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### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

### PHILADELPHIA ELECTRIC COMPANY

### DOCKET NO. 50-352

### LIMERICK GENERATING STATION, UNIT 1

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 67 License No. NPF-39

The Nuclear Regulatory Commission (the Commission) has found that:

- A. The application for amendment by Philadelphia Electric Company (the licensee) dated November 30, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
- B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
- C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
- D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
- E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

9403020063 940216 PDR ADDCK 05000352 P PDR  Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-39 is hereby amended to read as follows:

# Technical Specifications and Environmental Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 67, are hereby incorporated into this license. Philadelphia Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Agules I. Miller

Charles L. Miller, Director Project Directorate I-2 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: February 16, 1994

# ATTACHMENT TO LICENSE AMENDMENT NO. 67

# FACILITY OPERATING LICENSE NO. NPF-39

## DOCKET NO. 50-352

Replace the following pages of the Appendix A Technical Specifications with the attached page. The revised page identified by Amendment number and is contains vertical lines indicating the areas of change. Overleaf page is provided to maintain document completeness.\*

Remove	Insert
3/4 6-3 3/4 6-4	3/4 6-3 3/4 6-4*

### CONTAINMENT SYSTEMS

# LIMITING CONDITION FOR OPERATION (Continued)

ACTION: (Continued)

- b. The combined leakage rate for all penetrations and all valves listed in Table 3.6.3-1, except for main steam line isolation valves\* and valves which are hydrostatically tested per Table 3.6.3-1, subject to Type B and C tests to less than or equal to 0.60 La, and
- c. The leakage rate to less than or equal to 11.5 scf per hour for any one main steam line through the isolation valves, and
- d. The combined leakage rate for all containment isolation valves in hydrostatically tested lines which penetrate the primary containment to less than or equal to 1 gpm times the total number of such valves,

prior to increasing the reactor coolant system temperature above 200°F.

#### SURVEILLANCE REQUIREMENTS

4.6.1.2 The primary containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR Part 50 using the methods and provisions of ANSI 45.4-1972 and BN-TOP-1 and verifying the result by the Mass Point Methodology described in ANSI N56.8-1981:

- a. Three Type A Overall Integrated Containment Leakage Rate tests shall be conducted at 40 +/- 10 month intervals during shutdown at  $P_a$ , 44.0 psig, during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.\*\*
- b. If any periodic Type A test fails to mee: 0.75 L<sub>a</sub>, the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet 0.75 L<sub>a</sub>, a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet 0.75 L<sub>a</sub>, at which time the above test schedule may be resumed.
- c. The accuracy of each Type A test shall be verified by a supplemental test which:
  - 1. Confirms the accuracy of the test by verifying that the difference between the supplemental data and the Type A test data is within 0.25La. The formula to be used is:  $[L_0 + L_{am} 0.25 L_a] \le L_c \le [L_0 + L_{am} + 0.25 L_a]$  where  $L_c =$  supplemental test result;  $L_0 =$  superimposed leakage;  $L_{am} =$  measured Type A leakage.
  - Has duration sufficient to establish accurately the change in leakage rate between the Type A test and the supplemental test.
  - Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be between 0.75 La and 1.25 La.

\* Exemption to Appendix "J" to 10 CFR Part 50.

Amendment No. 67

<sup>\*\*</sup> The interval between the second and third Overall Integrated Leakage Rate tests of the first 10-year service period will be extended to the sixth Unit 1 refueling outage. As a result, the duration of the first 10-year service period will be extended to the end of the sixth Unit 1 refueling outage.

#### CONTAINMENT SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

- d. Type B and C tests shall be conducted with gas at P<sub>a</sub>, 44.0 psig\*, at intervals no greater than 24 months except for tests involving:
  - 1. Air locks,
  - 2. Main steam line isolation valves,
  - Containment isolation valves in hydrostatically tested lines which penetrate the primary containment, and
- e. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.
- Main steam line isolation valves shall be leak tested at least once per 18 months.
- g. Containment isolation valves in hydrostatically tested lines which penetrate the primary containment shall be leak tested at least once per 18 months.
- h. The provisions of Specification 4.0.2 are not applicable to Specifications 4.6.1.2a., 4.6.1.2b., 4.6.1.2c., 4.6.1.2d., and 4.6.1.2e.

\*Unless a hydrostatic test is required per Table 3.6.3-1.