



Commonwealth Edison
1400 Opus Place
Downers Grove, Illinois 60515

February 24, 1994

Dr. Thomas E. Murley, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Attention: Document Control Desk

Subject: Additional Information Pertaining to the Byron/Braidwood Proposed License Amendment Regarding Steam Generator Tube Sleeving

Byron Station Units 1 and 2
NPF-37/66; NRC Docket Nos. 50-454/455

Braidwood Station Units 1 and 2
NPF-72/77; NRC Docket Nos. 50-456/457

- References:
- (a) J. Bauer letter to T. Murley dated August 13, 1993, transmitting Application for Amendment to Facility Operating Licenses for Byron and Braidwood Stations pertaining to Steam Generator Tube Sleeving Methodology
 - (b) Teleconference on February 18, 1994, between Commonwealth Edison Company and the Nuclear Regulatory Commission concerning the Proposed Sleeving Amendment
 - (c) R. Assa letter to D. Farrar dated February 22, 1994, regarding the NRC's Steam Generator Sleeving Review

Dear Dr. Murley:

Commonwealth Edison Company (CECo) and the Nuclear Regulatory Commission (NRC) participated in a teleconference on February 18, 1994 to further discuss issues concerning the Byron and Braidwood proposed license amendments transmitted by Reference (a). Following this discussion, the NRC transmitted a letter (Reference (c)) stating that the NRC Technical Staff found the subject license amendments to be acceptable provided four contingency actions, which will be stated in the forthcoming Safety Evaluation Report, are met. CECo was requested to make a formal commitment to address these four contingencies.

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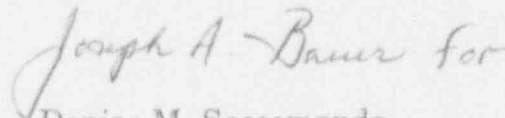
CECo agrees to the contingencies as stated in Reference (c). These contingencies are summarized below.

1. Amend the Byron and Braidwood licenses to reflect a primary-to-secondary leakage limit of 150 gallons per day through any one steam generator.
2. Amend the Byron and Braidwood licenses to reflect an inservice inspection of a minimum of 20 percent of a random sample of the sleeves for axial and circumferential indications at the end of each cycle. In the event that an imperfection of 40 percent or greater depth is detected, an additional 20 percent (minimum) of the unsampled sleeves should be inspected, and if an imperfection of 40 percent or greater depth is detected in the second sample, all remaining sleeves should be inspected.
3. Add a condition to the Byron and Braidwood licenses to conduct additional corrosion testing to establish the design life for the kinetically or laser welded sleeved tubes in the presence of a crevice.
4. Perform post weld heat treatment at 1400° F minimum soak temperature with a 5-minute minimum soak time on freespan kinetically or laser welded joints until additional supporting data becomes available.

CECo will submit a proposed license amendment for Byron and Braidwood within 90 days from the issuance date of the pending sleeving amendment. This amendment will address items 1, 2 and 3 above. CECo does not intend to ask for an extension of this 90 day period. Additionally, CECo will ensure that the sleeve post weld heat treatment is performed as specified in item 4.

CECo appreciates the Staff's efforts in expediting the review and issuance of the Byron and Braidwood steam generator sleeving license amendment. If you have any questions or comments concerning this matter, please contact this office.

Sincerely,



Denise M. Saccomando
Nuclear Licensing Administrator

cc: R. Assa, Braidwood Project Manager - NRR
G. Dick, Byron Project Manager - NRR
S. DuPont, SRI - Braidwood
H. Peterson, SRI - Byron
B. Clayton, Branch Chief - Region III
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