DCS	Nos.	50029	820504	
			820508	820629
			820527	

U.S. NUCLEAR REGULATORY COMMISSION

Region I

Report No. 50-29/82-08

Docket No. 50-29

License No. DRP-3

Priority

Category C

Licensee: Yankee Atomic Electric Company 1671 Worcester Road Framingham, Massachusetts 01701

Facility Name: Yankee Nuclear Power Station

Inspection at: Rowe, Massachusetts

Inspection conducted: July 9-30, 1982

Inspectors:

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Minum S. O. Collins, Senior Resident Inspector

B 13 BZ date signed

Approved by: Projects Reacto Section 1A

date signed

Inspection Summary:

Inspection on: July 9-30, 1982

Areas Inspected: Routine, onsite regular and backshift inspection by the resident inspector (43 hours). Areas inspected included previous inspection items; reviews of plant operation; review of events; review of Licensee Event Reports; IE Bulletin followup and review of onsite review committee activities.

Noncompliance : One: Failure to properly control an access controlled area Portal (detail 3.B.2).

DETAILS

Persons Contacted 1. Plant Operations

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- *H. Autio, Plant Superintendent
- W. Billings, Chemistry Manager E. Chatfield, Training Manager
- *B. Drawbridge, Technical Director
- L. French, Plant Engineer T. Henderson, Reactor Engineering Manager
- K. Jurentkuff, Assistant Plant Operations Manager
 W. Loomis, Instrument and Control Supervisor
 *R. Sedgwick, Security Supervisor

- *N. St. Laurent, Assistant Plant Superintendent
- *J. Staub, Technical Services Manager

The inspector also interviewed other licensee employees during the inspection, including members of the Operations, Health Physics, Instrument and Control, Maintenance, Reactor Engineering, Security and General Office Staffs.

Quality Assurance

L. Reed, Operational Quality Assurance Coordinater

Yankee Atomic Electric Company

*D. E. Moody, Manager of Operations

*Denotes those present at exit interview on August 12, 1982

2. Licensee Action on Previous Inspection Findings

- a. (closed) Inspector Follow Item (29/81-06-03) Revise OF-7200 To Clearly Define Acceptance Criteria for Rod Bowing. The inspector reviewed OP-7200, Receiving, Unloading and Inspecting New Reactor Fuel, Revision 5, dated July, 1982 and noted that Part C, New Fuel Inspection Acceptance Criteria now contains a maximum allowable fuel rod bow specification. This item is closed.
- b. (closed) Violation (29/82-04-01) Failure to Barricade, Post and Control a High Radiation Area. The inspector reviewed licensee response letter FYR 82-73, dated July 8, 1982 and determined that the corrective steps taken were adequate. The inspector also reviewed General Employee Training lecture attendance records and lesson plan dated July 8 and 9, 1982 and determined that IR 82-04 Appendix A Violation was included and scheduled for review with plant personnel. This item is closed.

3. Review of Plant Operations

(C)**

- A. <u>Daily Inspection</u> The inspector verified the following by direct observation of activities, tours of the facility, discussions with plant personnel, independent verification, and facility record review:
 - 1. Control room activities were observed to verify proper manning and access control; adherence to approved procedures; adherence to Limiting Conditions for Operation (LCO's), ESF status and selected value confirmations using a unit specific checklist; selected instrument and recorder trace review; control room board annunciator status and followup action review; nuclear instrumentation (N/I) and reactor protection system (RPS) operability verification; conformance with shutdown margin limits; verification of containment status; primary vent stack trace review and release followup; verification of onsite and offsite emergency power source availability; control room documents, including operator logs, maintenance and surveillance documentation and operating orders were reviewed to note trends, apparent anomolies, rout he operations, and establish items requiring inspector followup.
 - 2. During daily entry and egress from the protected area (PA) security activities were observed to verify access controls in conformance with the security plan for personnel, packages, vehicles, guard manning and conduct; selected PA barriers, and gates were examined; isolation zone conditions were observed; and licensee monitoring for radioactive materials prior to personnel, materials and equipment release for unrestricted use was monitored during egress from the PA. These checks were performed on the following dates: 7/12, 7/14, 7/16, 7/20, 7/23, 7/26, and 7/30.

No inadequacies were identified.

3. cont.

B. Biweekly Inspection

 A review of the licensees sampling program was conducted by monitoring results of liquid and gaseous samples during the period to verify conformance with regulatory requirements; and boric acid tank (BAT) level and sample results were reviewed for conformance with technical specifications on the following dates: 7/14, 7/21, and 7/26.

No inadequacies were identified.

2. Accessible facility areas were toured to make an independent assessment of plant and equipment. On a sampling basis the following items were observed or verified: condition of selected vital and access controlled barriers; radiation work permit completion and use; protective clothing and where applicable, proper respirator use; personnel monitoring practices; operational status of selected personnel monitors, area radiation monitors and air monitors; equipment tagout sample to verify LCO compliance for equipment out of service; plant housekeeping and cleanliness conformance with approved programs, and communication system operability.

Inspector tours included the following areas: Control room, turbine building, switchgear room, screenwell house, spent fuel pit, primary auxiliary building and safety injection building.

Except for the following the inspector had no further questions:

PAGES 5 & 6, CONTAINING SAFEGUARDS INFORMATION NOT FOR PUBLIC DISCLOSURE, IS INTENTIONALLY LEFT BLANK.

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Inspector Review of Plant Events

On June 29, 1982 during normal operation in Mode 1 the plant tripped when a lightening strike on the distribution system resulted in loss of the offsite power line Z-126 (Harriman Station Line). All systems responded normally. The plant returned to power on June 30, 1982. Prior to the trip the plant had operated at power for 289 days, a plant record.

During the inspection period the plant operated continually at power.

No inadequacies were identified.

5. Review of Licensee Event Reports (LERs)

A. LERs submitted to NRC:RI were reviewed to verify that the details were clearly reported, including accuracy of the description of cause and adequacy of corrective action. The inspector determined whether further information was required from the licensee, whether generic implications were indicated, and whether the event warranted onsite followup. The following LERs were reviewed.

LER No.	Date of Event	Date of Report	Subject
82-08/03L	05/04/82	06/03/82	Underground Fire Main Fitting Leak
82-09/03L	05/08/82	06/07/82	Inoperable containment Isolation Valve AS-V-719
82-12/03L	05/27/82	06/25/82	Removal of SG No. 4 Blow- down Monitor from Service
82-15/03L	06/04/82	07/02/82	Valve SI-MOV-S17 Breaker Open
82-19/03L	06/29/82	07/29/82	Loss of Z-126 Line Due to Electrical Storm

No inadequacies were identified

6. IE Bulletin/Circular/Immediate Action Letter Followup

A. For the IE Bulletin (IEB) listed below the inspector ascertained whether the following actions taken by the licensee met Bulletin requirements and licensee commitments: Written response was within the time period stated in the Bulletin, includes the information required to be reported, includes adequate corrective action commitments based on information presented in the Bulletin and licensee's response, verified licensee management forwarded copies of the written response to appropriate onsite management representatives, that information discussed in the licensee's written response was accurate, and corrective action taken by the licensee was as described in the written response. * * *

> -- IEB 82-01, Alteration of Radiographs of Welds in Piping Subassemblies, Rev. 1, dated March 31, 1982.

The inspector reviewed applicability of the subject bulletin and noted in Table 1 that no written response was required to this bulletin.

-- IEB 82-02, Degradation of Threaded Fasteners in the Reactor Coolant Pressure Boundary of PWR Plants, dated May 30, 1982.

The inspector reviewed the licensee's response FYR 82-81 dated July 30, 1982. The inspector determined that the licensee made an adequate search through archival records to provide the information requested by the bulletin and that the response adequately addressed the bulletin requirements.

No inadequacies were identified.

7. Onsite Review Committee

On the following date the inspector observed a meeting of the Yankee NPS onsite review committee to ascertain that the provisions of Technical Specification 6.5.1 were met.

- PORC meeting conducted July 16, 1982

No inadequacies were identified.

8. Management Meetings

At periodic intervals during the course of the inspection, meetings were held with senior facility management to discuss the inspection scope and preliminary findings of the resident inspector. A summary of findings was also provided to the licensee at the conclusion of the report period.