Attachment 3 to TXX-94046 Affected Technical Specification Pages (NUREG-1468) 1

Pages

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SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS

2.2 LIMITING SAFETY SYSTEM SETTINGS

REACTOR TRIP SYSTEM INSTRUMENTATION SETPOINTS

2.2.1 The Reactor Trip System Instrumentation and Interlock Setpoints shall be set consistent with the Trip Setpoint values shown in Table 2.2-1.

APPLICABILITY: As shown for each channel in Table 3.3-1.

ACTION:

- a. With a Reactor Trip System Instrumentation or Interlock Setpoint less conservative than the value shown in the Trip Setpoint column but more conservative than the value shown in the Allowable Value column of Table 2.2-1, adjust the setpoint consistent with the Trip Setpoint value.
- b. With the Reactor Trip System Instrumentation or Interlock Setpoint less conservative than the value shown in the Allowable Values column of Table 2.2-1, either:
 - Adjust the setpoint consistent with the Trip Setpoint value of Table 2.2-1 and determine within 12 hours that Equation 2.2-1 was satisfied for the affected channel, or

 Beclare the channel inoperable and apply the applicable ACTION statement requirement of Specification 3.3.1 until the channel is restored to OPERABLE status with its setpoint adjusted consistent with the Trip Setpoint value.

Equation 2.2-1

 $Z + R + S \leq TA$

Where:

- Z = The value from Column Z of Table 2.2-1 for the affected channel,
- R = The "as measured" value (in percent span) of rack error for the affected channel,
- S = Either the "as measured" value (in percent span) of the sensor error, or the value from Column S (Sensor Error) of Table 2.2-1 for the affected channel, and
- TA * The value from Column TA (Total Allowance) of Table 2.2-1 for the affected channel.

TABLE 2.2-1

REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

NCHE PEAK	FUNCTIONAL UNIT	TOTAL ALLOWANCE (TA)	Z	SENSOR ERROR (S)	TRIP SETPOINT	ALLOWABLE VALUE
1	1. Manual Reactor Trip	N.A.	N.A.	N.A.	N.A.	N.A
UNITS	2. Power Range, Neutron Flux					
-	a. High Setpoint	7.5	4.56	1.25	\leq 109% of RTP*	≤111.7% of RTP*
AND 2	b. Low Setpoint	8.3	4.56	1.25	≤25% of RTP*	≤27.7 of RTP*
	 Power Range, Neutron Flux, High Positive Rate 	1.6	0.5	0	≤5% of RTP* with a time constant ≥2 seconds	≤6.3% of RTP* with a time constant ≥2 seconds
2-5	 Power Range, Neutron Flux, High Negative Rate 	1.6	0.5	0	≤5% of RTP* with a time constant ≥2 seconds	<pre>≤6.3 of RTP* with a time constant ≥2 seconds</pre>
Unit	5. Intermediate Range, Neutron Flux	17.0	8.41	0	≤25% of RTP*	≤31.5 of RTP*
11	6. Source Range, Neutron Flux	17.0	10.01	0	≤10 ⁵ cps	≤1.4 x 10 ⁵ cps
Amendment Amendment	7. Overtemperature N-16 a. Unit 1	10.53	6.70	1.0+1.10+ 0.76 ⁽¹⁾	See Note D	See Note 2
ent	b. Unit 2	10.0	6.75	1.0+1.38+ 0.96 ⁽²⁾	See Note 1	See Note 2
No.				0.30		
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*RTP = RATED THERMAL POWER (1) 1.0% span for N-16 power monitor, 1.10% for T_{cold} RTDs and 0.76% for pressurizer pressure sensors. (2) 1.0% span for N-16 power monitor, 1.38% for T_{cold} RTDs and 0.96% for pressurizer pressure sensors.

TABLE 2.2-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

FUNCTIONAL UNIT	TOTAL ALLOWANCE (TA)	Z	SENSOR ERROR (S)	TRIP SETPOINT	ALLOWABLE VALUE
8. Overpower N-16	4.0	2.05	1.0+0.05(3)	≤112% of RTP*	≤114.5% of RTP*
9. Pressurizer Pressure-Low					
a. Unit l	4.4	0.71	2.0	≥1880 psig	>1863.6 psig
b. Unit 2	4.4	1.12	2.0	≥1880 psig	2:863.6 psig
10. Pressurizer Pressure-High				생활은 걸 것 같다.	21965,2 PS19)
a. Unit 1	7.5	5.01	1.0	<2385 psig	<2400.8 psig
b. Unit 2	7.5	1.12	2.0	≤2385 psig	<2401.4 psig
11. Pressurizer Water Level-High					
a. Unit I	8.0	2.18	2.0	<92% of instrument	<93.9% of instrume
1				span	span
b. Unit 2	8.0	2.35	2.0	592% of instrument	393.9% of instrume
M				span	span
12. Reactor Coolant Flow-Low			철도 다양 소양		
a. Jnit 1	2.5	1.18	0.6	≥90% of loop	≥88.6% of loop
	1.2.4			design flow**	design flow**
b. Unit 2	2.5	1.25	0.87	≥90% of loop	≥88.8% of loop
				minimum measured	minimum measured
				TIOW	flow***

** Loop design flow = 99,050 gpm
*** Loop minimum measured flow = 98,500 gpm

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		BLE 2.2-1		아이는 것 같은 동안을 받았네.	
	REACTOR TRIP SYS	STEM INSTR	UMENTATION	TRIP SETPOINTS	
FUNCTIONAL UNIT	TOTAL ALLOWANCE (TA)	Z	SENSOR ERROR (S)	TRIP SETPOINT	ALLOWABLE VALUE
 Steam Generator Water Level - Low-Low 					
a. Unit 1	25.0	22.08	2.0	≥25.0% of narrow range instrument	≥23.1% of narro range instrumer
b. Unit 2	35.4	22.2	2.0	span ≥35.4% of narrow range instrument span	span ≥33.4% of narro range instrumen span
14. Undervoltage - Reactor Coolant Pumps					-pan
a. Unit 1	7.7	1.2	0	≥4830 volts- each bus	≥4753 volts- each bus
b. Unit 2	7.7	1.2	0	≥4830 volts- each bus	≥4753 volts- each bus
 Underfrequency - Reacto Coolant Pumps 	r				
a. Unit 1 C	4.4	0	0	257.2 Hz	257.06 Hz
b. Unit 2 16. Turbine Trip	4.4	0	0	≥57.2 Hz	≥57.06 Hz
a. Low Trip System Pre	ssure N.A.	N.A.	N.A.	≥59 psig	≥46.6 psig
b. Turbine Stop Valve	N.A.	N.A.	N.A.		
Closure		110194	n.n.	≥l% open	≥l% open
17. Safety Injection Input from ESF	N.A.	N.A.	N.A.	Ν.Α.	N.A.

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COMANCHE			REAC	TAB		(Continued) MENTATION TR	1P SETPOINTS	
HE PEAK -	FUNC	TIONAL UNIT		TOTAL ALLOWANCE (TA)	Z	SENSOR ERROR (S)	TRIP SETPOINT	ALLOWABLE VALUE
UNITS	18.	Reactor Tr Interloc						
1 AND			ediate Range n Flux, P-6	N.A.	N.A.	N.A.	1 x 10 ⁻¹⁰ amps	≥6 x 10 ⁻¹¹ amps
N		b. Low Po Block,	wer Reactor Trips P-7					
		1) P-	10 input	N.A.	N.A.	N.A.	10% of RTP*	≤12.7% of RTP*
2-8		2) P-	13 input	Ν.Α.	N.A.	N.A.	10% RTP* Turbine First Stage Pres- sure Equivalent	≤12.7% RTP* Turbine First Stage Pressure Equivalent
		c. Power Flux,	Range Neutron P-8	N.A.	N.A.	Ν.Α.	48% of RTP*	≤50.7% of RTP*
		d. Power Flux,	Range Neutron P-9	N.A.	N.A.	N.A.	≤50% of RTP*	≤52.7% of RTP*
		e. Power Flux,	Range Neutron P-10	N.A.	N.A.	N.A.	10% of RTP*	≥7.3% of RTP*
	19.	Reactor Tr	ip Breakers	N.A.	N.A.	N.A.	N.A.	N.A.
	20.	Automatic Logic	Trip and Interlock	N.A.	N.A.	N.A.	Ν.Α.	N.A.

*RTP = RATED THERMAL POWER

TABLE 2.2-1 (Continued) TABLE NOTATIONS (Continued)

NOTE 1: (Continued)

For Unit 2

- (1) for $q_t q_b$ between -52% and +5.5%, $f_1(\Delta q) = 0$, where q_t and q_b are percent RATED THERMAL POWER in the top and bottom halves of the core respectively, and $q_t + q_b$ is total THERMAL POWER in percent of RATED THERMAL POWER,
- (ii) for each percent that the magnitude of $q_{\rm p} = q_{\rm p}$ exceeds -52%, the N-16 Trip Setpoint shall be automatically reduced by 2.15% of its value at RATED THERMAL POWER, and
- (iii) for each percent that the magnitude of q, q_b exceeds +5.5%, the N-16 Trip Setpoint shall be automatically reduced by 2.17% of its value at RATED THERMAL POWER.
- NOTE 2: The channel's maximum Trip Setpoint shall not exceed its computed Trip Setpoint by more than 3.51% of span for Unit 1 or 2.88% of span for Unit 2.

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COMANCHE PEAK - UNITS

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2.2 LIMITING SAFETY SYSTEM SETTINGS

BASES

2.2.1 REACTOR TRIP SYSTEM INSTRUMENTATION SETPOINTS

The Reactor Trip Setpoint Limits specified in Table 2.2-1 are the nominal values at which the Reactor trips are set for each functional unit. The Trip Setpoints have been selected to ensure that the core and Reactor Coolant System are prevented from exceeding their safety limits during normal operation and design basis anticipated operational occurrences and to assist the Engineered Safety Features Actuation System in mitigating the consequences of accidents. The setpoint for a Reactor Trip System or interlock function is considered to be adjusted consistent with the nominal value when the "as measured" setpoint is within the band allowed for calibration accuracy and instrument drift.

To accommodate the instrument drift assumed to occur between operational tests and the accuracy to which setpoints can be measured and calibrated. Allowable Values for the Reactor Trip Setpoints have been specified in Table 2.2-1. Operation with setpoints less conservative than the Trip Setpoint but within the Allowable Value is acceptable since an allowance has been made in the safety analysis to accommodate this error. [An optional provision has been included for determining the OPERABILITY of a channel when its Trip Setpoint is found to exceed the Allowable Value. The methodology of this option utilizes the "as measured" deviation from the specified calibration point for rack and sensor components in conjunction with a statistical combination of the other uncertainties of the instrumentation to measure the process variable and the uncertainties in calibrating the instrumentation. In Equation 2.2-1, $Z + R + S \leq TA$, the interactive effects of the errors in the rack and the sensor, and the "as measured" values of the errors are considered. Z, as specified in Table 2.2-1, in percent span, is the statistical summation of errors assumed in the analysis excluding those associated with the sensor and rack drift and the accuracy of their measurement. TA or Total Allowance is the difference, in percent span, between the Trip Setpoint and the value used in the analysis for Reactor trip. R or Rack Error is the "as measured" deviation. in percent span, for the affected channel from the specified Trip Setpoint. S or Sensor Error is either the "as measured" deviation of the sensor from its calibration point or the value specified in Table 2.2-1, in percent span, from the analysis assumptions. Use of Equation 2.2-1 allows for a sensor drift factor, an increased rack drift factor, and provides a threshold value for REPORTABLE EVENTS.

The mathadology to derive the Trip Setpoints is based upon combining all of the uncertainties in the channels. Inherent to the determination of the Trip Setpoints are the magnitudes of these channel uncertainties. Sensors and other instrumentation utilized in these channels are expected to be capable of operating within the allowances of these uncertainty magnitudes. Rack drift in excess of the Allowable Value exhibits the behavior that the rack has not met its allowance. Being that there is a small statistical chance that this will happen, an infrequent excessive drift is expected. Rack or sensor drift, in excess of the allowance that is more than occasional, may be indicative of more serious problems and should warrant further investigation.

COMANCHE PEAK - UNITS 1 AND 2 B 2-3

INSTRUMENTATION

3/4.3.2 ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.2 The Engineered Safety Features Actuation System (ESFAS) instrumentation channels and interlocks shown in Table 3.3-2 shall be OPERABLE with their Trip Setpoints set consistent with the values shown in the Trip Setpoint column of Table 3.3-3.

APPLICABILITY: As shown in Table 3.3-2.

ACTION:

- a. With an ESFAS Instrumentation or Interlock Trip Setpoint trip less conservative than the value shown in the Trip Setpoint column but more conservative than the value shown in the Allowable Value column of Table 3.3-3, adjust the Setpoint consistent with the Trip Setpoint value.
- b. With an ESFAS Instrumentation or Interlock Trip Setpoint less conservative than the value shown in the Allowable Value column of Table 3.3-3, either:
 - Adjust the Setpoint consistent with the Trip Setpoint value of Table 3.3-3, and determine within 12 hours that Equation 2.2-1 was satisfied for the affected channel, or
 - Beclare the channel inoperable and apply the applicable ACTION statement requirements of Table 3.3-2 until the channel is restored to OPERABLE status with its Setpoint adjusted consistent with the Trip Setpoint value.

Equation 2.2-1

 $Z + R + S \leq TA$

Where:

- Z = The value from Column Z of Table 3.3-3 for the affected channel,
- R = The "as measured" value (in percent span) of rack error for the affected channel,
- S = Either the "as measured" value (in percent span) of the sensor error, or the value from Column S (Sensor Error) of Table 3.3-3 for the affected channel, and
- TA = The value from Column TA (Total Allowance) of Table 3.3-3 for the affected channel.

c. With an ESFAS instrumentation channel or interlock inoperable, take the ACTION shown in Table 3.3-2.

COMANCHE PEAK - UNITS 1 AND 2 3/4 3-13

TABLE 3.3-3

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

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PEAK - FUNC	TIONAL UNIT	TOTAL ALLOWANCE (TA)	Z	SENSOR ERROR (S)	TRIP SETPOINT	ALLOWABLE VALUE
UNITS 1 AND 2 3/4	Safety Injection (ECCS, Reactor Trip, Feedwater Isolation, Control Room Emergency Recirculation, Emergency Diesel Generator Operation, Contain- ment Vent Isolation, Station Service Water, Phase A Isolation, Auxiliary Feedwater-Motor Driven Pump, Turbine Trip, Component Cooling Water, Essential Ventilation Systems, and Containment Spray Pump).			ł		
w 1	a. Manual Initiation	N.A.	N.A.	N.A.	N.A.	N.A
25	 Automatic Actuation Logic and Actuation Relays 	N.A.	N.A.	N.A.	N.A.	N.A
Unit	c. Containment PressureHigh 1	2.7	0.71	1.7	≤3.2 psig	≤3.8 psig
1	d. Pressurizer PressureLow I) Unit I 2) Unit 2	15.0 15.0	10.91 11.3	2.0	≥1820 psig ≥1820 psig	≥1803.6 psig ≥1803.6 psig
Amendmont	e. Steam Line PressureLow 1) Unit 1 2) Unit 2	17.3 17.3	15.01 9.15	2.0 2.0	≥605 psig* ≥605 psig*	≥593.5 psig* ≥578.4 psig*
N 2.	Containment Spray					
. 14	a. Manual Initiation	N.A.	N.A.	N.A.	N.A.	N.A
	b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.	N.A.	N.A.	N.A
	c. Containment PressureHigh-3	2.7	0.71	1.7	≤18.2 psig	≤18.8 psig

COMANC			TU FFATH	TABLE 3.3-3 (TON TOTO SETUCIN	TS
	CTIONA	<u>ENGINEERED SAFE</u>	TY FEATU	TOTAL ALLOWANCE (TA)	<u>Z</u>	SENSOR ERROR (S)	ION TRIP SETPOIN TRIP SETPOINT	ALLOWABLE VALUE
UNITS .	Cont	ainment Isolation	1					
1 AND 2	a.	Phase "A" Isolation 1) Manual Initiation 2) Automatic Actuation Lo	aic	N.A. N.A.	N.A. N.A.	N.A. N.A.	N.A. N.A.	N.A N.A
ω		 and Actuation Relays Safety Injection 	gic		ve for		Injection Trip	
3/4 3-26	b.	Phase "B" Isolation 1) Manual Initiation		See Item 2.a at	oove. P	'hase "B" i	isolation is man	ually initiated when
		 Automatic Actuation Lo and Actuation Relays 	gic	containment spi N.A.	N.A.	N.A.	nually initiate N.A.	d. N.A
		3) Containment Pressure High-3	7	2.7	0.71	1.7	≤18.2 psig	≤18.8 psig
		Containment Vent Isolatio	n				Contractor Name	. legletion is manually
		1) Manual Initiation	,	See Items 3.a. initiated when function is man	Phase '	'A" isolati	ion function or	t Isolation is manually containment spray
		 Automatic Actuation Lo and Actuation Relays 	igic.	N.A.	N.A.	N.A.	N.A.	N.A
		3) Safety Injection		Sco Item 1. ab Allowable Valu		all Safet	y Injection Trip	Setpoints and

COMANCHE		ENGINEERED SAFETY FE	TABLE 3.3-3 (ATURES ACTUATION SY			ATION TRIP SETPOI	<u>NTS</u>
PEAK		L UNIT m Line Isolation	TOTAL ALLOWANCE (TA)	Z	SENSOR ERROR (S)	TRIP SETPOINT	ALLOWABLE VALUE
	a.	Manual Initiation	N.A.	N.A.	N.A.	N.A.	N.A
AND 2	b.	Automatic Actuation Logic and Actuation Relays	N.A.	N.A.	N.A.	N.A.	N.A
	с.	Containment PressureHigh-2	2.7	0.71	1.7	≤6.2 psig	≤6.8 psig
3/4 3-2	d.	Steam Line PressureLow 1) Unit 1 2) Unit 2	17.3 17.3	15.01 9.15	2.0 2.0	≥605 psig* ≥605 psig*	≥593.5 psig* ≥578.4 psig*
27	e.	Steam Line Pressure - Negative RateHigh	8.0	0.5	0	<100 psi**	<178.7 psi**
	(1) Unit 1 2) Unit 2	8.0	0.5	0	<100 psi**	≤178.7 psi**

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COMANCHE		ENGINEERED SAFETY FE	TABLE 3.3-3 (ATURES ACTUATION S)			TION TRIP SETPOIN	<u>VTS</u>
	TIONA	AL UNIT	TOTAL ALLOWANCE (TA)	Z	SENSOR ERROR (S)	TRIP SETPOINT	ALLOWABLE VALUE
S. UNITS 1		oine Trip and Feedwater lation					
AND 2	a.	Automatic Actuation Logic and Actuation Relays	N.A.	N.A.	N.A.	N.A.	N.A
3/4	b.	Steam Generator Water LevelHigh-High 1) Unit 1	7.6	4.78	2.0	≤82.4% of narrow range instrument	≤84.3% of narrow range instrument span
3-28		2) Unit 2	18.5	12.4	2.0	span. ≤81.5% of narrow range instrument span.	<83.5% of narrow range instrument span

c. Safety Injection

See Item 1. above for all Safety Injection Trip Setpoints and Allowable Values.

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COMP		ENGINEERED SAFETY FE	TABLE 3.3-3 (ATURES ACTUATION S)	Contract of the second second second	and the second s	TION TRIP SETPOI	NTS
COMANCHE PEAK	CTION	AL UNIT	TOTAL ALLOWANCE (TA)	Z	SENSOR ERROR (S)	TRIP SETPOINT	ALLOWABLE VALUE
1 6.	Aux	iliary Feedwater					
UNITS	a.	Automatic Actuation Logic and Actuation Relays	N.A.	N.A.	N.A.	N.A.	N.A
	b.	Steam Generator Water LevelLow-Low					
AND 2		1) Unit 1	25.0	22.08	2.0	≥25.0% of narrow range instrument	≥23.1% of narrow range instrument span.
3/4 3.		2) Unit 2	35.4	22.2	2.0	span. ≥35.4% of narrow range instrument span.	≥33.4% of narrow range instrument span.
-29	с.	Safety Injection - Start Motor Driven Pumps	See Item 1. abo Allowable Value		all Safet	y Injection Trip	Setpoints and
	d.	Loss-of-Offsite Power	(N.A.	N.A.	N.A.	N.A.	N.A
Un	e.	Trip of All Main Feedwater Pumps	N.A.	N.A.	N.A.	N.A.	N.A
Unit 1		omatic Initiation of ECCS tchover to Containment Sump					
- Ame	a.	Automatic Actuation Logic and Actuation Relays	N.A.	N.A.	N.A.	N.A.	N.A
Amendment	b.	RWST LevelLow-Low 1) Unit 1	2.5	0.71	1.25	≥40.0% of	≥38.9% of span
nt No.		2) Unit 2	2.5	0.99	1.25	span ≥40.0% of span	≥39.1% of span
no •		Coincident With Safety Injection	See Item 1. abo Allowable Value		all Safet	y Injection Trip	Setpoints and
4							

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

S		TABLE 3.3-3 (Continu	ied)		
COMANCHE	ENGINEERED SAFETY FEA	TURES ACTUATION SY	STEM II	NSTRUMENTA	TION TRIP SETPOI	NTS
		TOTAL		SENSOR		
PEAK FUNC	CTIONAL UNIT	ALLOWANCE (TA)	Z	<u>(S)</u>	TRIP SETPOINT	ALLOWABLE VALUE
- UNITS	Loss of Power (6.9 kV & 480 V Safeguards System Undervoltage)					
good .	 a. 6.9 kV Preferred Offsite Source Undervoltage 	N.A.	N.A.	N.A.	≥5004 ¥	≤5900 V ≥4900 V
AND 2	b. 6.9 kV Alternate Offsite Source Undervoltage	N.A.	N.A.	N.A.	≥5004 ¥	≤5900 V ≥4900 V
	c. 6.9 kV Bus Undervoltage	N.A.	N.A.	N.A.	≥2037 V	≥1935 V ≤3450 V
ω	d. 6.9 kV Degraded Voltage	N.A.	N.A.	N.A.	≥6054 V	≥5933 V
1/4	e. 480 V Degraded Voltage	N.A.	N.A.	N.A.	≥439 V	≥435 V
3-30	f. 480 V Low Grid Undervoltage	N.A.	N.A.	N.A.	≥447 V	≥443 V
9.	Control Room Emergency Recirculation					
Unit	a. Manual Initiation	N.A.	N.A.	N.A.	N.A.	N.A
it 1 -	b. Safety Injection	See Item 1. ab Allowable Valu		all Safet	y Injection Trip	Setpoints and
Amendment	Engineered Safety Features Actuation System Interlocks					
men	a. Pressurizer Pressure, P-11	1			1000	1075 2 pain
It No.	1) Unit 1 2) Unit 2	N.A. N.A.	N.A. N.A.	N.A. N.A.	≤1960 psig ≤1960 psig	≤1975.2 psig ≤1976.4 psig
14	b. Reactor Trip, P-4	N.A.	N.A.	N.A.	N.A.	N.A
11.	Solid State Safeguards Sequencer (SSSS)	N.A.	N.A.	N.A.	N.A.	N, A

3/4.3 INSTRUMENTATION

BASES

3/4.3.1 and 3/4.3.2 REACTOR TRIP SYSTEM and ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

The OPERABILITY of the Reactor Trip System and the Engineered Safety Features Actuation System instrumentation interlocks ensures that: (1) the associated ACTION and/or Reactor trip will itiated when the parameter monitored by each channel or combination thereof reaches its Setpoint, (2) the specified coincidence logic and sufficient redundancy is maintained to permit a channel to be out-of-service for testing or maintenance consistent with maintaining an appropriate level of reliability of the reactor protection and engineered safety features instrumentation, and (3) sufficient system functional capability is available from diverse parameters.

The OPERABILITY of these systems is required to e the overall reliability, redundancy, and diversity assumed available in the facility design for the protection and mitigation of accident and transient conditions. The integrated operation of each of these systems is consistent with the assumptions used in the safety analyses. The Surveillance Requirements specified for these systems ensure that the overall system functional capability is maintained comparable to the original design standards. The periodic surveillance tests performed at the minimum frequencies are sufficient to demonstrate this capability. Specified surveillance intervals and surveillance and maintenarro outage times have been determined in accordance with WCAP-10271, "Evaluati, Surveillance Frequencies and Out of Service Times for the Reactor Protection __trumentation System", WCAP-10271 Supplement 2, "Evaluation of Surveillance Frequencies and Out of Service Times for the Engineered Safety Features Actuation System" and supplements to these reports as approved by the NRC and documented in the SER (letters to the Westinghouse Owners Group (WOG) dated February 21, 1985, February 22, 1989, and April 30, 1990).

The Engineered Safety Features Actuation System Instrumentation Trip Setpoints specified in Table 3.3-3 are the nominal values at which the bistables are set for each functional unit. A Setpoint is considered to be adjusted consistent with the nominal value when the "as measured" Setpoint is within the band "lowed for calibration accuracy.

to accommodate the instrument drift assumed to occur between operational tests and the accuracy to which Setpoints can be measured and calibrated, Allowable Values for the Setpoints have been specified in Table 3.3-3. Operation with Setpoints less conservative than the Trip Setpoint but within the Allowable Value is acceptable since an allowance has been made in the safety analysis to accommodate this error. An optional provision has been included for determining the OPERABILITY of a channel when its Trip Setpoint is found to exceed the Allowable Value. The methodology of this option utilizes the "as measured" deviation from the specified calibration point for rack and sensor components in conjunction with a statistical combination of the other uncertainties of the instrumentation to measure the process variable and the uncertainties in calibrating the instrumentation. In Equation 2.2-1, $Z + R + S \leq TA$, the interactive effects of the errors in the rack and the sensor, and the "as measured" values of the errors are considered. Z, as specified in Table 3.3-3, in percent span, is the statistical summation of errors assumed in the analysis excluding those associated with the sensor and rack drift and the accuracy of their measurement. TA or Total

COMANCHE PEAK - UNITS 1 AND 2

B 3/4 3-1 Unit 1 - Amendment No. 13

INSTRUMENTATION

BASES

REACTOR TRIP SYSTEM and ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION (Continued)

Allowance is the difference, in percent span, R or Rack Error is the "as" measured" deviation, in the percent span, for the affected channel from the specified Trip Setpoint. S or Sensor Error is either the "as measured" deviation of the sensor from its calibration point or the value specified in Table 3.3-3, in percent span, from the analysis assumptions. Use of Equation 2.2-1 allows for a sensor drift factor, an increased rack drift factor, and provides a threshold value for REPORTABLE EVENTS.

The methodology to derive the Trip Setpoints is based upon combining all of the uncertainties in the channels. Inherent to the determination of the Trip Setpoints are the magnitudes of these channel uncertainties. Sensor and rack instrumentation utilized in these channels are expected to be capable of operating within the allowances of these uncertainty magnitudes. Rack drift in excess of the Allowable Value exhibits the behavior that the rack has not met its allowance. Being that there is a small statistical chance that this will happen, an infrequent excessive drift is expected. Rack or sensor drift, in excess of the allowance that is more than occasional, may be indicative of more serious problems and should warrant further investigation.

The measurement of response time specified in the Technical Requirements Manual at the specified frequencies provides assurance that the Reactor trip and the Engineered Safety Features actuation associated with each channel is completed within the time limit assumed in the safety analyses. No credit was taken in the analyses for those channels with response times indicated as not applicable. Response time may be demonstrated by any series of sequential, overlapping, or total channel test measurements provided that such tests demonstrate the total channel response time as defined. Sensor response time verification may be demonstrated by either: (1) in place, onsite, or offsite test measurements, or (2) utilizing replacement sensors with certified response time.

The Engineered Safety Features Actuation System senses selected plant parameters and determines whether or not predetermined limits are being exceeded. If they are, the signals are combined into logic matrices sensitive to combinations indicative of various accidents events, and transients. Once the required logic combination is completed, the system sends actuation signals to those Engineered Safety Features components whose aggregate function best serves the requirements of the condition. As an example, the following actions may be initiated by the Engineered Safety Features Actuation System to mitigate the consequences of a steam line break or loss-of-coolant accident: (1) ECCS pumps start and automatic valves position, (2) Reactor trip, (3) feedwater isolation, (4) startup of the emergency diesel generators, (5) containment spray pumps start and automatic valves position (6) containment isolation, (7) steam line isolation, (8) turbine trip, (9) auxiliary feedwater pumps start and automatic valves position, (10) station service water pumps start and automatic valves position, (11) Control Room Emergency Recirculation starts, and (12) essential ventilation systems (safety chilled water, electrical area fans, primary plant ventilation ESF exhaust fans, battery room exhaust fans, and UPS ventilation) start.

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