

Northern States Power Company

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February 11, 1994

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10CFR 20.407 and Monticello Technical Specification Sections 6.7.A.2 & 6.7.A.6

US Nuclear Regulatory Commission Attn: Document Control Desk Washington, D.C. 20555

> MONTICELLO NUCLEAR GENERATING PLANT Docket No. 50-263 License No. DPR-22

Annual Report of Occupational Radiation Exposure and Safety Relief Valve Challenges - January 1 through December 31, 1993

Attached are three reports for 1993:

- Annual Report of Personnel Whole Body Exposure (Required by 10 CFR Part 20, Section 20.407(a)(1) and Section 20.407(b))
- Annual Report of Occupational Exposure (Required by Technical Specification 6.7.A.2)
- 3) Annual Report of Safety and Relief Valve Failures and Challenges (Required by Technical Specification 6.7.A.6 and NUREG 0737, Item II.K.3.3)

This letter contains no new NRC commitments, nor does it modify any prior commitments.

Please contact Marv Engen at (612) 295-1291 if you require additional information concerning these reports.

Roger O Anderson Director Licensing and Management Issues

cc: Director, Office of Nuclear Regulatory Research Regional Administrator-III, NRC NRR Project Manager, NRC Resident Inspector, NRC State of Minnesota, Attn: Kris Sanda

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Attachments:	Attachment	1 .	×	Personnel Whole Body Exposure Report				
	Attachment	L		Exposure				
	Attachment	3	÷.	Annual Report of Safety/Relief Valve Failures and Challenges				

ATTACHMENT 1

License: DPR-22

PERSONNEL WHOLE BODY EXPOSURE REPORT

FOR ALENDAR YEAR 1993

ANNUAL DOSE RANGES	NUMBER OF INDIVIDUALS
REM	IN EACH RANGE
No measurable exposure	446
<0.100	274
0.250 - 0.249	146
0.250 - 0.499	148
0.500 - 0.749	96
0.750 - 0.999	105
1.000 - 1.999	181
2.000 - 3.000	4
>3.000	0

TOTAL NUMBER OF INDIVIDUALS REPORTED 1402

0

TOTAL PERSON-REM 493.966

The information submitted is for the total number of individuals for whom personnel monitoring was required during the calendar year.

ATTACHMENT 2

MONTICELLO NUCLEAR GENERATING PLANT ANNUAL REPORT OF OCCUPATIONAL RADIATION EXPOSURE

1993

Number of Personnel and Man-Rem by Work and Job Function (1993)

	Number of	f Persons	> .100	To	tal Man-R	em
WORK & JOD FUNCTION	Station (.ontract	Station		
Rx Ops & Surv Operations Maintenance Supv & Engr Health Physics Instr & Control Security	45 40 23 8 0	0 93 3 0 0	0 3 10 18 0 0	30.052 9.378 21.642 10.573 4.868 0.0	0.0 28.287 1.165 0.0 0.0 0.0	0.0 2.421 3.887 4.416 0.0 1.320
Routine Maintenance Operators Maintenence Supv & Engr Health Physics Instr & Control Security	10 45 6 10 0	240 0 0 0	0 29 11 16 0 0	3.208 25.831 2.158 2.906 0.245 0.0	0.0 81.861 0.339 0.0 0.0 0.0	0.0 8.049 3.669 4.434 0.0 0.0
Special Maintenance Operators Maintenance Supv & Engr Health Physics Instr & Control Security	34 26 19 10 8 0	0 144 2 0 0	0 29 15 21 0	11.260 12.633 6.405 5.876 4.885 0.0	0.0 88.667 1.057 0.0 0.0 0.0	0.0 9.018 4.226 10.078 0.0 0.0
Waste Processing Operators Maintenance Supv & Engr Health Physics Instr & Control Security	0 0 1 0 0	0 8 0 0 0	0 0 8 2 0 0	0.188 0.218 0.241 0.073 0.005 0.0	0.0 2.785 0.010 0.0 0.0 0.0	0.0 0.059 2.529 0.605 0.0 0.0
Refueling Operators Maintenance Supv & Engr Health Physics Instr & Control Security	48 9 2 3 0	0 30 0 0 0	0 0 5 3 0 0	14.503 2.238 0.594 1.257 0.071 0.0	0.0 7.293 0.143 0.0 0.0 0.0	0.0 0.0 1.159 1.256 0.0 0.014
Inservice Inspection Operators Maintenance Supv & Engr Health Physics Instr & Control Security	0 1 1 2 0 0	0 4 5 0 0 0	0 20 32 3 0 0	0.341 0.254 0.320 0.444 0.026 0.0	0.0 15.791 0.122 0.0 0.0 0.0	0.0 13.233 23.045 1.017 0.0 0.0
Unassigned Exposure				-0.682	0.0	0.0

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Operators	137	0	0	59.552	0.0	0.0
Maintenance	121	560	81	50.552	224.684	32.780
Supv & Engr	69	5	81	31.360	2.836	38.515
Health Physics	48	0	63	21.129	0.0	21.806
Instr & Control	16	0	0	10.100	0.0	0.0
Security	0	0	0	0.0	0.0	1.334
Unassigned				-0.682	0.0	0.0
Grand Totals	391	565	225	172.011	227.520	94.435
Grand Torate	1. A. M.	222	ALC ALC LOT			

* Special Maintenance Items: 1) Plant Painting
2) Hard Pipe Drywell Vent Project
3) Refueling Outage

** INDIVIDUALS MAY BE LISTED UNDER MORE THAN ONE WORK AND JOB FUNCTION

ATTACHMENT 3

MONTICELLO NUCLEAR GENERATING PLANT

ANNUAL REPORT OF SAFETY/RELIEF VALVE FAILURES AND CHALLENGES

1993

This report is made in accordance with NUREG-0737, Item II.K.3.3 and Monticello Technical Specification 6.7.A.6 and covers the period from January 1 through December 31, 1993.

Failures

03/25/93 The topworks (pilot and second stage assembly) associated with safety/relief valve RV-2-71B was changed out due to excessive pilot/second stage leakage.

The topworks associated with safety/relief valve RV-2-71H was changed out due to a suspected small leak on the pilot stage bellows.

Challenges

03/23/93 Safety/relief valve RV-2-71H lifted automatically once at a reactor pressure of 1052 psig following a reactor scram during plant startup. The lift was initiated by the valve's low-low set logic. The valve lift setpoint under this logic scheme is 1052 psig. The self actuation setpoint of the valve is 1109 psig. This safety relief valve challenge was previously reported in Monticello Licensee Event Report 93-006.