



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555-0001

DETROIT EDISON COMPANY

DOCKET NO. 50-341

FERMI-2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 97
License No. NPF-43

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Detroit Edison Company (the licensee) dated May 24, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

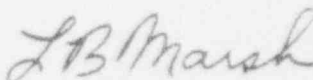
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-43 is hereby amended to read as follows:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 97, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. DECo shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance with full implementation within 45 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Ledyard B. Marsh, Director
Project Directorate III-1
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 22, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 97

FACILITY OPERATING LICENSE NO. NPF-43

DOCKET NO. 50-341

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

REMOVE

3/4 6-5

3/4 6-6

INSERT

3/4 6-5

3/4 6-6*

*Overleaf page provided to maintain document completeness. No changes contained on these pages.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- (c) Dewpoint temperature sensors shall have an accuracy of $\pm 1^\circ\text{F}$ or better over the dewpoint temperature range expected during the test $\pm 20^\circ\text{F}$ and a repeatability of at least $\pm 0.5^\circ\text{F}$.
 - (d) Pressure sensors should have a range such that P_a is between 25 and 75% of full scale. Accuracy shall be at least 0.015% of full scale with resolution and repeatability of 0.001% of full scale.
 - (e) The number and location of temperature and dewpoint sensors shall be determined prior to each Type A test based on a temperature survey of the containment.
 - (f) A sufficient number of dry bulb temperature sensors must be functioning properly during the test such that no sensor contributes more than 10% to the calculated temperature.
 - (g) At least two-thirds of the dewpoint temperature sensors shall be functioning properly during the test. However, if data recorded over the last 5 hours indicate that dewpoint temperatures have stabilized and any changes are not of an order to cause error in leak rate calculations, then malfunction of any or all but three of the dewpoint sensors shall not require aborting the test.
 - (h) At least one precision pressure gauge shall be functioning properly during the test.
 - (i) Prior to each Type A test and following the failure of any sensor, an instrument error analysis shall be performed using the Instrument Selection Guide (ISG) formula of ANSI/ANS-56.8-1981. The ISG shall not exceed $0.25 L_a$ at the end of a test except as noted in (g) above.
8. Three Type A Overall Integrated Containment Leakage Rate tests shall be conducted at 40 ± 10 month intervals during shutdown at P_a , 56.5 psig, during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.
9. If any periodic Type A test fails to meet $0.75 L_a$, the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet $0.75 L_a$, a type A test shall be performed at least every 18 months* until two consecutive Type A tests meet $0.75 L_a$, at which time the above test schedule may be resumed.

* An exemption from the 18-month accelerated test frequency requirement incurred after the failure of the two successive Type A tests conducted during the first and third refueling outages is allowed.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

10. The accuracy of each Type A test shall be verified by a supplemental test which:
 - (a) Confirms the accuracy of the test by verifying that the difference between the supplemental data and the Type A test data is within $0.25 L_a$.
 - (b) Has duration sufficient to establish accurately the change in leakage rate between the Type A test and the supplemental test.
 - (c) Requires that the rate of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 75% but not more than 125% of L_a at P_a , 56.5 psig.
- b. Type B and C tests shall be conducted with gas at P_a , 56.5 psig*, at intervals no greater than 24 months** except for tests involving:
 1. Air locks,
 2. Main steam line isolation valves,
 3. Penetrations using continuous leakage monitoring systems,
 4. Valves pressurized with fluid from a seal system,
 5. ECCS and RCIC containment isolation valves in hydrostatically tested lines which penetrate the primary containment, and
 6. Purge supply and exhaust isolation valves with resilient material seals.
- c. Air locks shall be tested and demonstrated OPERABLE per Specification 4.6.1.3.
- d. Main steam line isolation valves shall be leak tested at least once per 18 months.
- e. Type B tests for penetrations employing a continuous leakage monitoring system shall be conducted at P_a , 56.5 psig, at intervals no greater than once per 3 years.
- f. Leakage from isolation valves that are sealed with fluid from a seal system may be excluded, subject to the provisions of Appendix J, Section III.C.3, when determining the combined leakage rate provided the seal system and valves are pressurized to at least $1.10 P_a$, 62.2 psig, and the seal system capacity is adequate to maintain system pressure for at least 30 days.

*Unless a hydrostatic test is required per Table 3.6.3-1.

**With the exception of valves E11-F009, E11-F400 and E11-F608 for which the surveillance interval has been extended until startup from the first refueling outage late in 1989.