

## UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

February 9, 1994

Docket No. 52-004

Mr. Patrick W. Marriott, Manager Licensing & Consulting Services GE Nuclear Energy 175 Curtner Avenue San Jose, California 95125

Dear Mr. Marriott:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION (RAI) REGARDING THE SIMPLIFIED

BOILING WATER REACTOR (SBWR) DESIGN

The staff has determined that it needs additional information to support its review activities related to the SBWR design certification. Some additional information on the SBWR testing programs is needed (Q900.62)\*. Please provide a written response to the enclosed question within 90 days of the date of this letter.

You have previously requested that portions of the information submitted in the August 1992, application for design certification of the SBWR plant, as supplemented in February 1993, be exempt from mandatory public disclosure. The staff has not completed its review of your request in accordance with the requirements of 10 CFR 2.790; therefore, that portion of the submitted information is being withheld from public disclosure pending the staff's final determination. The staff concludes that this RAI does not contain those portions of the information for which you are seeking exemption. However, the staff will withhold this letter from public disclosure for 30 calendar days from the date of this letter to allow GE the opportunity to verify the staff's conclusions. If, after that time, you do not request that all or portions of the information in the enclosure be withheld from public disclosure in accordance with 10 CFR 2.790, this letter will be placed in the NRC's Public Document Room.

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<sup>\*</sup>The number in parentheses designates the tracking number assigned to the question.

This RAI affects nine or fewer respondents, and therefore is not subject to review by the Office of Management and Budget under P.L. 96-511.

If you have any questions regarding this matter, please contact me at (301) 504-1178 or Mr. Son Ninh at (301) 504-1125.

## Sincerely,

(Original signed by)

Melinda Malloy, Project Manager Standardization Project Directorate Associate Directorate for Advanced Reactors and License Renewal Office of Nuclear Reactor Regulation

Enclosure: RAI on the SBWR Design

cc w/enclosure: See next page

Distribution (w/enclosure):

\*Central File PDST R/F MMalloy SNinh \*PDR RHasselberg JNWilson RBorchardt DCrutchfield/WTravers TMurley/FMiraglia,12G18 WRussell, 12G18 WHodges/BDLiaw, 7D26 AThadani/MVirgilio, 8E2 DMcPherson, 8E2 JNorberg, 7E23 DFischer, 7E23 RBarrett, 8H2 JMonninger, 8H2 MSnodderly, 8H2 RElliott, 8H2 RJones, 8E23 ALevin, 8E23 RCaruso, 8E1 SHou, 7H15 SA11, 7H15 GBagchi, 7H15 SLee, 7H15 BSheron/TKing, NLS007 LShotkin, NLN353 TLee, NLN353 JHan, NLN353 FEltawila, NLN344 Clinkler, NLN344 YChen, NLN344 ANotafrancesco, NLN344 WDean, 17G21 GSuh (2), 12E4 MFinkelstein, 15B18 JMoore, 15B18 ACRS (11)

\*To be held for 30 days

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OFFICIAL DOCUMENT NAME: SBWR9410.MM

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Mr. Steven A. Hucik GE Nuclear Energy 175 Curtner Avenue, MC-780 San Jose, California 95125

Mr. Frank A. Ross Program Manager, ALWR Office of LWR Safety & Technology U.S. Department of Energy NE-42 19901 Germantown Road Germantown, Maryland 20874

Mr. Victor G. Snell, Director Safety and Licensing AECL Technologies 9210 Corporate Boulevard Suite 410 Rockville, Maryland 20850

## REQUEST FOR ADDITIONAL INFORMATION (RAI) ON THE SIMPLIFIED BOILING WATER REACTOR (SBWR) DESIGN

## Wetwell-To-Drywell Vacuum Breaker Tests

- 900.62 The staff needs the following information in order to assess the adequacy of the wetwell-to-drywell vacuum breaker test program:
  - a. Drawings of the vacuum breaker and instrumentation.
  - b. Purchase Specification No. 25A5388, Revision 1, "Vacuum Breaker (Prototype)."
  - c. NEBO Engineering Operating Procedure No. 35-3.00, "Engineering Tests."
  - d. Detailed specification of the conditions, both normal and design basis, under which the vacuum breakers will be required to operate.
  - e. Detailed test procedures for the tests listed in the test matrix.
  - f. Any information appropriate to ensuring that the data obtained on the reliable performance of the prototypical valve are applicable to (or will be bounding with respect to) the valves to be incorporated into the certified plant design.