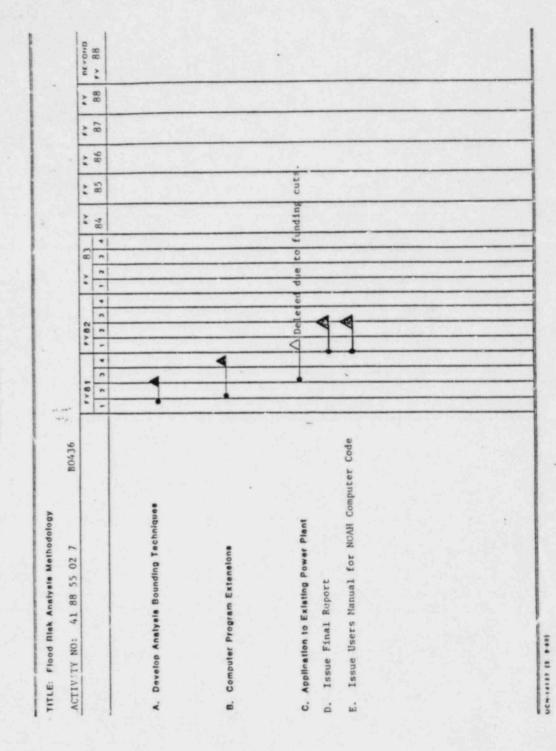




Title: Evaluation of Pressurized Thermal Shock Activity No.: 189 B0468/ORNL #41 88 55 04 1 FY 1982 FY 1983 | +: 1984 BEYOND 1 2 3 411 2 3 4 FY 1 2 3 4 Task 1--Re-analyze Oconee-1 A. Gather Required Plant Data B. Construct Overcooling Event Sequences C. Estimate Sequence Probabilities T-H dalks, delayed 21 months D. Specify, Coordinate, and Review T-H Calculations Performed By Others E. Estimate Threat to Vessel Integrity T-H Cales delayed 25 months T-H dales, delayed 25 months F. Estimate Result Sensitivities G. Analyze Effectiveness of Potential VI-# Cales, delayed 25 months Corrective Measures H. Perform T-H Calculations (LANL; INEL) Calls delayed 25 months Task 2--Analyze Representative W Plant Plant designated July 7, 1982: Work to begin September 15, 1982 on accelerated schedule. Task 3--Analyze Representative CE Plant Plant designated June 22, 1982; Work to begin August 1, 1982 on accelerated schedule.

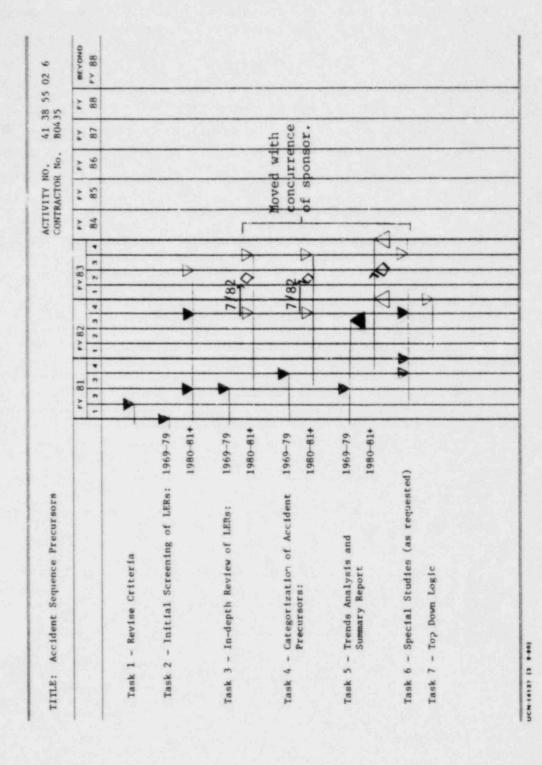
COST/BUDGET REPORT P. K. Hiyogi B0436 Program Director: NRC Act. No. Report For Period Ending: 41 88 55 02 7 60 81 03 10 Lotts/Flanagan 06-31-82 Program Manager: G. F. Flanagan Program Title: NAC DIV. RES/: RA Methodology for Flood Risk Analysis ORNE DIV. EPD G. F. Flanagan 8000 F Α J A S M J 0 N D M ž. Manpower man (So & Tech) months Manpower (Other Direct) Salaries (Sc & Tech) -1023 Material & Services 789 208 Total Costs -24 Planned: Actuat Monthly Planned 0 Cumulative Planned 0 -2 0 0 0 Monthly Actual Cumulative Actual 0 0 Monthly Variance 0 0 0 +3 Cumulative Variance 0 0 +3 Comments Variance: 0.0% FY cerry ourry over Finance 9/30/82 Plan 82 Funding 10/01/81 Changes 0 0 5 5 0 0 0 2 0 0 0 0 0 0 3 0 0

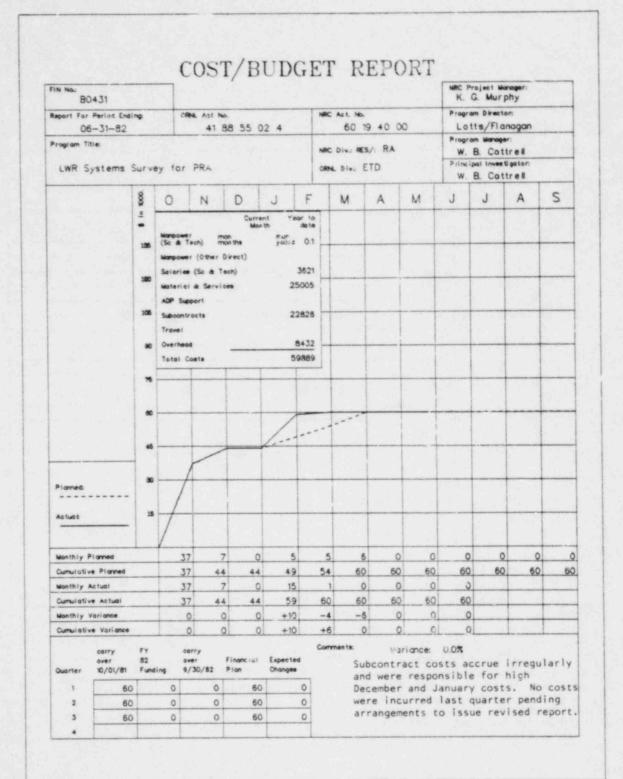
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COST/BUDGET REPORT NRC Project Manager F. Manning 80435 Report For Period Ending: Program Director: NRC Act. No. Lotts/Flanagan 41 88 55 02 6 60 19 40 01 06-31-82 Program Manager: Program Title: NRC DIV. RES/: RA W. B. Cottrell Principal Investigator LWR Accident Sequence Precursors ORNE DIV. ETD W. B. Cottres Study S 000 A N M <u>=</u> 3.0 years 12 Manpower (Other Direct) Salaries (Sc & Tech) 18281 Waterial & Services 32 14359 163708 10.31 10698 30041 291138 58548 Total Costs Planned: Actuat Wonthly Planned 42 48 40 40 40 42 42 40 40 -6 Cumulative Planned 51 135 177 225 265 305 385 425 9 93 345 Monthly Actual 19 72 42 39 59 15 -6 42 Cumulative Actual 51 60 79 151 193 291 -6 9 Monthly Variance 0 -33 -23 +30 -6 +19 0 -32 -14 -56 -26 Variance: -4.6% Financial Expected over 82 10/01/81 Funding over Finan 9/30/82 Plan Changes 0 307 118 0 118 2 118 389 82 425 0 0 118 389 82 425

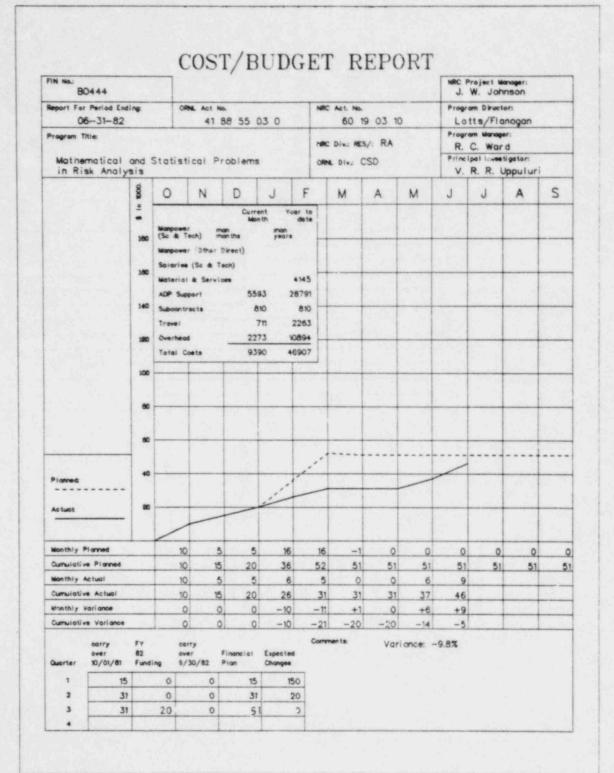
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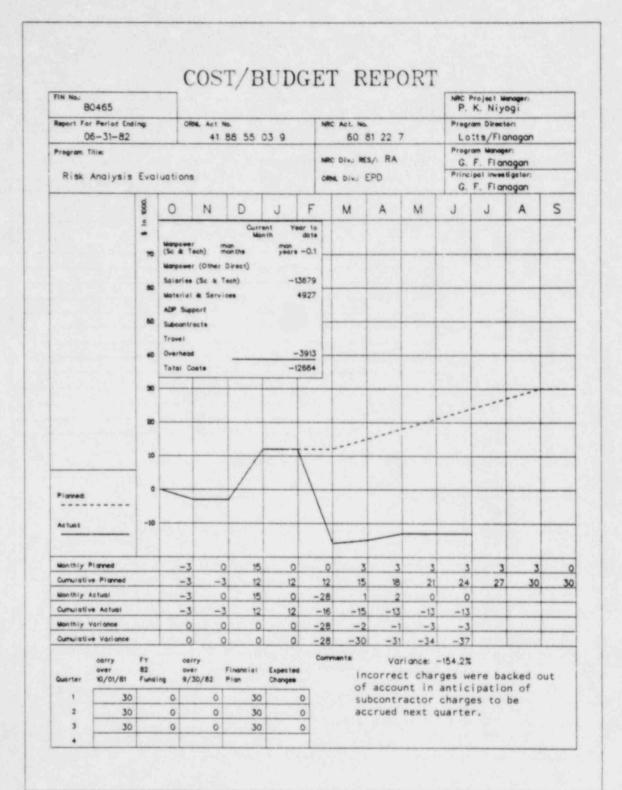
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Task A: LWR Systems Survey 1. 7 3 4 1 7 3 4 1 7 3 4 1 7 3 4 1 8 8 8 8 8 8 8 8 8 8 8 8 8 8 8 8 8 8	IIIE:	IIILE: LWR Systems Survey		CONTRACTOR NO.	ACTOR		B0431	B0431
port (Data on all ts - updated Updated annually) ata by units,				7 78	-	-	-	
updated annually) Updated annually) Data by units, C	Task A:	LAR Systems Survey						
2. Final Report (Updated annually) Evaluation Data 1. Comparison of Data by units, vendors, AE, etc.		1. Freliminary Report (Data on all operating plants - updated annually)	1					
1. Comparison of Data by units, vendors, AE, etc.			0	Await	ing	furt	her	rect
Comparison of Data by units, vendors, AE, etc.	Task B:							
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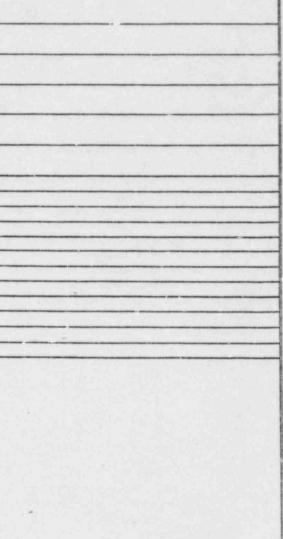
Title: Mathematical and Statistical Problems in Risk Analysis Activity No. 189 B0544/ORNL # 41 88 55 03 0

Propagation of Uncertainties Propagation of Uncertainties Propagation of Uncertainties All personnel shortage and delay in funding.		1 2 3 4 1 2 3 4 1 2	4	2	2	2	2	*	BEYOND FY
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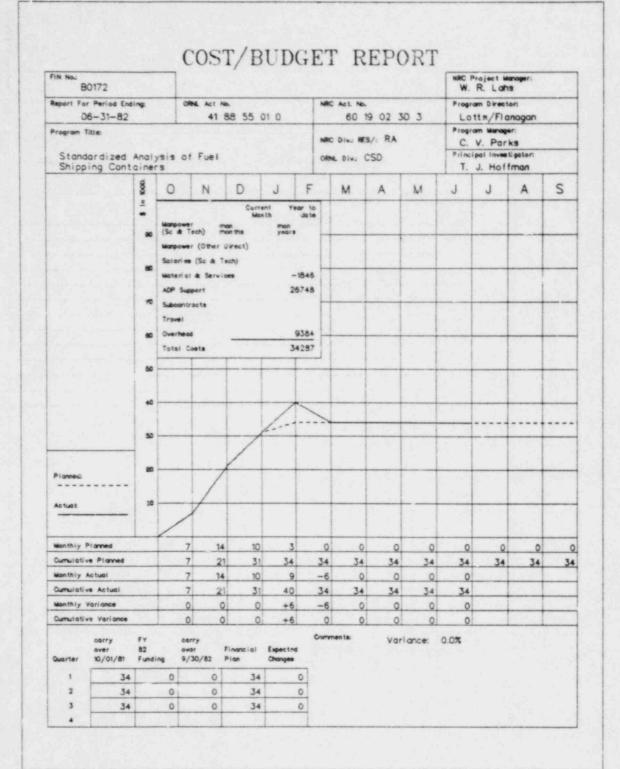


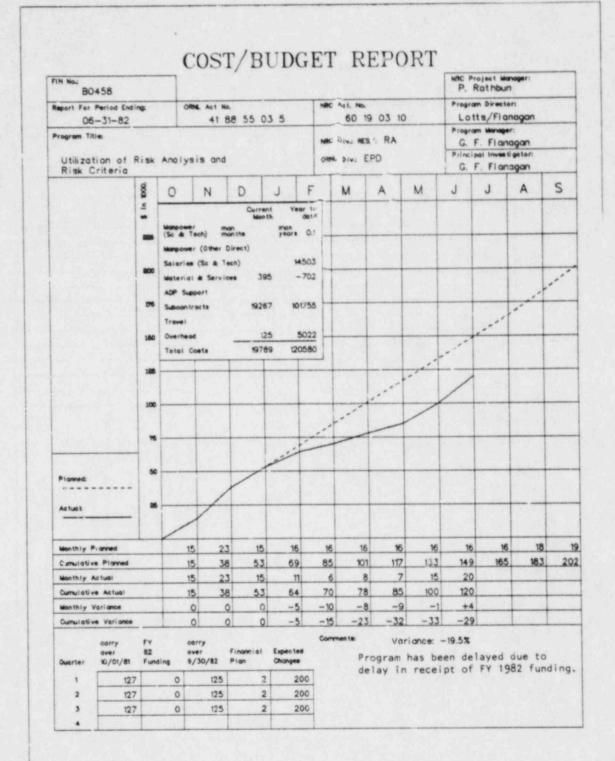
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86 87 88 rv 88 * 4 Define and Evaluate Issues Presented in PRA Evaluate the Implications of the Results with Regard to Numerical Safety Criteria Evaluate and Illustrate By Example the Techniques to Introduce Formal Decision Making Approaches into NRC's Use of Risk 80465 TITLE: Risk Analysis Evaluations ACTIVITY NO: 40 10 01 06 5 Analysis Α. 0



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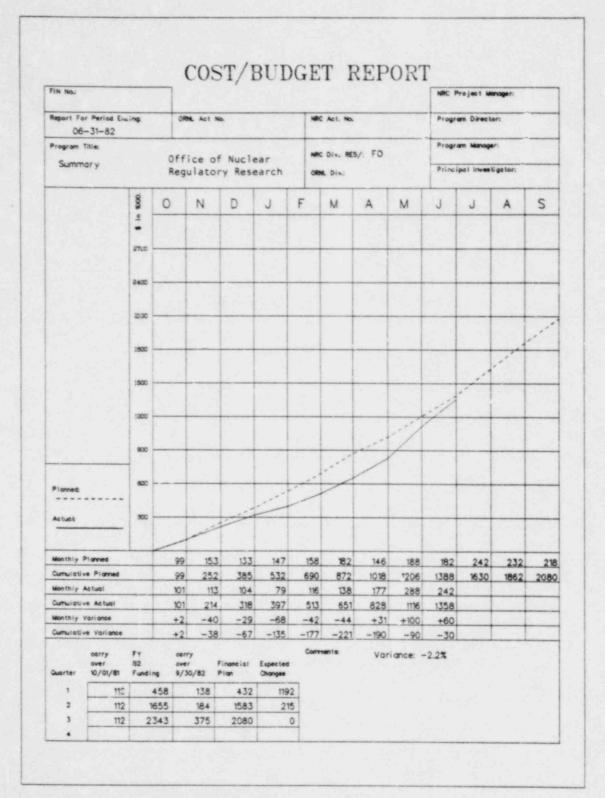


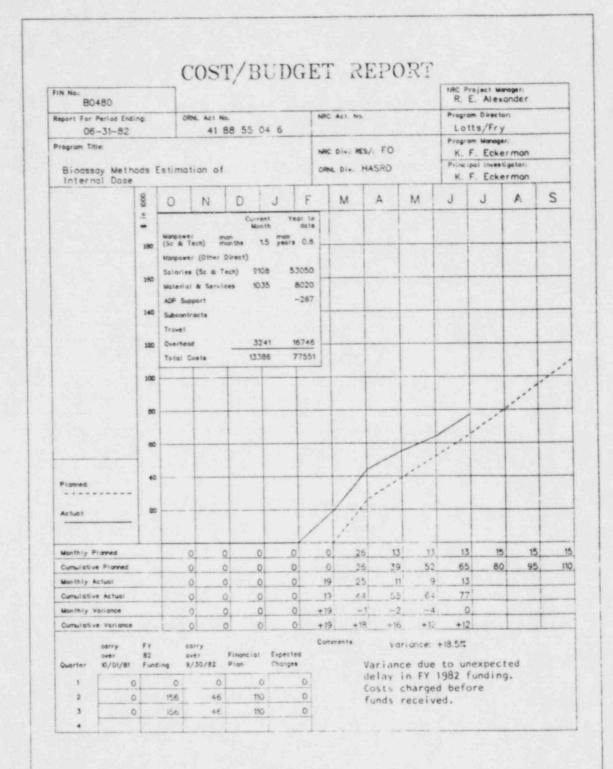
TITLE: Utilization of Risk Analysis and Risk Criteria

ACTIVITY NO: 40 10 01 06 5 B0458

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		1 2	3 4	1	2	3 4	1	2	3	4	84	85	86	87	88	FY 88
	Review Documents	1-	1	4						1						
	Detail Project Objectives	H	H	A						1						
	Define Risk Criteria Performance	Ш		4												
	Characteristics		1	11	1					1	Comp	leted	pric	r to	compl	etion da
	Evaluate Criteria	1	1	1	1	•				-						,
	1. Identify 2. Evaluate	11			1					1			. 10			100
	3. Interim Report (WD) on Tasks 1 and 2			11	1	182	9/	12		1				5-		
	n i normal Cathoria			14	A	X	DI			1	Dela	y in	FY-82	fund	ing.	
	Rank Proposed Criteria 1. Develop weighting scheme			П	٦					1						
	2. Apply weighting scheme						9	82		1						
	Approaches to Risk Management				V	Α,	10	>		1	Dela	y in	FY-82	func	ing.	
*	1. Identify alternatives			П	Y					1	- 41					
	2. Fyaluate alternatives				1					1						
	3. Interim Report (WD) on Tasks 1 and 2	11						91	82	1	Dal	y in	EV_R	fun	ing.	
	Prepare NUREG Report				1	+	14	77	1	1						
	and the set obtained that an				1	1	81	52		1	Dela	y in	F1-84	Tuno	ing.	
	Identify Hierarchy of Objectives that an Appropriate Risk Goal Should Serve		1	H	4	+				1						
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ı.	Develop Implementation Scheme for Managing Risks within NRC Organizational Structure				1				7	1		3		-		
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DIVISION OF FACILITY OPERATIONS

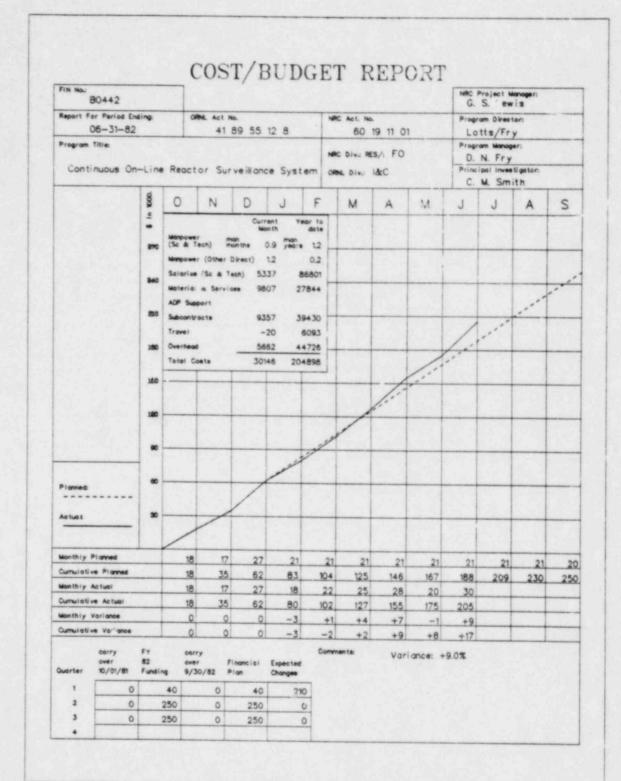




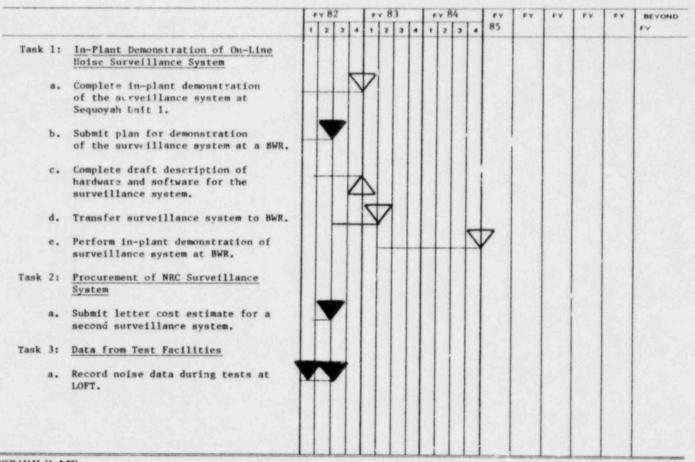
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Task 1: Develop bioassay formulations Task 2: Search bioassay/metabolic literature Task 3: Develop bioassay and dosimetric data reports Task 4: Develop bioassay and dosimetric data for additional radioelements					M
	1 2 3 4 1 2	2			ements
		Develop bioassay formulation Search bioassay/metabolic	literature Develop bioassay and dosime	Develop bioassay and dosime	data for additional radioe
		Task 1: Task 2:	Task 3:	Task 4:	

UCN 14127 (3 9-80)

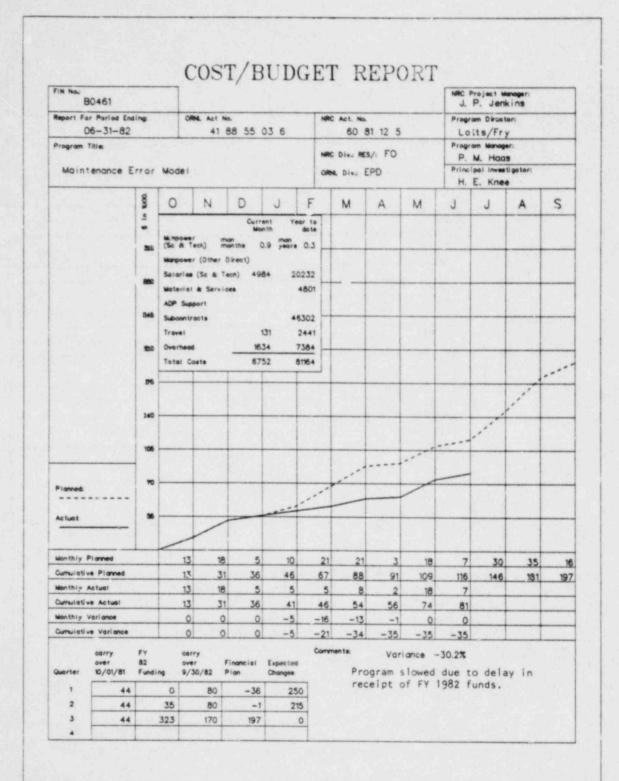


Title: Continuous On-line Resetor Nois: Surveillance System Evaluations Activity No: 41 89 55 12 2/Mac #60 19 11 01 (80442)



Title: Continuous On-Line Reactor Noise Surveillance System Evaluations Activity No: 41 89 55 12 8/NRC #60 19 11 01 (80442)

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Task 4:	. 4	Data Analysis								
	ė	Establish a baseline signature for normal Sequoyah FWR operating characteristics.								
	ż	Submit preliminary report on analysis of LOFT and Semiscale noise data.								
	3	Submit draft report on feasibility of on-line surveillance system for detection and diagnosis of anomalous PWR operating conditions.		7	1					
	÷	Complete evaluation of surveillance system in BWR and establish a BWR baseline signature for normal operating characteristics.					Q			



7.3

ACTIVITY NO: 41 88 55 03 6

B0461

BEYOND 87 85 86 FY 88 1. Task 1: Scoping Study a. Review existing human reliability methodology for application to NPP maintenance tasks b. Survey potential users to identify model requirements c. Perform initial job/task analysis d. Issue 'comprehensive program plan e. Review current job performance Deleted due to change in Program Plan aids (JPAs) for NPP maintenance tasks and provide comments to NRC on potential JPAs 2. Task 2: Model Development a. Formulate analytical model using existing methodology as possible b. Collect necessary job/task ' analytic data to support model development c. Develop computer simulation

UEN-14127 (3 9-80)

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. Task 2:	Model Development (Continued)												
	d. Debug and document computer model						1	Z					
. Task 3:	Model Validation												
	a. Select sample cases and identify required input data for validation							D					
	b. Collect necessary Input data							7	7				
	c. Apply model to predict maintenance personnel reliability for sample cases							-	4				
1	d. Collect and assess available data to validate model prediction			.				-	V				
.	e. Document model validation								Z				
Task 4:	Transfer Model to Users			11					1.				
	a. Develop necessary training materials and documentation						1	-	-	D			
	b. Provide training courses as necessary to transfer methodology and data collected to users									D			

UCN-14137 (3 9-30)

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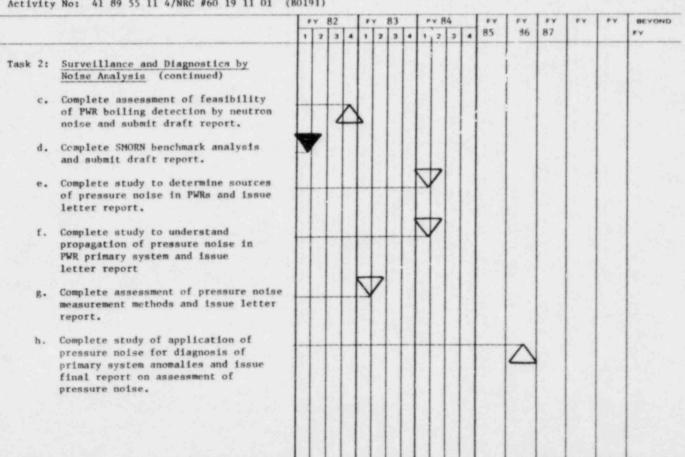
COST/BUDGET REPORT NRC Project Manage G. S. Lewis B0191 Program Director: Report For Period Ending: NRC Act. No. ORNE Act No. Lotts/Fry 60 19 11 01 06-31-82 41 89 55 11 4 Program Manager: Program Title: NRC L V. RES/: FO D. N. Fry Principal investigator Noise Diagnostics for Fafety Assessment DRNL DI. : I&C J. A. Mullens MOOD. F A S D A M Manpower man (Sc & Tech) months Manpower (Other Direct) 0.4 Scieries (Sc & Tech) 4038 95974 47170 Material & Services 1386 497 13855 74129 341 5905 3540 54478 23662 Total Costs 278165 Monthly Planned 58 28 28 28 28 29 29 29 29 29 Cumulative Planned 35 93 121 149 177 205 234 263 292 321 350 Monthly Actual 58 47 34 21 28 24 Cumulative Actual 35 93 140 171 205 226 254 278 Monthly Variance +19 +3 +6 Cumulative Variance 0 0 +15 +19 +22 +28 +21 +20 Comments: Variance: +5.7% carry over Financial Expected 9/30/82 Plan Changes over 82 10/01/81 Funding Quarter 0 48 0 302 48 0 350 0 350 0 0 350 0 350 0

Title: Noise Diagnostic Methods for Safety Assessments Activity No: 41 89 55 11 4/NRC #60 19 11 01 (80191)

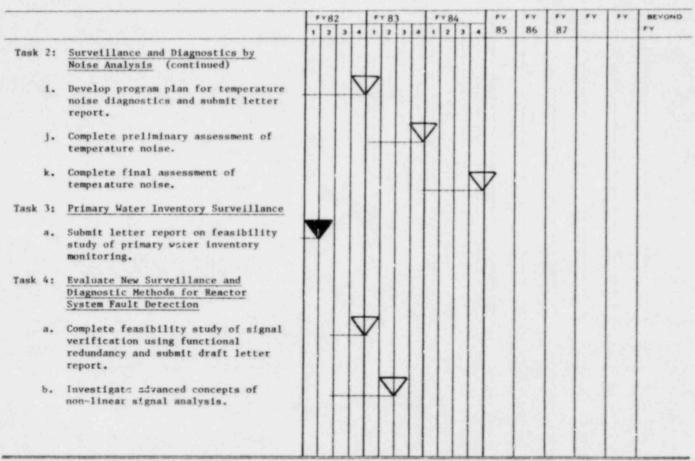
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ask	1:	Monitoring Methods to Detect and Quantify Flow-Induced Vibrations of In-Vessel Components																	
	а.	Complete assessment of sensitivity of neutron noise for PWR fuel assembly vibration monitoring and submit draft report.				4	7												
	ь.	Initiate study of feasibility of detecting thermal shield vibration with neutron noise.				Y													
	с.	Develop program to obtain capability to calculate vibrational frequencies of in-vessel components for NRC approval.				1													
isk	2:	Surveillance and Diagnostics by Noise Analysis						-											
	а.	Obtain continuous baseline neutron noise signatures from Sequoyah Unit 1 PWR.	H			V	7												
	b.	Complete analysis and cataloging of baseline neutron noise signatures obtained from operating reactors and issue draft of final report of baseline neutron noise signatures.				1	4	4	1										

UCN 14127 (3 9-80)

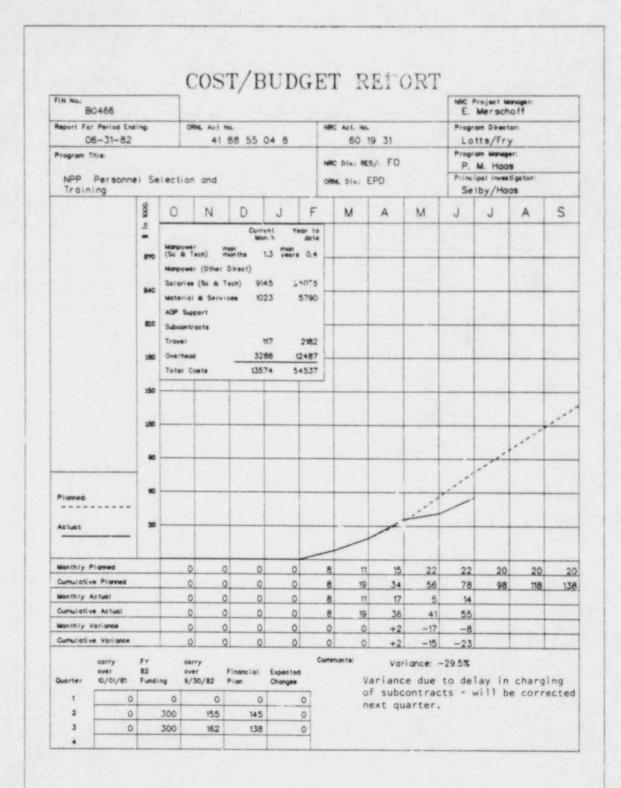
Title: Noise Diagnostic Methods for Safety Assessments Activity No: 41 89 55 11 4/NRC #60 19 11 01 (80191)



Title: Noise Diagnostic Methods for Safety Assessments Activity No: 41 89 55 11 4/NRC #60 19 11 01 (B0191)



UCN 14127 (1 9-80)



ACTIVITY NO: 40 10 01 06

			F	r 8:	2		FY	83	1		FY	34		FY	FY	FY	FY	FY	BEY	MON
31 10		,	,	2	3	4	,	2 3	1 4	,	2	3	4	85	86	87	88	89	FY	8
1. Task 1:	Assess the applicability existing methods such as Systems Approach to Training of Selection, qualification, training of nuclear power operators (SAT/ISD) to the area of selection, qualification, training of nuclear power operators C.: Using the INPO job/task and data, demonstrate the method used to determine selection qualification, and training program requirements d. Provide a comprehensive properators d. Provide a comprehensive properator selection and application of a system and application of a system and application and training requirements	ors, ors of ing Design and plant nalytic hods on, ng rogram idation em such ection,							A STATE OF THE PARTY OF THE PAR											

TITLE: NPP Personnel Select	ion and	Training
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				FY	82	1	¥	83	FY	84		FY	FY	FY	FY	FY	BEY	ONE
Training Simulators a. Develop and demonstrate a technique to select malfunctions which should be required for NPP training simulators b. Develop, as part of the comprehensive program plan in Task 1.d, a plan for research and assessment recessary to specify and validate NPP train-	2	1 1 1		1 2	1	4 1	2	3 4	1	2 3	4	85	86	87	88	89	FY	89
comprehensive program plan in Task 1.d, a plan for research and assessment recessary to specify and validate NPP train-	, Task 2:	a. Develop and demonst technique to select which should be req	rate a malfunctions uired for NPP			,	7											
		Task 1.d, a plan fo and assessment rece specify and validat	am plan in r research ssary to e NPP train-				Z	7										
							1 1 1 1 1 1 1 1											

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COST/BUDGET REPORT NRC Project Manager: S. D. Richardson BC486 Report For Period Ending: Program Director: ORNE Act No. 60 19 51 Lotts/Fry 41 88 55 05 0 06-31-82 Program Manager: Program Title: NAC DIV. RES/: FO W. B. Cottrell Principal Investigator: Nuclear Plant Management Appraisals ORNE DIV. ETD W. B. Cottrell F S D A Manpower man (Sc & Tech) months 1.6 years 0.5 Manpower (Other Direct) 0.7 0.2 Salaries (Sc & Tech) 9702 36570 4357 14381 368 4492 15649 Total Costs 18553 66971 Actuat Monthly Planned 0 0 0 9 8 8 8 8 8 8 Cumulative Planned 0 32 0 0 9 17 23 24 40 48 56 64 Monthly Actual 0 C 0 22 0 19 19 Cumulative Actual 0 0 0 0 0 48 67 26 Monthly Variance 0 0 0 -9 -8 +14 +11 +1 +18 Cumulative Variance -9 -17 -16 +16 +27 Comments: Variance: +67.5% carry carry 82 Funding over Finan 9/30/82 Plan Financial Expected Program closed out, costs will be 10/01/81 Changes Quarter adjusted next quarter. 0 0 0 0 0 2 0 0 64 0 64 3 0 0 64 0 64

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1 2 3 4 1 2 3 4 1 2 3 4 Task 2 - Evaluation of Performance Appraisals TITLE: Nuclear Plant Management Appraisals Task 3 - Development of Quantitative Appraisal Criteria Task 1 - Review of SALP Evaluation Guidance (B0486-2)

UCN 14127 (3 9-80)

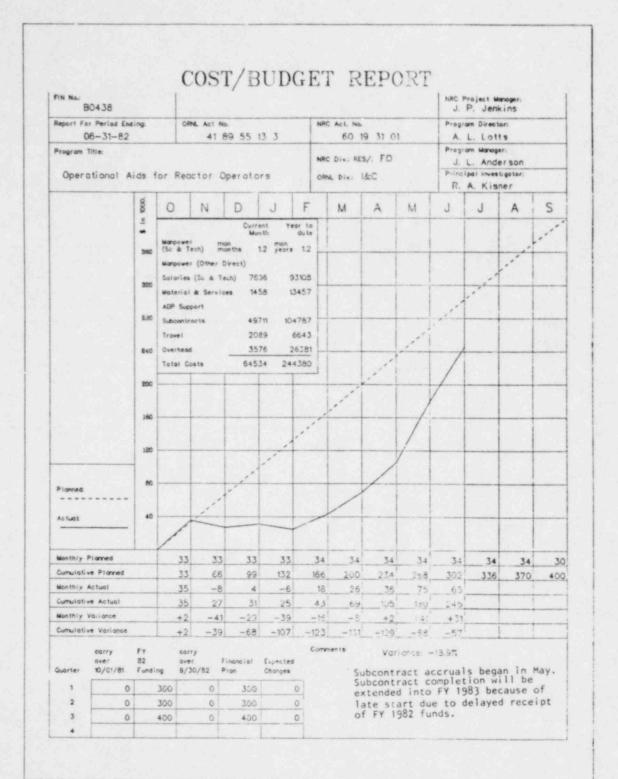
COST/BUDGET REPORT FIN No. NRC Project Manager: S. A. McQuire B0485 Report For Period Ending: Program Director: NEC Act. No. A. L. Lotts 06-31-92 41 88 55 04 9 60 19 31 Program Manager: Program Title: NAC DIV.: RES/: FO C. S. Sims Principal investigator Occupational Radiological Monitering AT Uranium Mills ORNE DIV. HSR R. E. Swaja F S A A Manpower man (Sc & 1 - un) months 0.4 man 0.1 Manpower (Other Direct) Salaries (Sc & Tech) 4383 Material & Services ADP Support Subcontracts 1098 1098 Travel 1750 1750 7233 17330 Total Costs 10 Actuat Monthly Planned 0 0 0 0 0 10 Cumulative Planned 0 0 0 0 0 0 15 25 35 40 55 Monthly Actual 0 0 0 0 0 0 10 Cumulative Actual 0 0 0 0 0 10 17 0 0 Monthly Variance 0 -5 0 0 0 0 0 -3 0 Cumulative Variance 0 0 0 0 0 -5 0 0 -8 Comments: Variance: -32.0% carry 10/01/81 Funding 9/30/82 Plan Variance to be made up next Changes quarter. 0 0 0 0 0 2 0 0 0 0 0 3 0 100 45 55 0

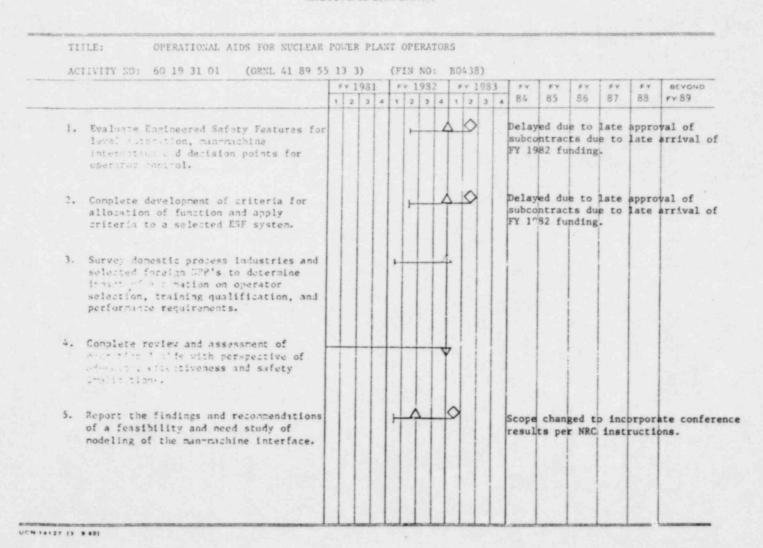
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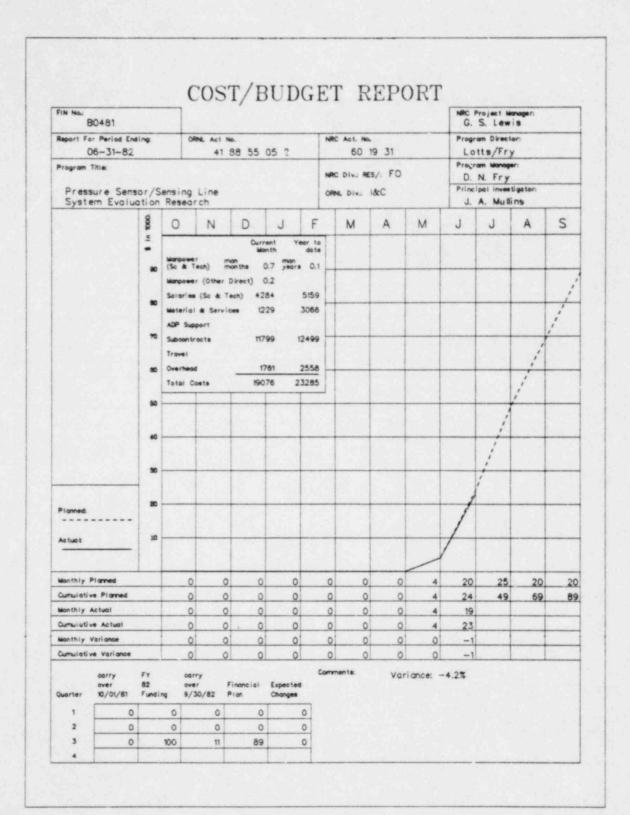
Title: Occupational Radiological Monitoring at Uranium Mills Activity Number: ORNL #41 88 55 04 9 (189 #B0485)/NRC #60 19 31

	Tasks Submit detailed outline of manual entitled "Occupational Radiological Monitoring at Uranium Mills" to NRC for approval by May 31, 1982. Provide NRC with 400 copies of the completed manual by May 1, 1983.	-	· •	^ 4	•	c × -	•	444			2
	detailed outline of manual entitled aational Radiological Monitoring at m Mills" to NRC for approval by i, 1982. He NRC with 400 copies of the sted manual by May 1, 1983.		•	4							
2. Provide complete	ted manual by May 1, 1983.			4							
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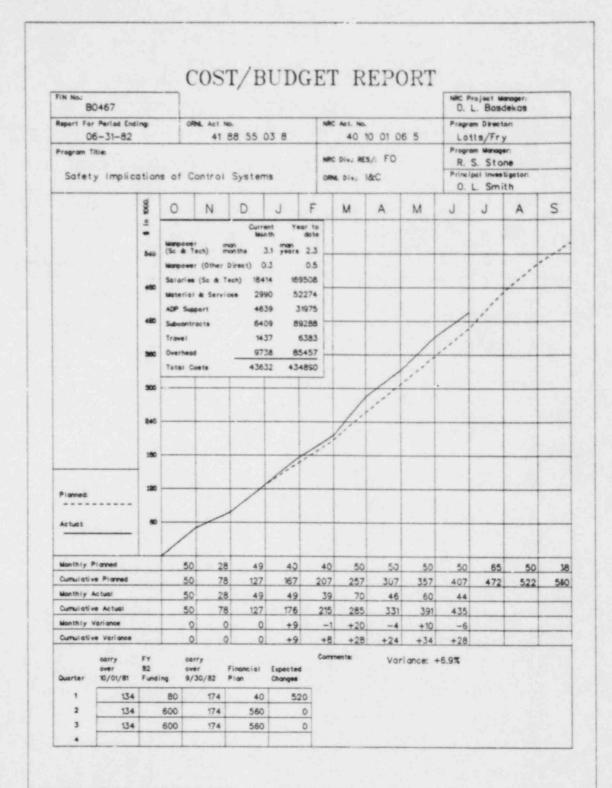


TITLE: Pressure Sensor/Sensing Line System Evaluation Research

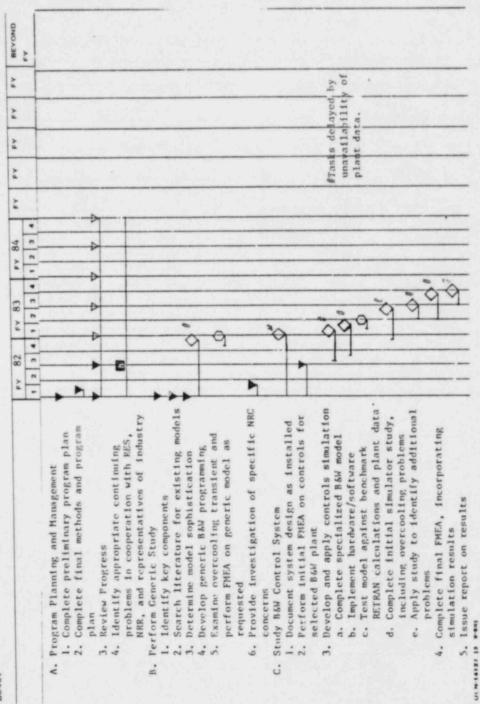
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ask 1:	Sensi	w Status of Pressure Sensor/ ng Line System Standards, ices and Technology																
		Complete survey of existing standards and current practices and submit letter report. Complete review of Sequoyah sensor/sensing line design practices and issue letter report.					4											
ask 2:	Press	Complete survey of experience in sensor/sensing line degradation and issue letter report. ure Sensor/Sensing Line Model opment		-	4	4												
	2.1.	Complete dynamic model of Foxboro E13DM force balance sensor and issue letter report.		-														
	2.2.	Complete dynamic model of another pressure senosr and issue letter report.				-	1	-		1								
	2.3.	Complete sensor/sensing line model development and testing.			-		1	1		4	4							

TITLE: Pressure Sensor/Sensing Line System Evaluation Research

			1 2 3	* 1 2 3 4	FV 84	2	<u>}</u>	2	*	BEYOND
Task 3:	Evalu Detec Line	Evaluate In-Situ Methods for Remote Detection of Pressure Sensor/Sensing Line Degradation								
	3.1.	Complete lab tests of in-situ method for Foxboro sensor and issue letter report.				Seguira.				
	3.2.	Complete lab evaluation of noise analysis for detection of sensor/ sensing line degradation and issue letter report.		1						
	3.3	Complete demonstration of Foxboro sensor in-situ method in operating reactor.		•	1	Later L. Burking	The Late of the late of the			

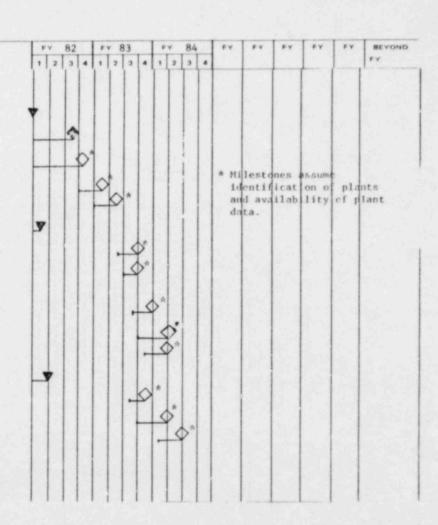


Safety Implications of Control Systems 80467



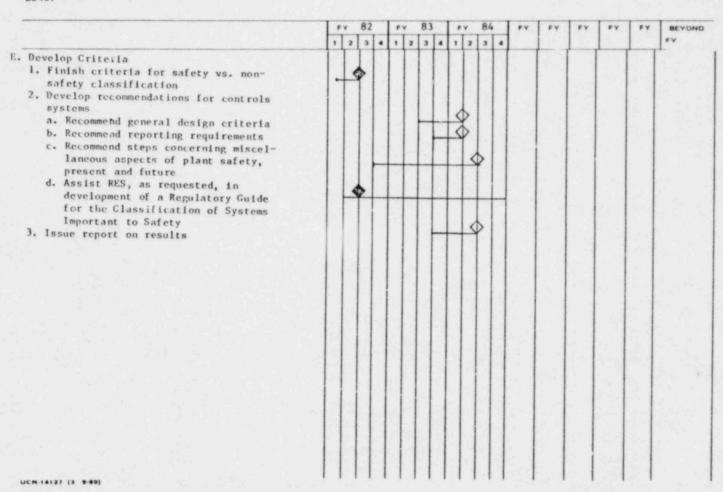
Safety Implications of Control Systems B0467

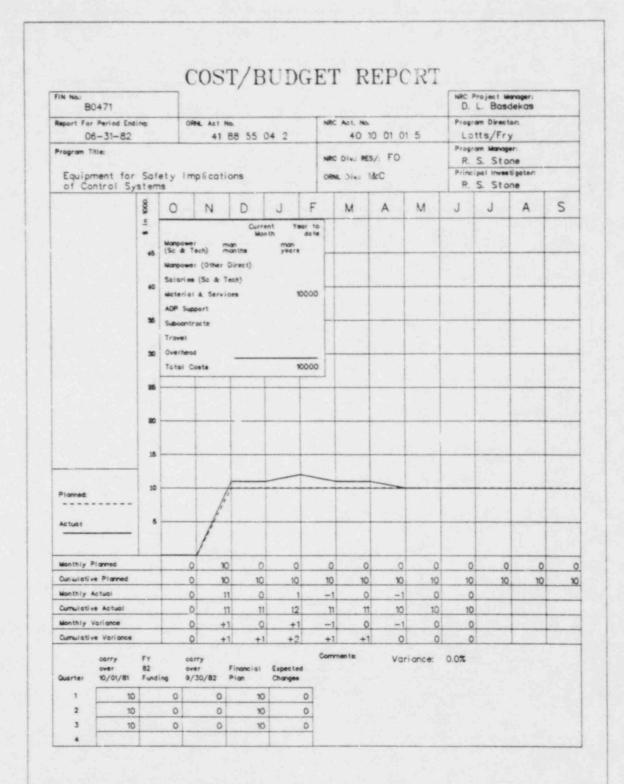
- D. Study Second Specific LWR Control System
 - 1. Document control system design
 - a. Complete catalog of needed information
 - Initiate requests for data from one more representative plant
 - c. Complete preliminary data collection
 - d. Complete 'inal data collection describing plant as installed
 - e. Issue documentary report.
 - 2. Develop and apply controls simulation
 - a. Complete generic model of LWR
 - b. Complete specialized model of 2nd PWR type
 - c. Implement hardware/software for 2nd PWR
 - d. Complete initial simulator study of failure modes of 2nd PWR, including overcooling
 - e. Apply simulation study to identify additional problems
 - f. Issue report on results.
 - 3. Perform FMEA of control systems
 - a. Develop and specify methodology
 - b. Complete initial FMEA for second representative PWR power plant
 - c. Complete final FMEA for second representative PWR power plant
 - d. Issue reports on results.



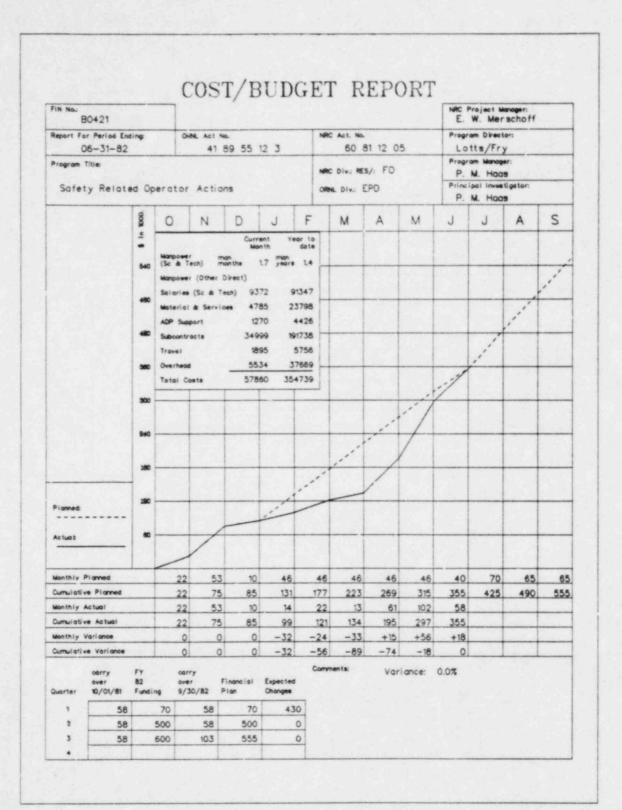
UFN 14127 (3 9-80)

Safety Implications of Contro! Systems B0467

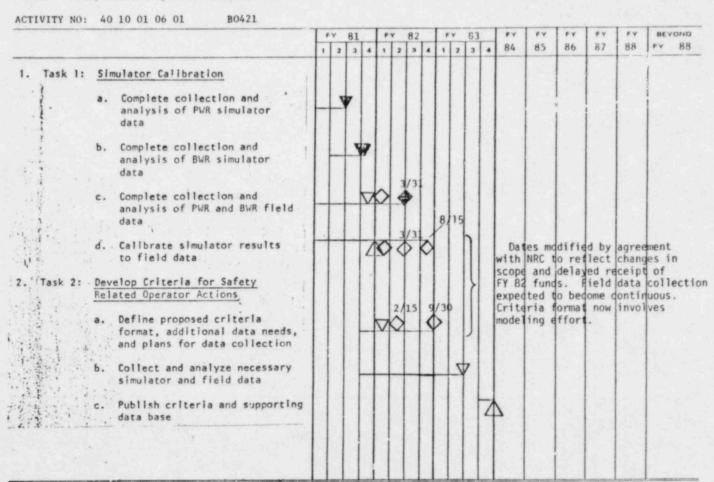




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TITLE: Safety Related Operator Actions



Collect and assess information on nuclear and non-nuclear industry practices industry practices industry practice including suggested revisions to ANSI/ANS 3.5 k. Analysis Techniques. Perform traditional task analysis perform a similar study using traditional task analysis					> 4	81	H	>	82			2 6	Y 20	FY FY	¥ 0		BEYOND FY 88
Collect and assess Information on nuclear and non-nuclear industry practices Industry practices Develop recommendations for future nuclear industry practice including suggested revisions to ANSI/ANS 3.5 Reform traditional task analysis for PWR scenarios used in Task i Demonstrate the use of PWR simulator data to supplement and validate traditional task analysis perform a similar study using traditional task analysis techniques and simulator data for the BWR scenarios used in Task I					-	0	-	~	6			420	+	+	+	+	1
Collect and assess Information on nuclear and non-nuclear industry practices Develop recommendations for future nuclear industry practice including suggested revisions to ANSI/ANS 3.5 Reform traditional task analysis Techniques used in Task i Demonstrate the use of PWR simulator data to supplement and validate traditional task analysis Perform a similar study using traditional task analysis techniques and simulator data for the BWR scenarios used in Task i	Task	3:		ulator Response Characteristics													
Develop recommendations for future nuclear industry practice including suggested revisions to ANSI/ANS 3.5 k Analysis Techniques. Perform traditional task analysis for PWR scenarios used in Task I Demonstrate the use of PWR simulator data to supplement and validate traditional task analysis Perform a similar study using traditional task analysis techniques and simulator data for the BWR scenarios used in Task I Task I	·		ė	Collect and assess Information on nuclear and non-nuclear industry practices		3,610											
a. Perform traditional task analysis for PWR scenarios used in Task i b. Demonstrate the use of PWR simulator data to supplement and validate traditional task analysis c. Perform a similar study using traditional task analysis techniques and simulator data for the BWR scenarios used in Task 1			٠	Develop recommendations for future nuclear industry practice including suggested revisions to ANSI/ANS 3.5		N	•										
Perform traditional task analyzis for PWR scenarios used in Task I Demonstrate the use of PWR simulator data to supplement and validate traditional task analysis Perform a similar study using traditional task analysis techniques and simulator data for the BWR scenarios used in Task I	Task	4		k Analysis Techniques											_		
Demonstrate the use of PWR simulator data to supplement and validate traditional task analysis Perform a similar study using traditional task analysis techniques and simulator data for the BWR scenarios used in Task 1	100	77	•	Perform traditional task analyzis for PWR scenarios used in Task 1	-	1	4										
Perform a similar study using traditional task analysis techniques and simulator data for the BWR scenarios used in Task 1	Mark 42 (200) - 41		ف			1	-			-					lon ster		
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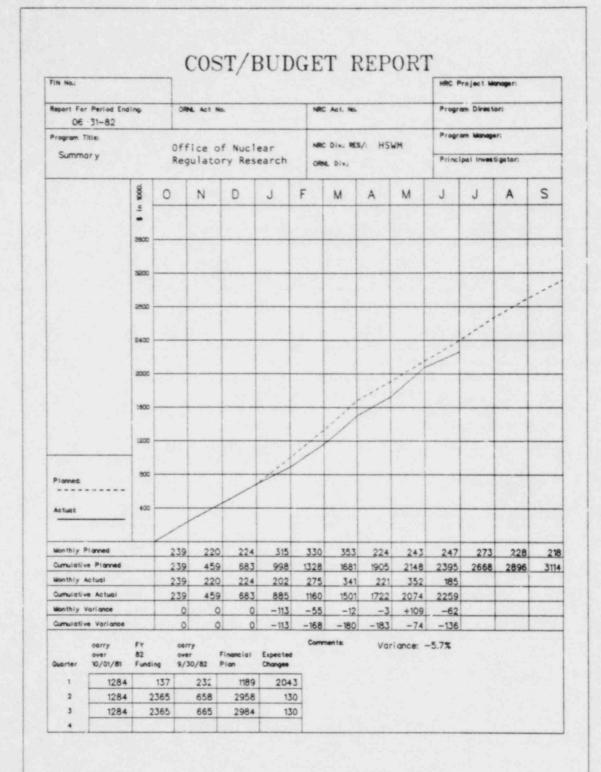
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a. Deter ance respondevent b. Ident shaping those addresexper the contown of the	Ify key performance ng factors (PSF); identify PSF which can be ssed by simulator iments within the scope of urrent program directed d criteria development n and perform simulator iments identified in sk 5.b, e.g., experiments termine the effects on tor performance of oms vs. event-oriented					\rightarrow \right							7	NO	TE:			d due Y 82	to fund	de-

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ask 6: Continued Simulator Experiments (Continued Simulator Experiments (Continued Simulator Experiments (Continued Simulator experiments continued simulator experiments to obtain data on operator performance under abnormal- emergency conditions c. Design, perform and analyze results of simulator experiments according to program plan in Subtasks 5 and 6 to be modifiled due to decrease in expected FY 82 funding.	ask 6: Continued Simulator Experiments (Continued Simulator Experiments b. Define a program plan for continued simulator experiments to obtain data on operator performance under abnormal— emergency conditions c. Design, perform and analyze results of simulator experiments according to program plan in Subtask 6.b Subtasks 5 and 6 to be modified due to decrease in expected FY 82 funding.	nts		FY 81	-	FY 8	82	*	83	*	*	*	*	**	5
b. b. Subtasks 5 decrease 11	b. Contasks Subtasks S decrease 1	Subtasks 5 decrease 1		0		~	*	-		78	85	86	87	88	24
Subtasks 5 decrease 1	Subtasks 5 decrease 1	Subtasks 5 decrease 1													
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UCN-14127 (3 9-80)

DIVISION OF HEALTH, SITING AND WASTE MANAGEMENT



COST/BUDGET REPORT NRC Project Managers C. Prichard 80454 Report For Period Ending: ORNE Act No. Program Director: 06-31-82 41 88 55 03 4 60 19 02 10 Lotts/Chester Program Manager: Program Title: HE DIV: RES/: HSHM H. I. Bowers Principal Investigator CONCEPT/OMCOST Code Development GRM DIN: ETD H. I. Bowers 1000 0 N D М A M A S 5 Manpower (Sc & Tech) man the -0.8 years 1.2 Manpower (Other Direct) Salaries (Sc & Tech) -5214 172 4964 Material & Services 583 7973 ADP Support 291 1527 -1331 30705 132880 Monthly Planned Cumulative Plannes 19 38 60 69 87 120 151 154 Monthly Actual 19 19 18 28 17 Cumulative Actual 19 38 123 60 78 106 131 138 133 Monthly Variance 0 0 0 +19 -25 -6 Cumulative Variance 0 0 0 +28 +36 +5 Variance: -7.0% Financial Expected Plan Changes over Finan 9/36/82 Plan 25 0 69 75 2 44 110 0 154 0 44 110 0 154 0

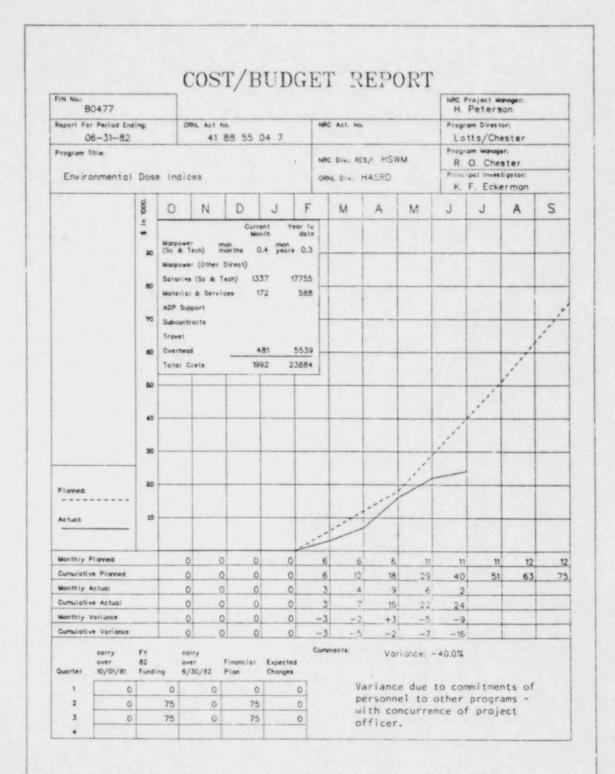
87 88 FY 89 A4 A4 A4 A4 CONCEPT/OMCOST Code Development ORNL No. 41 88 55 03 4 ACTIVITY NO: TITLE:

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		1	Complete draft report on power plant
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- Complete draft report on power plan O&M costs.
- Complete draft report on power plant capital investment costs.
- *3. Validate cost models in CONCEPT code.
- 4. Validate cost index data in CONCEPT code.

*Allestones 2 and 3 were re-scheduled because of delay in obtaining January, 1982, power plant cost data from DOE-NE.

UCN 14127 (3 9-80)



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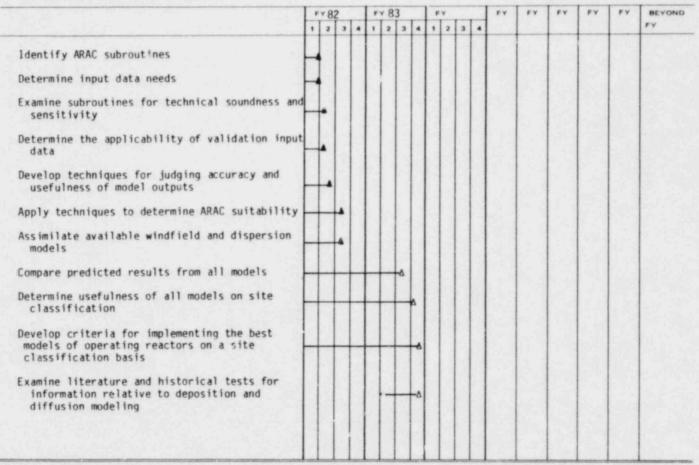
UCN-16127 (3 1-20)

COST/BUDGET REPORT R. F. Abbey B0446 Program Director: Report For Period Ending: ORNE Act No. 06-31-82 41 89 55 13 1 60 19 12 01 Lotts/Chester Program Manager: Program Title: MRC DIV. RES/: HSWM F. C. Kornegay Principal Investigator Evaluation of Atmospheric Dispersion Models ORNE DIVE EN F. C. Kornegay F S 0 N D M A 1.0 years 0.5 Marpower (Other Direct) Salaries (Sc & Yech) 6714 40399 701 24820 Material & Services ADP Support 36648 190290 Travel 1098 2389 17086 Overhead Total Costs 46435 273697 Monthly Planned 65 65 38 38 38 39 Cumulative Planned 76 90 155 220 285 323 361 399 514 437 475 Monthly Actual 54 25 59 26 46 46 Cumulative Actual 54 76 90 96 155 181 273 Monthly Variance -84 40 +8 +8 Cur stative Variance 0 -84 -124 -130 -142 -134 carry Variance: -31.6% over 10/01/81 over 9/30/82 Financial Expected Variance due to delay in model Funding Plan Changes evaluation task as requested by 375 12 0 387 288 NRC. Variance has decreased by 375 200 414 100 14.0 since last quarter. 161 375 100

3

TITLE: Evaluation of Atmospheric Dispersion Models

ACTIVITY NO: 60 19 12 01 (80446)



UCN-14127 (5 9 80)

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COST/BUDGET REPORT FIN NO. NRC Project Manager: Prichard/Cleary B0190 NRC Act. No. Program Director: Report For Period Ending: ORNE Act No. 06-31-82 41 88 55 01 3 60 19 32 02 Lotts/Chester Program Manager: Program Title: MRC DIV. RES/: HSWM R. B. Shelton Principal Investigator: D. M. Hamblin Forecasting Electricity Demand by States ORM DIV. EN 9000 S F A J J A D M M 0 N Current Month 0.6 years 0.5 Manpower (Other Direct) Salaries (Sc & Tech) 3432 42946 1346 5718 Material & Services 13329 ADP Support 2861 7303 Subcontracts 1099 4017 Travel 20701 2792 Overhead Total Costs 11534 Actuet Monthly Planned 20 10 18 Cumulative Planned 0 29 45 55 73 91 99 107 115 123 132 Monthly Actual 0 20 18 12 9 8 12 Cumulative Actual 0 29 35 53 65 74 82 94 Monthly Variance 0 0 0 -10 +8 -6 -9 0 +4 Cumulative Variance 0 0 0 -10 -8 -17 -17 -13 carry Variance: -12.1% Financial Expected
Plan Changee over 10/01/81 over 9/30/82 Variance temporary. Will be on Funding target next quarter. 2 20 0 22 110 2 2 100 0 102 30 3 2 100 0 102 30

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A. Data Base Maintenance B. Model Transfer C. Update Integrated System		ry 28%	f v 82	-		> 14		> 4	4	4.4	> 4	**	BEVOND
Model Transfer Update Integrated System			1 2 3	-	0	4	0				T		
Data Base Maintenance Model Transfer Update Integrated System				-1									
Wodel Transfer Update Integrated System	Α.	Data Base Maintenance	1	7									
Update Integrated System	. 8	Model Transfer	1	7									
		Update Integrated System	#	7									
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COST/BUDGET REPORT NRC Project Wanager: A. Brodsky B0475 Program Director: Report For Period Ending: ORNE Act No. NRC Act. No. 41 88 55 04 5 Lotts/Chester 06-31-82 Program Manager. Program Title: NAC DIV. RES/: HSWM K. F. Eckerman Principal investigator: Internal Dose for Specific Occupational Exposure Conditions ORNE DIV. HASRD K. F. Eckerman S A 0 D A Year to date Manpower man (Sc & Tech) months Manpower (Other Direct) Salaries (Sc & Tech) Material & Services ADP Support Subcontracts Total Costs 20 10 Planned: Actuat Monthly Planned Cumulative Planned 0 0 10 0 Monthly Actual 0 0 Cumulative Actual 0 0 0 Monthly Variance Cumulative Variance 0 0 0 0 Comments: Variance: -100.0% over 9/30/82 Financial Expected 10/01/81 Funding No requests received yet 0 0 0 for dose evaluations. 0 0 10 0 3 0 0 10 0

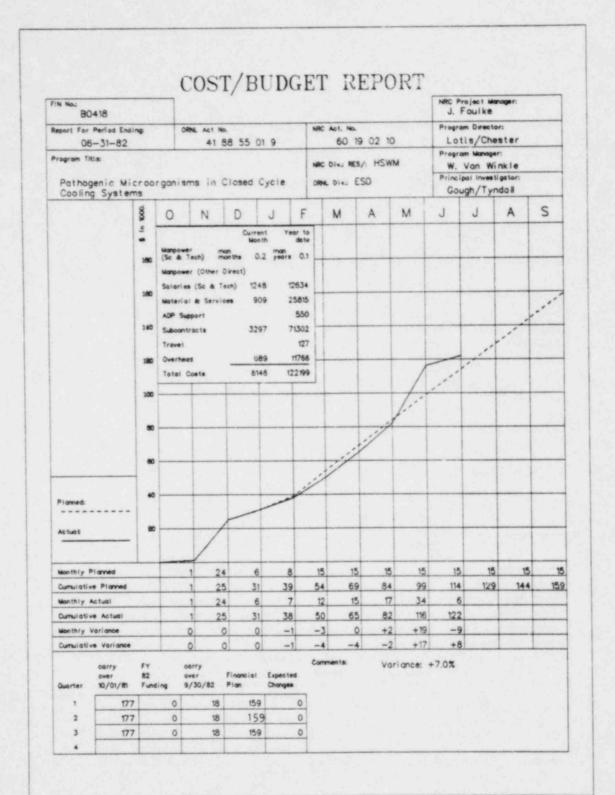
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Internal Dose for Specific Occupational Exposure Conditions (80475)

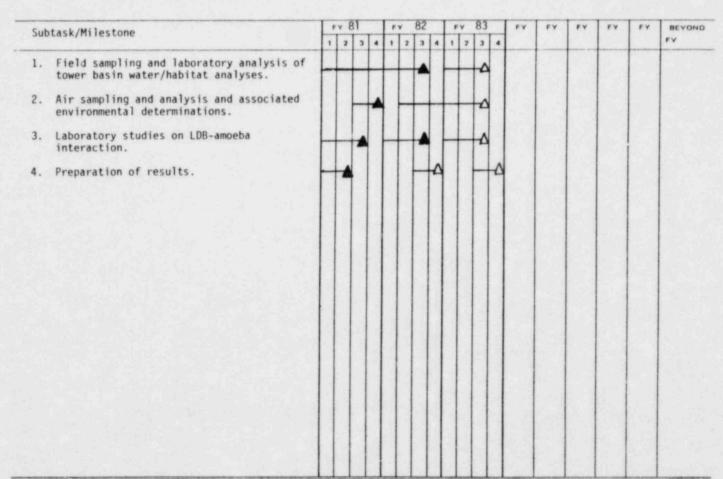
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TASK 1:	TASK 1: Evaluation of dose for specific requests		D					
TASK 2:	TASK 2: Annual report	4	A					
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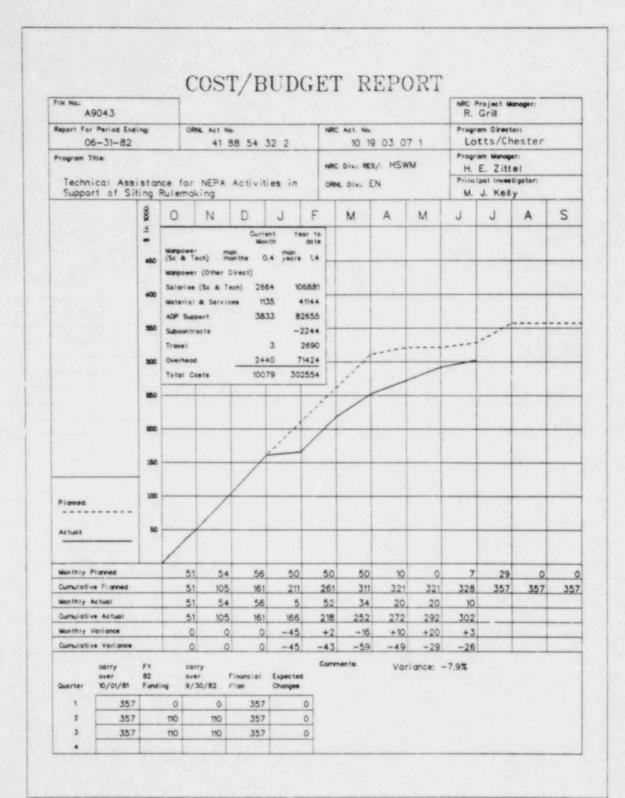
UCN-14127 (3 9-80)

COST/BUDGET REPORT NRC Project Manager: J. D. Foulke B0410 Report For Period Ending ORNE Act No. Program Director: NRC Act. No. 06-31-82 41 88 55 01 8 60 19 30 01 Lotts/Chester Program Wanager MRC DIV.: RES/: HSWM K. F. Eckerman Methods in Dosimetry for Nuclear Regulations Principal Investigator: ORNE DIV. HSRD K. F. Eckerman 1000 0 N D F A A S Year to (Sc & Tech) mon the mon years 0.4 Manpower (Other Direct) Salaries (Sc & Tech) 26030 -2723 Material & Services ADP Support 5920 Subcontracts 1192 Travel 1351 Overheed 12377 Total Costs 44151 Monthly Planned -80 0 Cumulative Planned 16 34 69 97 45 45 45 45 45 45 Monthly Actual 16 18 0 Cumulative Actual 16 34 41 48 50 Monthly Variance 0 0 0 -21 -26 -34 Cumulative Variance 0 0 -21 -47 -81 Comments: Variance: -2.2% Financial Expected Plan Changee 9/30/82 Plan 10/01/81 Funding 45 0 250 2 45 0 0 45 0 3 45 0 0 45 0



Pathogenic Microorganisms in Closed Cycle Cooling Systems



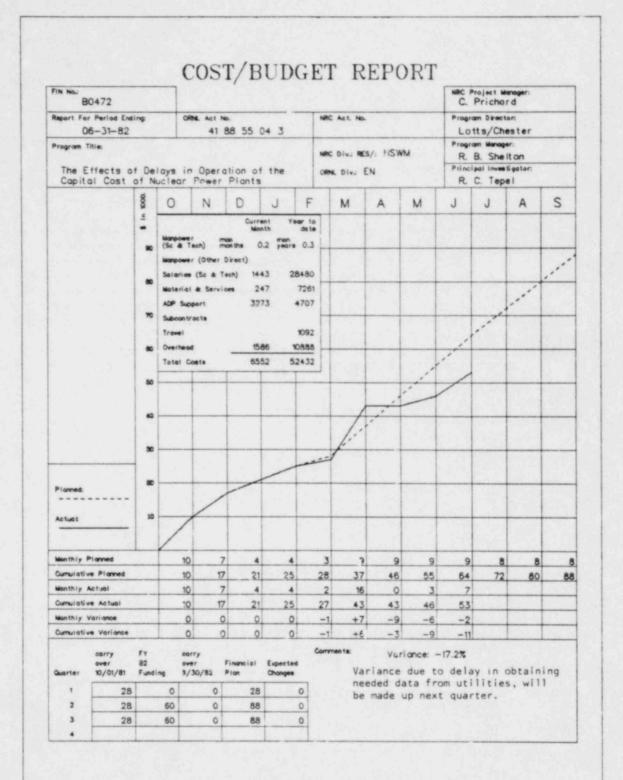


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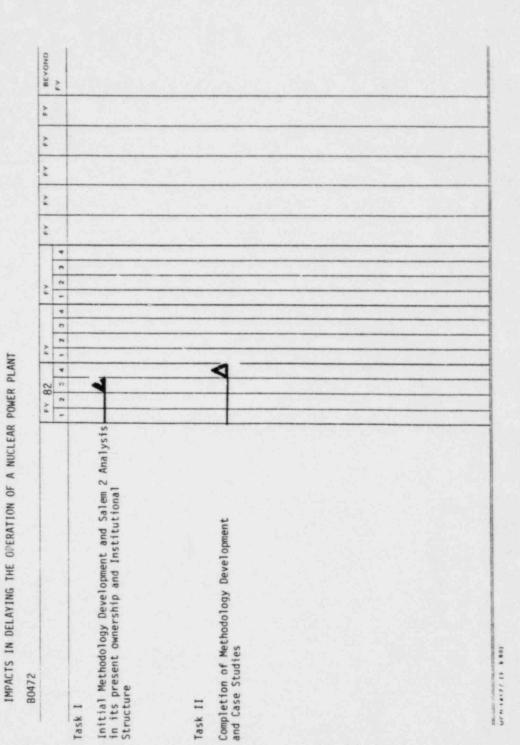
Title: Technical Assistance for NEPA Activities in Support of Siting Rulemaking A9043

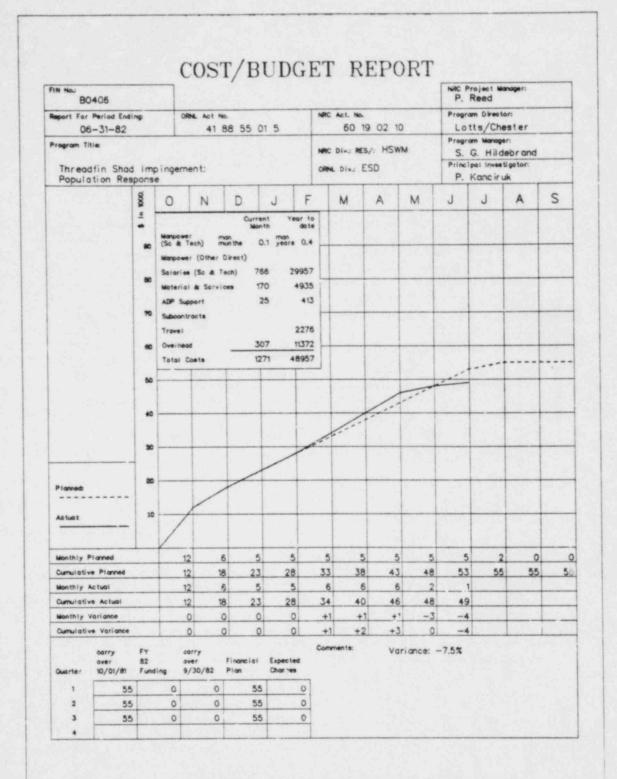
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Siting Rule Support Documents:						-								
 Protective Actions as a Factor in Power Reactor Siting. 						-								
a. Complete Draft. b. Issue Report.		P		9					<u>Los</u>					
 Effect of Population Alternatives on Availability of Acceptable Sites. 														
a. Complete Draft. b. Issue Report.		DD		*										
3. Evaluation of the Effects of Population Criteria														
a. Complete Draft. b. Issue Report.		D D	1	*										
*Reports delayed pending NRC review.														
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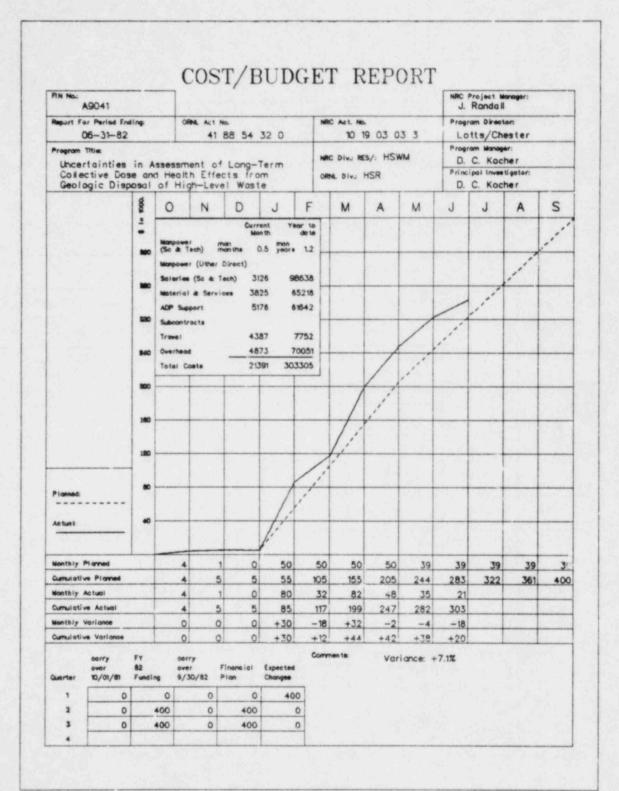


MILESTONE BAR CHART





Threadfin Shad Impingement: Population Response NRC Activity No. 60 19 02 10 BEYOND 1 2 3 4 1 2 3 4 1 2 3 4 FY Task (Subtask H) Determine potential of downscan sonar as a technique for assessing nuclear energy related impacts on fisheries. 1. Acquire downscan sonar literature: USA Foreign 2. Evaluate existing systems from literature 3. Contact key persons (site visits where necessary) developing and utilizing downscan sonar to ascertain recent advances and to define field strengths and weaknesses. 4. Draft report prepared and distributed for review. 5. Final report to NRC.



TITLE: Uncertainties in Assessment of Long-Term Collective Dose and Health Effects from Geologic Disposal of High-Level Waste

ACTIVITY NO.: 10 19 03 03 3 A9041

FY 81 FY 82 FY 83 FY FY FY FY BEYOND FY

12. SUBTASK/MILESTONE SCHEDULE

1 2 3 4 1 2 3 4 1 2 3 4

- Task 1. Assessment of Uncertainties Associated with Geologic Disposal of High-Level Waste
 - Perform literature review to identify important processes and parameters for estimating uncertainties
 - Prepare ORNL/NUREG/TM progress report describing results of Task 1.a.
 - Analysis of uncertainties in groundwater transport from repository to biosphere
 - Analysis of uncertainties in long-term collective dose and health effects from transport through biosphere to man
 - Prepare ORNL/NUREG report on analyses of uncertainties performed in Tasks 1.c. and 1.d.
- Task 2. Symposium on Geologic Waste Disposal
 - Plan symposium on uncertainties associated with regulation of geologic disposal of high-level waste
 - b. Host symposium
 - c. Publish symposium proceedings

TITLE: Uncertainties in Assessment of Long-Term Collective Dose and Health Effects from Geologic Disposal of High-Level Waste

ACTIVITY NO.: 10 19 03 03 3 A9041

			F	Y 81			FY	32		FY	83		1	Y	FY	FY	FY	FY	BEYOND
12.	SUB	TASK/MILESTONE SCHEDULE	1	2	3	4 1	2	3	4	1	2	3 4							FY
Task	3.	Review currently available information regarding uncertainties in the geo- sciences relevant to licensing of HLW repositories, and assess the value of 10 CFR 60 in reducing or compensating for those uncertainties																	
	a.	Review earth sciences literature on factors affecting uncertainty in doing repository assessments			-			Y			-								
	b.	Evaluate the importance of the uncertainties identified in Task 3.a.					-		7	7									
	с.	Assess the effectiveness of 10 CFR 60 in reducing or compensating for uncertainties identified in Task 3.b.					-		7	7									
Task	4.	Using the results of Task 3 and other pertinent information, assess the value of past geologic records as indicators of future site evolution, emphasizing the effect of such evolution on repository performance							7	7									
Task	5.	Provide technical assistance for pre- paring contingency HLW radiological criteria				1													
	a .	Peview past radiological standards set by NRC, EPA, and ICRP for waste manage- ment		-		-						-							

TITLE: Uncertainties in Assessment of Long-Term Collective Dose and Health Effects from Geologic Disposal of High-Level Waste

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12. SU	18	12. SUBTASK/MILESTONE SCHEDULE	-	3	*	-	~	4	-	1 2 3 4 1 2 3 4 1 2 3	*						7.4
Task 5.		Task 5. (continued)		-													
Ď.	3	 Beview NRC's evaluation of the EPA HLW radiological standard 						_									
Ü		Summarize similarities and differences in approaches taken by NRC, EPA, and ICRP in setting radiological standards for radioactive waste disposal															

d. Assess advantages and disadvantages of the approaches summarized in Task 5.c.

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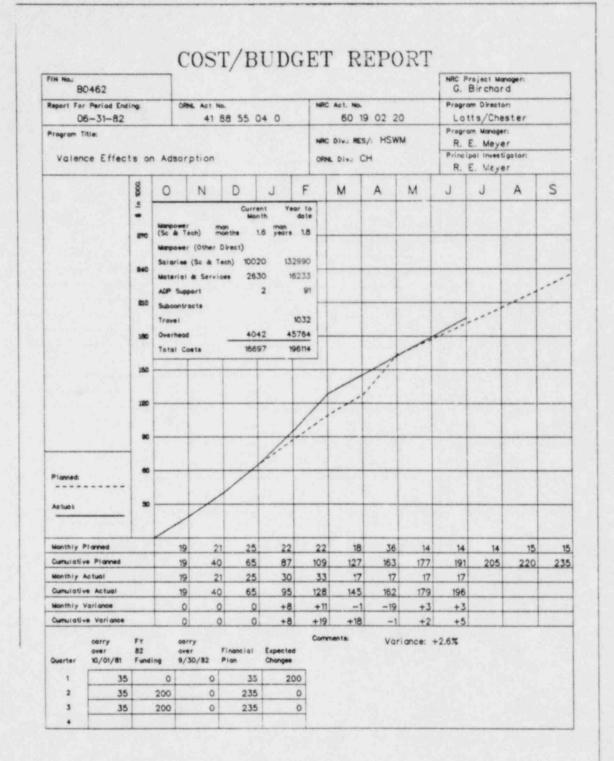
Heet with NRC personnel to plan remaining work on preparing contingency HLW radio-logical criteria

Prepare the contingency HLW radiological criteria

Short-Term Technical Assistance Task 6.

At request of NRC, provide general technical assistance on waste management matters relating to Tasks 1-5 g.

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TITLE: Valence Effects on Adsorption Fin. No: B0462 BEYOND FY 1. Task 1: Apparatus and Materials a. Initial porous electrode configurations b. Initial cells for study of direct reaction of nuclides with minerals, e.g., FeS, c. Revision of apparatus Task 2: Sorption Studies a. Sorption of technetium, neptunium, etc. on single minerals b. Sorption on mixtures (includes rocks and other formation materials) c. Draft copies of topical reports* Task 3: Direct Reaction Studies a. FeS₂ with technetium and other nuclides b. Other materials c. Draft copies of topical reports* *Also to be assembled into external publications.

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