APPENDIX

U. S. NUCLEAR REGULATORY COMMISSION REGION IV

NRC Inspection Report: 50-458/82-10

Docket: 50-458

Category A2

Licensee: Gulf States Utilities Post Office Box 2951 Beaumont, Texas 77704

Facility Name: River Bend, Unit 1

Inspection at: River Bend Site

Insp tion conducted: August 23-27, 1982

Gilbert, Reactor Inspector, Engineering Section Date

Chief, Reactor Project Section B

Inspection Summary:

Inspection on August 23-27, 1982 (Report 50-458/82-10)

Areas Inspected: Routine, unannounced inspection of licensee action on previous inspection findings; site tour; observation of work and review of records for welding and nondestructive examination (NDE) of safety-related piping. The inspection involved 36 inspector-hours onsite by one NRC inspector.

Results: In the areas inspected, no violations or deviations were identified.

DETAILS

1. Persons Contacted

Principal Licensee Personnel

*P. D. Graham, Director, QA *G. R. Kimmell, QA Engineer

*R. Kerr, QA Engineer, Management Analysis Corporation *M. Walton, Assistant Project Engineer

*K. C. Hodges, QA Engineer

*G. Davis, Engineer - Welding Specialist

Stone & Webster (S&W) Personnel

*R. L. Spence, Superintendent, Field Quality Control (FQC)
*A. Clawson, Inspection Supervisor, FQC

*W. R. Whitley, Assistant Superintendent, FQC R. Ferguson, QC Engineer - ASME
*V. Barton, FQC Inspection Supervisor
D. Johnson, Piping Supervisor

*C. A. Goody, Resident Manager *A. Kamdar, Assistant Superintendent of Engineering

Other Personnel

P. Morrison, Authorized Nuclear Inspector

The NRC inspector also interviewed other licensee and contractor employees during the course of the inspection.

*Denotes those attending the exit interview

2. Licensee Action on Previous Inspection Findings

(Closed) Violation (458/8205-01): Failure to follow welding instructions.

The NRC inspector reviewed Nonconformance and Disposition Report No. 2463 and Engineering and Design Coordination Report Nos. C-11315 and C-11404 which were issued to disposition the procedural violation and to change and clarify Weld Technique Sheet W3-NSSS-54. The NRC inspector also verified that the welding supervisor and field quality control inspectors were inspecting the dimetrics machine welding to ensure that the amperage and travel speed are in accordance with approved weld parameters.

This item is considered closed.

3. Site Tour

The NRC inspector toured the reactor building and auxiliary building for Unit 1 to observe construction in progress and to inspect housekeeping.

No violations or deviations were identified.

4. Safety-Related Pipe Welding and Nondestructive Examination

a. Observation of Welding Activities

The NRC inspector observed the welding and quality control activities associated with fabricating the following safety-related piping welds:

Weld	Control Drawing	System	Class
FW-A8	I-RCS-800-B	Reactor Coolant-Recirculation	T
FW-A9	1-RCS-800-B	Reactor Coolant-Recirculation	1
FW-R54	1-RCS-800-B	Reactor Coolant-Recirculation	1
FW-003	1-SFC-044-A	Fuel Pool Cooling and Clean-up	3

In the areas reviewed for the above welds, the entries on the weld data cards were consistent with the status of the welds.

b. Observation of Nondestructive Examination Activities

The NRC inspector observed the liquid penetrant examination of a Class 1 main steam system pipe end preparation repair on Piece 5 of weld FW-B7 on Control Drawing 1-MSS-700-A and the liquid penetrant examination of the following completed welds:

Weld	Control Drawing	System	Class
FW-A9	I-RCS-800-B	Reactor Coolant-Recirculation	1
FW-R54	1-RCS-800-B	Reactor Coolant-Recirculation	1
FW-003	1-SFC-044-A	Fuel Pool Cooling and Clean-up	3

The examinations were performed by S&W FQC personnel certified as Level II examiners. In the areas observed, the examinations were performed consistent with Liquid Penetrant Procedure QAD 9.32 RB, Revision 0.

c. Review of Records

The NRC inspector reviewed the weld history and nondestructive examination documentation related to the fabrication of the following 15 completed welds.

Weld	Control Drawing	System	Class
FW-002	1-FWS-047-A	Feedwater	1
FW-002 FW-B4	1-FWS-048-A 1-RCS-900-A	Reactor Coolant-Recirculation	1

Weld FW-IA FW-1A FW-002 FW-001 FW-003 FW-002 FW-004 FW-002 FW-006 FW-008	Control Drawing 1-ICS-007-A2 1-ICS-007-A3 1-ICS-008-B 1-ICS-004-A2 1-RHS-008-B 1-RHS-008-B 1-SAS-998-A 1-SAS-998-A 1-SFC-001-A 1-SFC-001-A 1-SVV-035-A	Reactor Core Isolation Cooling Reactor Core Isolation Cooling Reactor Core Isolation Cooling Reactor Core Isolation Cooling Residual Heat Removal Residual Heat Removal Service Air System Service Air System Fuel Pool Cooling and Clean-up Fuel Pool Cooling and Clean-up Main Steam Safety & Relief Valve	Class 1 1 2 2 2 2 2 2 2 2 3 3
FW-008	1-SVV-035-A	Main Steam Safety & Relief Valv	es 3
FW-008	1-SVV-036-A	Main Steam Safety & Relief Valv	es 3

In the areas reviewed, the records indicated that specified inspections were completed; the records reflected adequate weld quality; and the weld history records were adequate.

No violations or deviations were identified.

5. Exit Meeting

The NRC inspector met with licensee representatives (denoted in paragraph 1) and R. L. Brown (NRC Resident Reactor Inspector) on August 27, 1982, and summarized the scope and findings of the inspection.