

UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON D.C. 20555-0001

ENCLOSURE

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION RELATED TO NRC BULLETIN 90-01, SUPPLEMENT 1, LOSS OF FILL-OIL

IN TRANSMITTERS MANUFACTURED BY ROSEMOUNT

GRAND GULF NUCLEAR STATION, UNIT 1

ENTERGY OPERATIONS, INC.

DOCKET NO. 50-416

1. INTRODUCTION

NRC Bulletin 90-01, Supplement 1 was issued by the NRC on December 22, 1992. to inform addressees of activities taken by the NRC staff and the industry in evaluating Rosemount Transmitters, and to request licensees to take actions to resolve this issue. The supplement requests utilities to review the information for applicability to their facilities, perform testing on the transmitter commensurate with its importance to safety and demonstrated failure rate, and modify, as appropriate, their actions and enhanced surveillance programs. The Supplement also requested that the licensee provide a response that included a statement as to whether or not the licensee will take the actions requested, and a list of the specific actions that the licensee would complete and the schedule for completing the actions. Additionally, when the specific actions committed to in the licensee's response were completed, the licensee was required to provide a statement confirming said completion. If the licensee did not plan to comply with the requested actions as delineated in the Supplement, a statement was required identifying those requested actions not taken, as well as an evaluation which provided the bases for those requested actions not taken.

2.0 DISCUSSION AND EVALUATION

The licensee for Grand Gulf Nuclear Station, Unit 1, responded to NRC Bulletin 90-01, Supplement 1 in submittals dated March 5, 1993, and December 21, 1993.

The requested actions delineated in Supplement 1, asked that licensees review plant records and identify any Rosemount Model 1153 Series B, Model 1153 Series D, and Model 1154 transmitters manufactured before July 11, 1989, that are used or may be used in the future in either safety-related systems or systems installed in accordance with 10 CFR 50.62 (the ATWS rule). Additionally, the licensee was to commit to a specified enhanced surveillance monitoring frequency that corresponded to the normal operating pressure of the

9403020012 940216 PDR ADOCK 05000416 PDR PDR transmitters identified. Furthermore, the licensee was requested to evaluate their enhanced surveillance monitoring program.

The submittal identified the applicable Rosemount Transmitter models and described the actions taken and an evaluation of the enhanced surveillance monitoring program. Furthermore, the licensee committed to tagging non-installed Model 1153B/D transmitters manufactured before July 11, 1989, to preclude inadvertent installation. Additionally, a commitment was made to replace five transmitters during the 1993 refueling outage. The replacement of the five transmitters has been completed. A detailed evaluation of the licensee's response is documented in the attached contractor's report.

3.0 CONCLUSION

We have reviewed the licensee's response to NRC Bulletin 90-01, Supplement 1, and conclude that the licensee conforms to the requested actions of NRC Bulletin 90-01, Supplement 1, and has completed the reporting requirements. Compliance and applicable Commission requirements may be the subject of NRC audits or inspections in the future.

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Date: February 16, 1994

Attachment: EG&G Technical Evaluation Report EGG-ERTP-10908