

LICENSEE EVENT REPORT

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

LICENSEE CODE: NYJAF1; LICENSE NUMBER: 00-0000-0000; LICENSE TYPE: 41111; CAT 58: 45

REPORT SOURCE: L; DOCKET NUMBER: 05000333; EVENT DATE: 071382; REPORT DATE: 083082

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES: 14-Day Follow-up to LER-82-034/03L-0. Instrument flow components drift resulting in non-conservative calculation of core thermal power. Rated thermal power limit of license TS 1.0.N was exceeded on approximately 75 days of Cycle 5 operation. See attachment for additional details.

SYSTEM CODE: CH; CAUSE CODE: B; CAUSE SUBCODE: A; COMPONENT CODE: INSTRU; COMP. SUBCODE: T; VALVE SUBCODE: Z; LER/RO REPORT NUMBER: 82; EVENT YEAR: 82; SEQUENTIAL REPORT NO.: 034; OCCURRENCE CODE: 03; REPORT TYPE: X; REVISION NO.: 1; ACTION TAKEN: EX; EFFECT ON PLANT: Z; SHUTDOWN METHOD: Z; HOURS: 0000; ATTACHMENT SUBMITTED: Y; NPRO-4 FORM SUB.: N; PRIME COMP. SUPPLIER: N; COMPONENT MANUFACTURER: B080

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS: Investigation of the event showed instrument drift as recorded would result in a non-conservative calculation of approximately 1.1% in core thermal power. See attachment for additional details.

FACILITY STATUS: G; % POWER: 000; OTHER STATUS: NA; METHOD OF DISCOVERY: B; DISCOVERY DESCRIPTION: Surveillance

ACTIVITY CONTENT RELEASED OF RELEASE: Z; AMOUNT OF ACTIVITY: NA; LOCATION OF RELEASE: NA

PERSONNEL EXPOSURES NUMBER: 000; TYPE: Z; DESCRIPTION: NA

PERSONNEL INJURIES NUMBER: 000; DESCRIPTION: NA

LOSS OF OR DAMAGE TO FACILITY TYPE: Z; DESCRIPTION: NA; PDR ADOCK 05000333

PUBLICITY ISSUED DESCRIPTION: N; PDR: 8209090254 820830

NRC USE ONLY

POWER AUTHORITY OF THE STATE OF NEW YORK  
JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

ATTACHMENT TO LER 82-034/03L-0

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Event Description and Probable Consequences:

While performing a follow-up calibration check (Cycle 5 increased surveillance, LER-82-002) of the feedwater instrumentation, it was determined that the two (2) feedwater flow transmitters were out of procedural tolerance. Analysis and review of plant logs determined that the rated core power had been exceeded at some time on each of approximately seventy-five (75) days of Cycle Five. The maximum recorded power level was approximately 2464 MWt compared to a limit of 2436 MWt as contained in Technical Specification paragraph 1.0.N.

Since transient analyses are performed from a core power level of 2535 MWt, the reactor was not operated in a manner less conservative than that assumed in the analyses.

Cause Description and Corrective Action:

During the increased frequency surveillance calibration check, transmitter 'B' (which had 0.9% "as found" out of tolerance factor) showed an instability over the calibration range while being adjusted. The sensor strain gage was replaced due to instability and satisfactory adjustment was achieved.

Plant data showed transmitter 'B' to have been stable and performing tracking of the feedwater flow changes during the period preceding the calibration check. It appeared the instability occurred during the July 11, 1982 plant shutdown. Investigation is continuing to approximate the occurrence of the event. If the exact time of the occurrence can be achieved the LER will be revised to indicate the correction.

Two (2) differential pressure transmitters were installed July 14, 1982 to monitor the behavioral characteristic of the A and B feedwater transmitters. Follow-up comparison check of the transmitters using the two independent transmitters shows them to be within tolerance at this time. If a long-term drifting or instability characteristic is discovered the transmitters will be replaced by the end of the next refueling outage.