



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 14, 1994

Docket No. 50-333

LICENSEE: Power Authority of the State of New York
FACILITY: James A. FitzPatrick Nuclear Power Plant
SUBJECT: SUMMARY OF JANUARY 26, 1994, MEETING TO DISCUSS THE STATUS OF
LICENSING ACTIONS AND ACTIVITIES ASSOCIATED WITH THE JAMES A.
FITZPATRICK PLANT

A meeting was held in the NRC One White Flint North Office in Rockville, Maryland, with the Power Authority of the State of New York (PASNY) and NRC staff representatives to discuss the status of licensing actions and activities associated with the James A. FitzPatrick Nuclear Power Plant. The licensee had requested this meeting. Enclosure 1 is a list of meeting attendees. Enclosure 2 is a copy of the handout material provided by PASNY.

The attendees initially discussed the current plant status and scheduled upcoming unit outages. A 27-day maintenance/surveillance outage is planned for April 2, 1994. Work activities planned for this outage include replacement of 27 control rod drives, electrical bus maintenance, and extensive local leak rate testing. A refueling outage is scheduled to begin on November 29, 1994. Work activities planned for this outage include a shroud inspection and significant maintenance and surveillance.

PASNY then provided an outline of plant accomplishments and improvements made in 1993. Accomplishments and improvements made at FitzPatrick during 1993, as presented by Mr. Michael Colomb, General Manager of Support Services, included: (1) implementation of "Reliable On-Line Maintenance Environment (ROME)," which is designed to improve work planning; (2) implementation of a 13-week rolling schedule of system windows for work and surveillance which is also designed to improve work planning; (3) implementation of training program review and steering committees which enhance management oversight of training; (4) initiation of a comprehensive surveillance test adequacy review; and, (5) spent fuel pool cleanup.

The meeting attendees then discussed the current and anticipated licensing issues for the FitzPatrick plant. PASNY identified the highest priority licensing actions as being: (1) an exemption from 10 CFR Part 50, Appendix J, pertaining to containment leak rate testing of the shutdown cooling isolation valves; (2) a Technical Specification (TS) amendment pertaining to the main steam line radiation monitor's scram and isolation functions; and (3) a TS amendment pertaining to the east and west cable tunnel suppression systems. The NRC presented the status of the staff reviews associated with each

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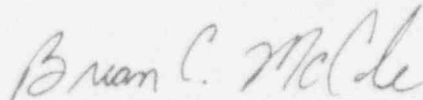
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February 14, 1994

licensing action and activity. Future licensing actions planned by PASNY include those associated with a 24-month operating cycle and those associated with removing fire protection requirements from the TS.

Mr. William Cook, the NRC Senior Resident Inspector at FitzPatrick, outlined upcoming inspection activities planned for FitzPatrick. The meeting concluded with an agreement among the attendees to conduct the next quarterly licensing meeting at the FitzPatrick site late in the month of March 1994.



Brian C. McCabe, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. List of Attendees
2. Licensee Handout Material

cc w/enclosures:
See next page

Power Authority of the State of New York

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Power Plant

cc:

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JANUARY 26, 1994

STATUS OF LICENSING ACTIONS AND ACTIVITIES
ASSOCIATED WITH THE JAMES A. FITZPATRICK PLANT

LIST OF ATTENDEES

<u>NAME</u>	<u>AFFILIATION</u>
R. Capra	NRC
B. McCabe	NRC
N. Conicella	NRC
W. Cook	NRC
J. Gray	PASNY
M. Colomb	PASNY
M. Breck	NUS

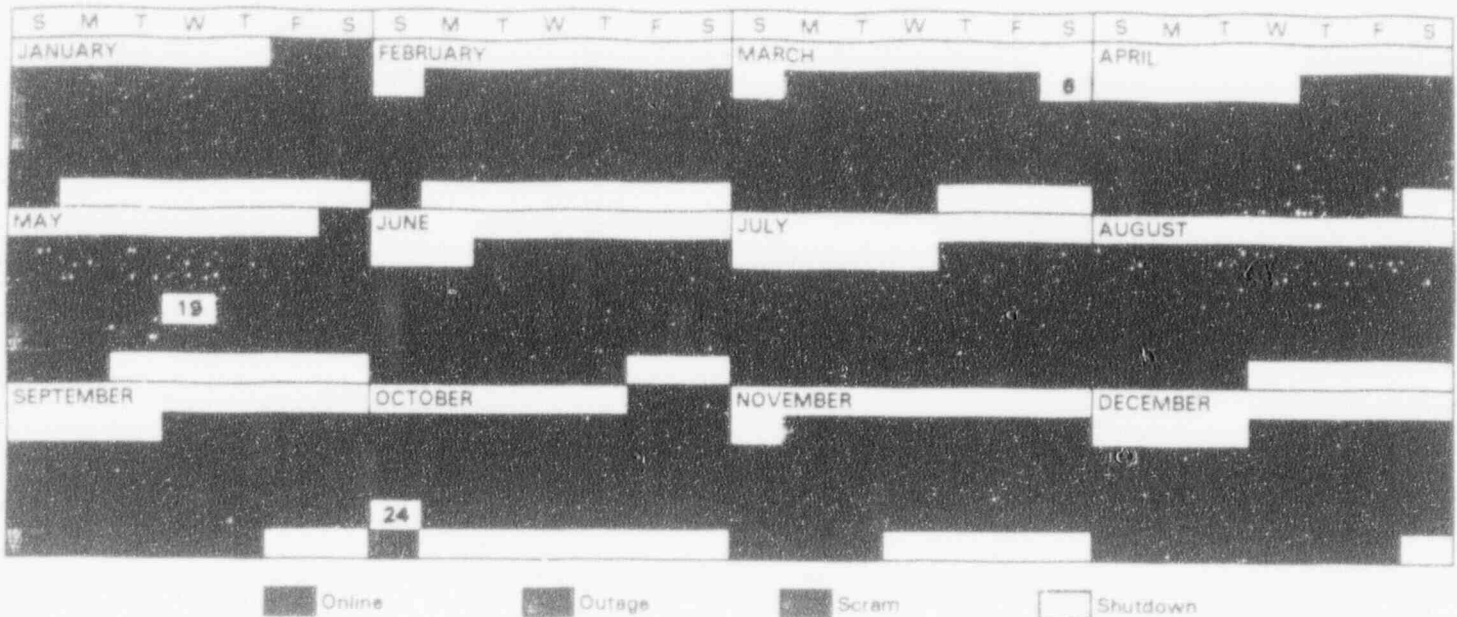
PLANT STATUS

- **100 % Power**

- **Planned Maintenance/Surveillance Outage**
 - **April 2**
 - **27 days (current plan)**

- **1994/1995 Refuel Outage**
 - **November 29 (will include shroud inspection)**

James A. FitzPatrick Nuclear Power Plant 1993 Performance



Key Events During 1993

- 1/23/93 - Unit Returns to service following 14 month shutdown
- 2/25/93 - Manual scram - Loss of intake due to ice
- 3/8/93 - Controlled shutdown - Discovery of leak in drywell during plant startup at 1000 psi
- 4/20/93 - Automatic scram - A Feed Pump motor gear unit failure coupled with failure of A Feed Pump discharge check valve
- 5/19/93 - Controlled shutdown - To effect repairs to HPCI outside containment check valve body to bonnet leak
- 5/25/93 - Automatic scram - At 5% power the T APRM produces a spurious trip with D Main Steam Line rad monitor out of service for repairs
- 9/24/93 - Automatic scram - At 17% power with Generator removed from service, troubleshooting of DC ground results in Turbine bypass valve closure
- 10/24/93 - Controlled Shutdown - Planned entry into 1993 Fall Maintenance Outage

Key Performance Indicators - Year End 1993

	Goal	Result		Goal	Result
Annual Capacity Factor	86.5	69.5	Personnel Exposure (Man-Rem)	220	231
Cycle Capacity Factor	—	73.9	Personnel Contamination Events	25	11
Forced Outage Rate	—	13.3	RCA Contamination Free (%)	93	88
Jumpers	52	63	Cum Hydrogen Availability (%)	90	81
Control Room Deficiencies	20	31	Liquid Radwaste Discharged (gal)	312000	312
Control Room Alarms at 100% power	0	0	Chemistry Performance Index	40	41
Unplanned Scrams at Power	0	4	Fuel Performance Index (ucv sec)	300	22
Unplanned Safety System Actuations	0	1	Corrective Action PIDs - Nonoutage	1000	10
Thermal Performance	98.4	99.9	Corrective Action PIDs - Outage	—	0
Heat Rate (Btu/Kwhr)	10,100	9972	Lost Time Incident Rate	0	0
			Site Reportable Incident Rate	2.5	2.5

1993 IMPROVEMENTS/ACCOMPLISHMENTS

- **ROME**

Implemented "Reliable On-Line Maintenance Environment." A Central Planning group was established. Improvements in work planning.

- **13 Week Schedule**

System windows for work and surveillance are rotated through a 13 week rolling schedule. Improvements in work planning.

- **Minor Maintenance**

Revised the definition of "minor maintenance" in the Administrative Procedure. Allowed specific work in balance of plant system to be done as minor maintenance. Assembled a multi-discipline crew to address minor maintenance issues.

- **Implementation of Training Program Review and Steering Committees**

Each accredited program has a review committee which includes the Training Manager, Department Manager, a Trainer, and a job incumbent. The Steering Committee provides oversight. Improved line management ownership in Training

- **Outage Management**

Shift Managers assigned for each outage. (24 hour/7 day coverage). Scope control prior to and during outages. Improved adherence to outage schedule and plan.

- **Surveillance Test Adequacy Review**

The initial scope of LSFT review is complete. Fire Protection review is complete. The standard for periodic reviews was improved. The review will continue of the current level of effort.

- **Spent Fuel Pool Cleanup**

All irradiated hardware (CRBs, LPRMs, IRMs, SRMs, Jet Pump Beam hold down bolts, shroud head bolts, and vacuum filters) has been processed and packaged. Three TN-RAM cask shipments, an SEG 10-142B cask shipment and a shipment of shroud head bolts (18 of the 36) have been completed. The third TN-RAM cask shipment (12/10/93) arrived at Barnwell, SC contaminated above DOT transportation limits; however, the liner was unloaded and buried on 12/18/93. We are still awaiting a decision from NRC Region I on the outcome of the incident. Two TN-RAM cask shipments, one SEG 10-142B cask shipment (vacuum filters), and the remainder of the shroud head bolts (18) are scheduled to be loaded and shipped from JAF starting the week of February 21, 1994.

- **Fuel Leak Action**

Following indication of the existence of a very small leaker in the Cycle 11 core, early in the cycle, we have effectively controlled the fuel to prevent degradation of the clad. Initially this was accomplished by limitations on movement of all control rods and the rate of core power change.

In July, 1993, a multi-disciplinary team comprised of Site Chemistry, WPO Reactor Engineering and Site Reactor Engineering personnel were successful in identifying the control cell containing the leaking fuel bundle.

Following this, controls were implemented to suppress nodal powers in this cell.

Most recently, restrictions on all control rod manipulations and overall core power change rate were lifted and replaced by controls which control the level and rate of change of power in the cell containing the leaking fuel pin.

- **Emergency Planning**

Drills are performed from the simulator. Simulator data is provided to the ERO facilities via the computer network. The OSC procedure and facility was upgraded to enhance response.

- **Deviation/Event Report Program (DER)**

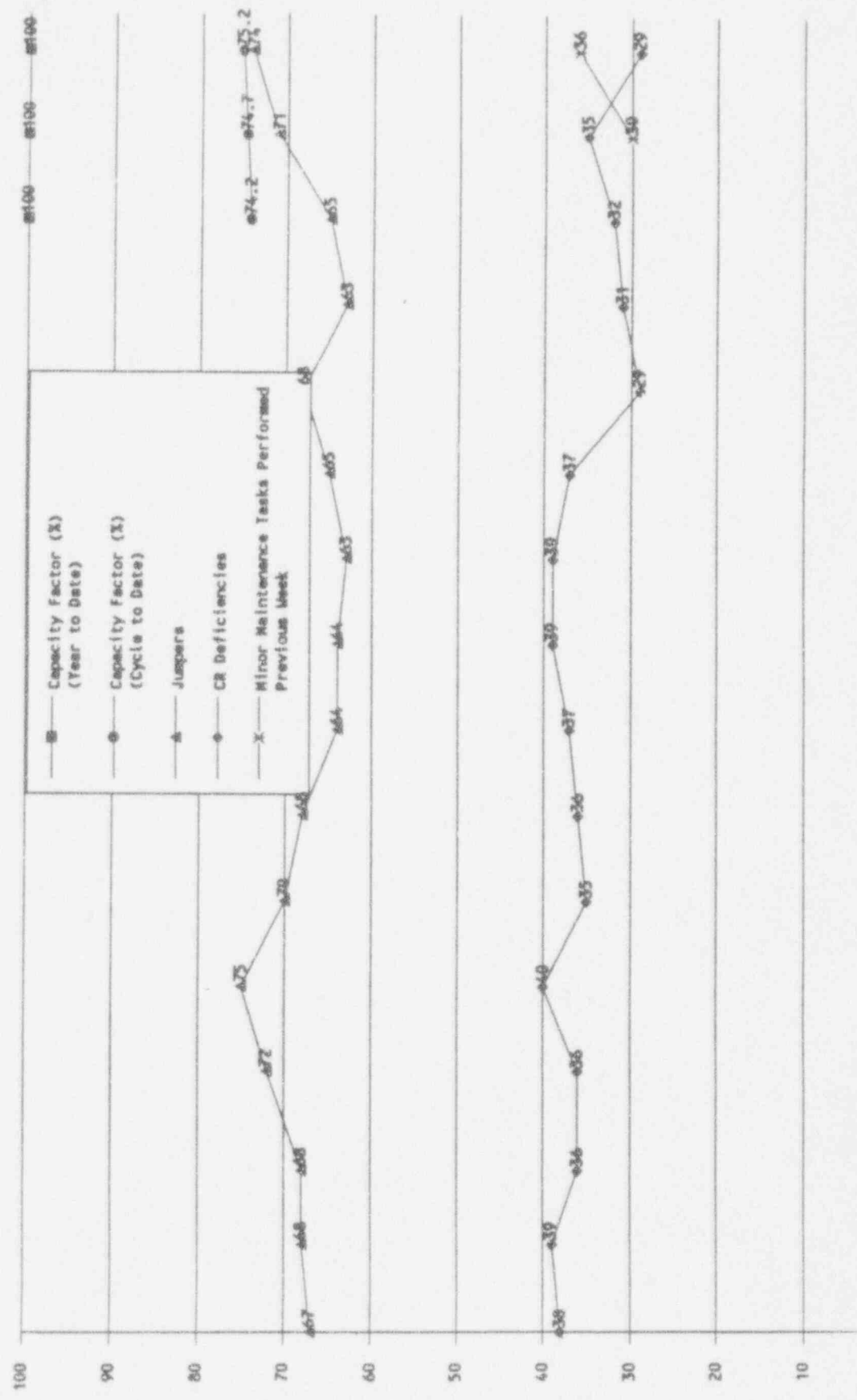
This program was implemented early in 1994. The Operational Review Group Trends DER causal factors and programs quarterly. JAF requested and received an INPO assist visit in the area of Root Cause Evaluations.

- **Top 10 Technical Issues**

A list of high priority technical issues has been established. The Technical Services Manager is the focal point. Each issue has an owner (formatted similar to startup Issues).

JAMES A. FITSPATRICK NUCLEAR POWER PLANT

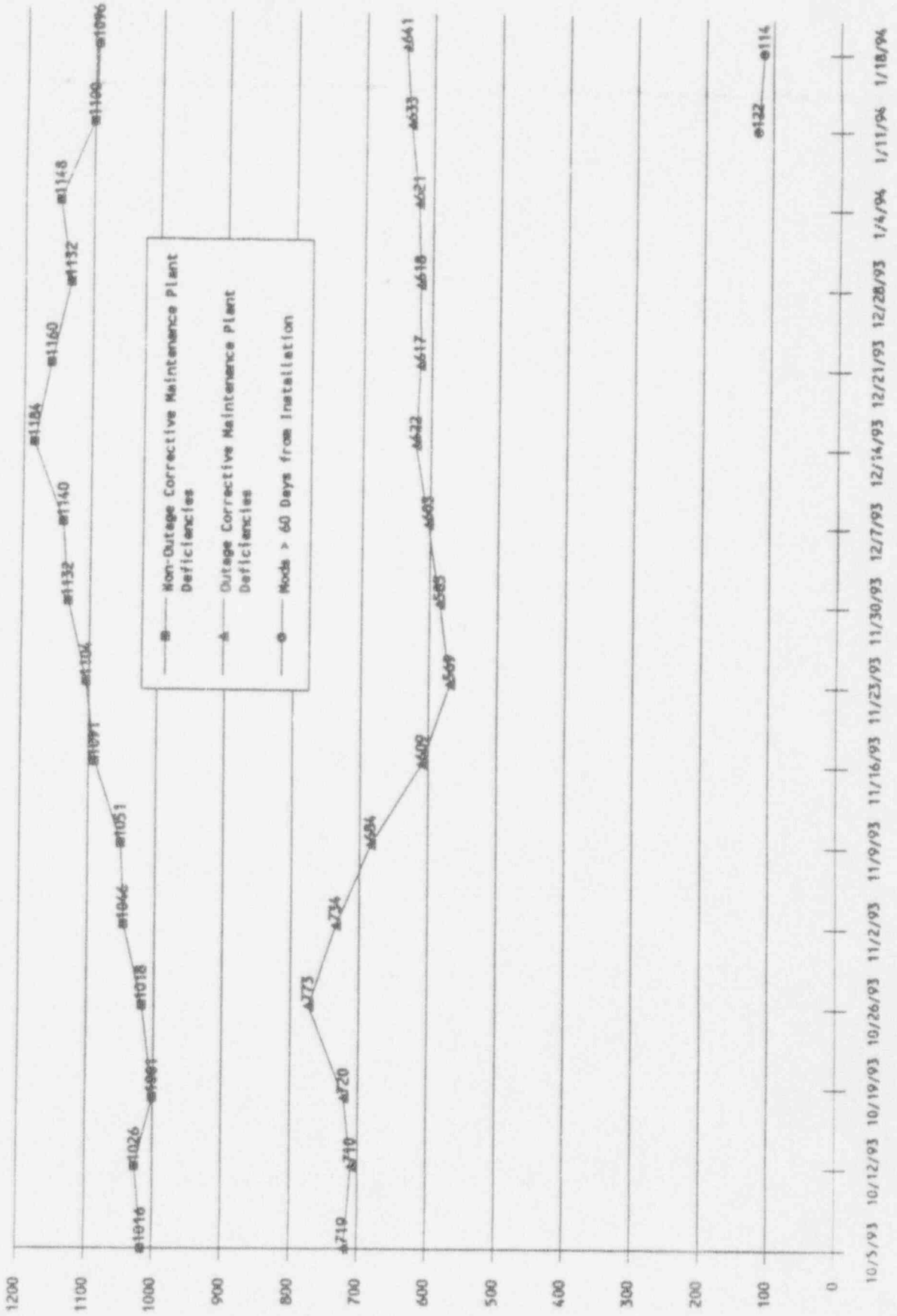
PLANT TRENDS 1



10/5/93 10/12/93 10/19/93 10/26/93 11/2/93 11/9/93 11/16/93 11/23/93 11/30/93 12/7/93 12/14/93 12/21/93 12/28/93 1/4/94 1/11/94

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

PLANT TRENDS 2



JAMES A. FITZPATRICK NUCLEAR POWER PLANT
TOP TECHNICAL ISSUES

January 7, 1994
JTS-94-0014

Attached is the latest copy of the JAF Top Technical Issues for your information.

Revisions to the Top Technical Issues should be made by the first of every month. The updated report will be presented at the monthly Engineering Meetings (2nd Tuesday of every month) and the monthly Project Meetings (3rd Wednesday of every month).

Should you have any questions regarding the overall report (requirements for adding an issue, closing an issue, etc.), please contact Floyd Edler at extension 6501. Any revisions that need to be made to this report should be done through Lisa Snyder (ext. 6698, fax. 6502).

Distribution:

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D. Lindsey
D. Ruddy
D. Kieper
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T. Dougherty, WPO
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J. Gray, WPO
K. Mavrikis, WPO
J. Kaucher, WPO
J. Kelly, WPO
R. Penny, WPO
L. Katz, WPO
RMS, JAF
RMS, WPO

Tech. Issue Lead Individuals:

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V. K. Kapur, WPO
J. Goldstein, WPO
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J. Lafferty, WPO
S. Kohr, WPO
H. Hartjen, WPO
J. Ellmers, WPO
G. Stranovsky, WPO
M. Tallents
A. DiRocco, WPO

1994 SPRING MAINTENANCE OUTAGE JAF TEAM GOALS

DURING THE OUTAGE

- Complete Outage within planned duration of **(TBD)** Days J. Flaherty
- Completion of originally planned outage activities > 90%
- Completion of approved emergent work > 90%
- Overall schedule adherence \geq 75%
- Limit outage scope growth to < 17%
- Control overtime expenses to < 50% of regular hours
- LER's from personnel errors - 0
- NRC violations - 0
- Radiation exposure < 91 manRem
- Personnel contamination events of > 1000 dpm \leq 20 events
- Lost time accidents - 0
- OSHA recordable injuries < 2
- Fires - 0
- DERs due to procedural adherence - 0

UPON CONCLUSION OF THE OUTAGE OUR SUCCESS WILL BE MEASURED BY

- Controlled, safe startup to 100% with no scrams or forced shutdowns
- Drywell unidentified leak rate < 0.27 GPM
- Zero locked in Control Room alarms at 100% Power
- Outage-required Control Room devices out of service - 0
(with exception of those requiring a refuel outage)
- Less than **(TBD)** jumpers R. Locy
- Installed catch containments **(TBD)** J. Sipp/J. DeRoy
- Plant contamination area equal to or less than pre-outage levels within 2 weeks of achieving 100% power
- All systems operational (with exception of Relay Room CO₂)
- No forced shutdowns due to equipment failures prior to next Refuel Outage

1994 MAINTENANCE OUTAGE WORK SCOPE SUMMARY

AS OF 01/18/93

- Corrective maintenance work items - 307
- Preventive maintenance work items - 251
- Valves to be local leak rate tested - 77
- I&C IMPs (calibrations) - 396
- Operations surveillance tests - 154
- I&C surveillance tests - 73
- Bus outages
 - 10500
 - 10600
 - 10300
 - 10400
 - A & B RPS
 - B D/C
- 27 CRDs to be exchanged
- SQUG walkdown
- 10 modifications (F1 & M1)
- T-1A inspection
- Main turbine bearing measurements
- Torus desludge and inspection

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 1

STATUS: OPEN

TITLE: CONTROL & RELAY ROOM VENTILATION

LEAD INDIVIDUAL: LAZARUS

DEPARTMENT: JAF-TS

PROBLEM STATEMENT:

Recently, there has been several questions related to plant Ventilation Systems. In fact, two major issues during the forced outage were related to Ventilation Systems.

The Control Room has been the focus for most of the issues. Currently, the Control Room Ventilation is in the "isolate" or emergency operational mode because the normal lineup cannot comply with design requirements of single mode failure. While following up on material and design issues related to the Control Room Ventilation System, several other issues have been identified. These issues include:

Valves that do not perform their intended functions

System design is not as described in the FSAR or our NUREG-0737 response.

Ventilation trains have higher than design allowable flows

Pressure distribution was not as specified

Since the Control Room is only one of ten systems, an overall ventilation plan has been written.

Cable separation deficiencies and single failure deficiencies for originally installed equipment.

IMPACT ON PLANT OPERATION: (REGULATORY, OPERATING RESTRICTIONS, DESIGN ETC.)

The system must operate in the emergency or "isolate" mode while the Reactor is greater than 212°F, until the "pre-restoration" items listed in the Control Room Vent Consolidated Plan are complete.

REFERENCE DOCUMENTS:

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 1

STATUS: OPEN

TITLE: CONTROL & RELAY ROOM VENTILATION

LEAD INDIVIDUAL: LAZARUS

DEPARTMENT: JAF-TS

INTERDEPARTMENTAL INTERFACES:

DEPARTMENT	INDIVIDUAL	PHONE
TS	Lazarus	6270
TS	Costedio	6358
NED	Young	6416
Licensing	Ellmers	6280

ACTION PLAN:

As discussed in the Control Room Vent Consolidated Action Plan. Monthly updates to the action plan will be issued by 1/14/94.

WORK ACCOMPLISHED THIS PERIOD:

Baseline data collection, inspections, documentation review, and drawing development are almost complete. The Johnson-Yokogawa procedure for

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DEPARTMENT: JAF-TS

ventilation system balancing, which will be used to perform CR and RR balancing activities is under review and will be submitted for PORC approval by 1/31/94. TS & NED are currently working on a contract extension to continue paying for J-Y support.

Each readjustment of the CREVASS exhaust fans was successful in improving an vibration levels establishing a positive duct pressure upstream of the isolation MOV's, and in clearing the low flow alarms that were in. These results will enable us to: 1) take the FN-4A control switch out of "pull-to-lock" and 2) provide a potential basis for justifying the existing mod's as isolation devices due to outward leakage.

Adjusted and repaired various dampers.

Sealed leaks in emergency fans and working on filter housing door leaks.

DC mod complete and material being supplied to restore isolation mod's. Working on PWT plan to pressure test valves for leakage.

Working with Maintenance Planning on preparing work packages for numerous WR's.

Working on developing an overall PM scope and schedule for CREVASS instrumentation.

Replaced "B" train charcoal filters.

Walkdown and inspections of the Control Room and Relay Room ventilation hardware and equipment was completed by the Johnson-Yokogawa technicians and NYPA personnel. All identified deficiencies were addressed by issuing PIDs and the J-Y technicians and system engineer are supporting the planners in development of work packages. The Technical Services and Planning Departments compiled a listing of all outstanding PIDs and work requests against the system, and reviewed and prioritized each item to determine which work items were required prior to restoring the system to normal. Central Planning is in the process of planning many work packages which are being added to the work schedule. Work has already begun on some items and many items will begin working shortly; a vent. system shutdown is required for several of these. A TOP and NSE are being prepared to justify the temporary shutdown and control the work activities. Planning and Technical Services have evaluated when the work could be performed, and which tasks would require an LCO entry, system shutdown, or plant outage so that the work could be planned in the most efficient manner. In order to support these work activities, the Site Engineering Department is working on the following modifications necessary to be completed prior to restoring the system to normal:

- S-93-027 - Replace bag filters in CR & RR; 70F-17/18.
- E-93-124 - 70TCV-120A,B & 121 A,B actuator substitution.
- M1-91-301 - Replace existing heater controls for 70E-7, 8, 16
- M1-92-353 - Replace 70DPS-111A,B with a flow switch; and replace 70DPS-112A,B with switches with expanded range.
- E-93-145 - Install blank plate to isolate intakes while repairing

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

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LEAD INDIVIDUAL: LAZARUS

DEPARTMENT: JAF-TS

MOV's.

M1-92-394 - Replace damper actuators.

- Installation of Test Ports in Ventilation Ductwork.

WORK TO BE ACCOMPLISHED NEXT PERIOD:

Continue progress on issues required for "pre-restoration" as described in the plan.

The major unresolved design issues which are currently being evaluated by NED are:

1. Determine Control Room envelope relative pressurization requirements, and prepare any resulting mod proposals necessary. There is a strong likelihood that a mod will be required to install dp monitoring devices for all areas adjacent to the Control Room envelope.

Expected Determination Date: 1/15/94

2. Determine if a single SBM control switch for controlling both emergency CREVASS trains is acceptable based on single failure criteria. Note that the same situation exists for the Relay Room control switch, and the decision was made to install a second switch and logic train via Mod F1-92-377 ('95 RPO). If two Control Room switches are required, a similar Mod to install a second switch & logic train in the Control Room will be required. Note that from a Licensing position, the single switch is currently considered acceptable because its existence was identified in our original NUREG-0737 response, and this submittal was accepted in the NRC response.

Expected Determination Date: 12/31/93

3. Review the results of the SWEC CREVASS Single Failure Evaluation Report, and provide recommendations to address any identified discrepancies prior to restoring the system to normal. (All items have previously been reviewed for operability prior to plant startup while in the "isolate" mode.)

Expected Determination Date: 12/31/93

4. Determine if the maximum unfiltered system inleakage rate is 13,150 CFM (rather than the assumed 1500 CFM). Based on the maximum flow and the resulting allowable time to initiate system isolation, determine if a modification to provide automatic isolation is necessary. Note that Standard Review Plan Section 6.4.III.3.d(3) does not allow crediting manual action for time periods less than 20 minutes. The existing analysis allows 30 minutes at an inleakage rate of 1500 CFM. The habitability analysis will have to be updated as necessary.

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

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LEAD INDIVIDUAL: LAZARUS

DEPARTMENT: JAF-TS

Expected Determination Date: 12/31/93

5. Review cable routing and locate all applications where existing configuration is inadequate to meet separation criteria or single failure requirements. Determine what modification scope is necessary to correct any identified deficiencies. One known example is the auto-transfer to start the standby emergency fan that was temporarily addressed for startup by Mod M1-93-141.

Expected Determination Date: 1/31/92

6. Perform the same type of cable routing review for the Relay Room since its design is similar to the Control Room design.

Expected Determination Date: 3/15/94

7. Determine if information regarding chemical storage at Nine Mile Point Unit I and the Site Independence Station (under construction) needs to be factored into the Control Room habitability evaluation. If toxicity levels at the Control Room exceed allowable limits, then a modification to install chemical detection instrumentation may be necessary.

Expected Determination Date: 1/15/94

The development of the Control Room and Relay Room Ventilation DBD is in progress. A meeting was held at JAF on November 17, 1993 to discuss the 3D freeze package boundaries and schedule. The draft DBD is expected to be submitted by SWEC on 1/31/94, with final issue of the DBD by 4/25/94.

The major remaining licensing issues are:

Development and approval of technical specification interpretation for Section 3.11.A and 4.11.A to clarify guidance for determining CREVASS operability.

Expected Due Date: 12/31/93

2. Evaluation of the physical separation criteria for the CREVASS emergency train equipment.

Expected Due Date: 12/31/93

3. Update of LER-93-09 to document all inadequacies identified in our original response to NUREG 0737 (JPN-81-60).

Expected Due Date: 2/28/94

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

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TITLE: CONTROL & RELAY ROOM VENTILATION

LEAD INDIVIDUAL: LAZARUS

DEPARTMENT: JAF-TS

FINAL TECHNICAL ISSUE DISPOSITION: (i.e. mod initiated, procedure rev. etc.)

Restoration of system to normal operating mode and resolution of all follow-up "post-restoration" tasks identified in the Control Room plan.

In order to restore the CREVASS to its normal mode of operation, based on the current status, the following work is required:

1. Completion of all WR, PID, and modification work items identified to Planning as required prior to system restoration.

2. Completion of system balancing after hardware and equipment repairs.

Note: Balancing of the system will require placing the system in the normal mode of operation during this evolution; therefore, unless all outstanding design issues are resolved prior to balancing, this work, must be performed during an outage when the system is not required to be operable. Furthermore, in order for the balancing to be effective, work activities identified to planning as necessary prior to balancing should be completed before the balancing activities are undertaken.

3. Completion of the identified design issues, along with any resulting modifications.

4. Completion of the identified licensing issues.

5. Development and performance of a revised surveillance test (ST-18) to properly demonstrate system operability. This item is dependent upon the outcome of the design issues, tech. spec. interpretation, and DBD.

6. Determination of how to address design leakage through the isolation MOD's and control of their associated bypass dampers. If system balancing can establish a positive pressure upstream of the MOD with the respective outboard MOV failed open (outward inleakage), and the inaccessible bypass can be permanently closed, then surveillance testing may be allowable to demonstrate acceptability. If this is not attainable, a modification to install a zero-leakage MOD or MOV as a replacement for this boundary may be necessary.

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 2

STATUS: OPEN

TITLE:

FIRE PROTECTION PROGRAM / APPENDIX 'R'
IMPROVEMENTS CEAR CJ114C

LEAD INDIVIDUAL: V.K. KAPUR

DEPARTMENT: PROJECT ENG.

PROBLEM STATEMENT:

Fire protection program improvements.

IMPACT ON PLANT OPERATION: (REGULATORY, OPERATING RESTRICTIONS, DESIGN ETC.)

Inability to demonstrate compliance with Fire Protection regulatory commitments.

Potential ANI surcharges.

Inability to demonstrate compliance with the Life Safety Code.

REFERENCE DOCUMENTS:

10CFR50.48
17CFR50 Appendix R
N C IN 88-04
L R 93-006
JPN-91-050
JPN-92-014
N-92-032
-93-031
71-92-377
71-91-355
-92-307
-92-109

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 2

STATUS: OPEN

TITLE: FIRE PROTECTION PROGRAM / APPENDIX 'R'
IMPROVEMENTS CEAR CJ114C

LEAD INDIVIDUAL: V.K. KAPUR

DEPARTMENT: PROJECT ENG.

F1-93-017
F1-92-344
M1-91-212
NRC IN 88-56
BTP 9.5-1 Appendix A
Generic Letter 86-10
Generic Letter 88-12
LER 92-004
NFPA 90A
NFPA 13, 14, 15, 16, 24
NFPA 72, 72E, 26

INTERDEPARTMENTAL INTERFACES:

DEPARTMENT	INDIVIDUAL	PHONE
NED	BLOISE	6271
FP	MACDONALD	6766
PES	KOHR	3320
PES	CACCAVALE	6591
PES	V.K. KAPUR	6293

ACTION PLAN:

Prepare work scopes, preliminary engineering packages, and milestone schedules for the following modifications:

1. Fire Pump 76P-4 Zebra Mussels Resolution
2. ESW/RHR Pump Room and West Diesel Fire Pump Room Ventilation
3. Refueling Floor Smoke Detectors
4. Fire Door Testing/Replacement
5. Upgrade of Additional Fire Dampers F1-91-212

Update Fire Hazards Analysis, including new combustible loading calculation. Scheduled completion date 12/93.

Install Cable Tunnel Suppression System Modification. Scheduled completion date 12-31-93.

Install Cable Relay Room CO2 System Modification. Scheduled completion during 1995 Refueling outage

Perform NFPA Code compliance walkdown 2/94.

Incorporate penetration seals database into ECRIS. Scheduled

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 2

STATUS: OPEN

FILE:

FIRE PROTECTION PROGRAM / APPENDIX 'R'
IMPROVEMENTS CEAR CJ114C

LEAD INDIVIDUAL: V.K. KAPUR

DEPARTMENT: PROJECT ENG.

completion 2/28/94.

Develop hydraulic calculations. Scheduled completion to be determined upon receipt of cost estimate and schedule from Quadren Corp.

WORK ACCOMPLISHED THIS PERIOD:

- Issued schedule of services for hydraulic calculation to Quadren Inc.
- Awarded contract for preliminary engineering packages for replacement of refueling floor detectors and resolution of zebra mussel problems for fire pump 76P-4.
- Prepared the schedule of services for screenwell analysis and CO2 fan testing.
- Completed fire hazard analysis and suppression effects analysis.
- Completed cable tunnel suppression system modification Mod. F1-92-109.
- Reviewed and incorporated site comments on NFPA code walkdown project instructions.

WORK TO BE ACCOMPLISHED NEXT PERIOD:

- Evaluate proposal for fire door testing and award contract.
- Review preliminary engineering packages to zebra mussel mod. for fire pump 76-4 and refuel floor detection modification.
- Continue review of NFPA code walkdown project.

FINAL TECHNICAL ISSUE DISPOSITION: (i.e. mod initiated, procedure rev. etc.)

Incorporate combustible loading into the NYPA ECRIS Fire Protection Schedule. Scheduled completion date 2/2.

Complete NFPA Code compliance walkdown.
[02/94]

Incorporate penetration seals database into NYPA ECRIS Fire Protection

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 2

STATUS: OPEN

TITLE:

FIRE PROTECTION PROGRAM / APPENDIX 'R'
IMPROVEMENTS CEAR CJ114C

LEAD INDIVIDUAL: V.K. KAPUR

DEPARTMENT: PROJECT ENG.

Module.
[2/28/94]

Issue final isometrics and hydraulic calculations. Schedule to be determined upon receipt of cost estimate and schedule from Quadren Inc.

JAMES A. FITZPATRICK NUCLEAR POWER PLANT
**TECHNICAL ISSUES
COMPLETE REPORT**
DATE 12/08/93

TECHNICAL ISSUE NUMBER: 3
STATUS: OPEN
TITLE: FUEL POOL CLEANUP ACTIVITIES
LEAD INDIVIDUAL: J. GOLDSTEIN DEPARTMENT: WPO-CHEMISTRY

PROBLEM STATEMENT:

A full core off load can not currently be performed. In order to perform a core off load new high density fuel storage racks must be installed in the spent fuel pool which require removal of the irradiated hardware (control blades, LPRM's, dry tubes, IRM's, shroud head bolts and jet pump beam bolts) that is currently being stored in the pool.

IMPACT ON PLANT OPERATION:

Currently full core off-load can not be performed due to a lack of open channels in the fuel pool racks. A prerequisite to installation of the last remaining set of high density storage racks is to remove the irradiated hardware hanging in the pool.

REFERENCE DOCUMENTS:

Contract No. S 93 46285
WR No. 93-00077

INTERDEPARTMENTAL INTERFACES:

DEPARTMENT	INDIVIDUAL	PHONE
RES	J. Sipp	6701
PES	R. Salcedo	6295
RES	J. Solini	6704
NUC. GEN.	K. Neal	3528

ACTION PLAN:

Contract with a WasteChem to volume reduce, process, package and transport non-fuel bearing irradiated hardware in the spent fuel pool. This includes 51 control rod blades, 51 velocity limiters, 15 LPRM's, 6 dry tubes, 4 IRM strings, 3 SRM strings, 36 shroud head bolts and 20 jet pump hold down beams. The 285 stellite roller bearings will remain in the CNS 1-13 liner in the pool since it was not cost effective to ship < 1ft³ for > \$250,000.

WORK ACCOMPLISHED THIS PERIOD:

1. The third TN-RAM cask shipment of control rod blades and LPRM's left JAF on 12/08/93 and was off-loaded at Barnwell on 12/18/93. The contamination levels on the cask at Barnwell exceeded the DOT allowable levels due to leaching through the stainless steel. A meeting was held at Barnwell with the state of South Carolina, Chem Nuclear (burial operator), WasteChem (cask lessee) and NYPA. A NRC violation was given to JAF for the higher than allowable contamination rates on the cask. The cask was decontaminated with a solution of citric acid at Barnwell and kept through the holidays. Due to the meetings being held, the contamination rates on the cask rain and high winds at Barnwell, the cask took an extra week to be off-loaded.
2. The fourth TN-RAM cask shipment is scheduled to arrive at JAF late on 1/05/94 if the surveys taken on the cask on 1/3/94 are acceptable.
3. All the remaining boron tubes were collected off the floor and placed in steel buckets. WasteChem attempted to place all these tubes in the two remaining liners but there was not sufficient room. A quote on a stainless steel liner will be compared with the cost of an extra shipment.
4. A final survey of the pool was performed by WasteChem with all fuel cells verified free of boron tubes. The remaining two liners were relocated, surveyed and the floor underneath them was vacuumed.
5. All the WasteChem equipment (tools and LPRM cutter) except that needed to support the TN-RAM shipment was removed from the reactor building and sent off-site.

WORK TO BE ACCOMPLISHED NEXT PERIOD:

Perform one additional TN-RAM cask shipment (4 out of a total of 5) based on recontamination rates, the weather conditions for travel and at Barnwell, for off-loading the cask.

This project (spent fuel pool cleanup) is not expected to be completed until March 1994 due to the availability of the casks. The vendor, WasteChem has moved the schedule up as much as possible but the cask is reserved for Dresden, Peach Bottom and Nine Mile Pt. for most of January and February. Currently the fifth and final TN-RAM cask for control rod blades and the SEG 10-142 cask for filters are scheduled to arrive the week of February 20 and 27 respectively, while the remaining 18 shroud head bolts will be loaded in the prefabricated box during after the filters are loaded. The IRM and SRM strings will also be placed in the two remaining liners. Additionally, two vacuum/filter units in the pool will be placed in a square top, shielded liner during this time.

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 11/11/1993

TECHNICAL ISSUE NUMBER: 4

STATUS: OPEN

TITLE: REACTOR VESSEL WATER LEVEL

LEAD INDIVIDUAL: LOMBARDOZZI

DEPARTMENT: I&C-WPO

PROBLEM STATEMENT:

A potential has been identified for significant Reactor Pressure Vessel (RPV) Water Level errors during rapid and slow depressurization below 450 psig. The concern is that dissolved gasses will come out of solution and displace reference leg water causing non-conservative reactor level indication (high) to the operators. This has become a national public issue requiring rapid NRC mandate for resolution.

NRC Bulletin 93-03 issued May 28, 1993 requires quick response.

IMPACT ON PLANT OPERATION: (REGULATORY, OPERATING RESTRICTIONS, DESIGN ETC.)

Operable as long as level mismatches do not exceed 6" as analyzed by Emergency Procedure Guidelines (EPG's).

NRC Bulletin 93-03 states that any plant which goes to cold shutdown after July 30, 1993 will not be allowed to restart unless the plant reactor water level system has been modified to preclude the occurrence of "notching".

REFERENCE DOCUMENTS:

NRC Generic Letter 92-04 - Resolution of the Issues Related to Reactor Vessel Water Level Instrumentation in BWR's Pursuant to 10CFR50.54 (F)

JTS-93-0076 - Reactor Vessel Condensing Chamber Thermocouple Mod.
M1-92-338 Phase II, WR 098702 - Technical Services Recommendations on Cable to EPIC Necessity

NRB 93-03 - Resolution of Issues Related to Reactor Vessel Water Level Instrumentation in BWR's

Letter dated April 7, 1993 from "We the People" Attorney Ernest L. Hadley to Joseph L. Lieberman, Chairman of US Senate Committee over the NRC

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 10/30/1993

TECHNICAL ISSUE NUMBER: 10

STATUS: OPEN

TITLE: EHC

LEAD INDIVIDUAL: TALLENTS

DEPARTMENT: MAINT

Responsible Department: SED

Item 2: Hydraulic System Testing

- Develop test and submit for approval by January 3, 1993. Implement test plan during 1994 Maintenance Outage.

Responsible Department: JMD

Item 3: EHC Pump Modification F1-89-107

- Modification assigned to: G. Ottman.

Responsible Department: SED

- Purchase Order for materials to be issued on 6/8/94.

Responsible Department: SED

WORK ACCOMPLISHED THIS PERIOD:

Item #1

Meeting was held with GE during week of 10/16/93 on the subject of purchase order items such as start-up, spare parts, etc.

Item #3

Modification schedule prepared.

Modification F1-89-107 added to activities list for 1995 RFO. *START AFTER 93 OUTAGE*

WORK TO BE ACCOMPLISHED NEXT PERIOD:

preliminary design/scoping of modification by SED (Ottman).

FINAL TECHNICAL ISSUE DISPOSITION: (i.e. mod initiated, procedure rev. etc.)

JAMES A. FITZPATRICK NUCLEAR POWER PLANT
TECHNICAL ISSUES
COMPLETE REPORT

DATE 12/16/1993

TECHNICAL ISSUE NUMBER: 11

STATUS: OPEN

TITLE: INTERIM RADWASTE STORAGE FACILITY

LEAD INDIVIDUAL: DiRocco

DEPARTMENT: RES - WPO

PROBLEM STATEMENT

The Authority constructed an interim onsite storage facility in 1985 to temporarily store low level radioactive waste produced at FitzPatrick in anticipation of denial of access to offsite radioactive waste disposal facilities. Low level waste would be stored until a New York State disposal facility became available.

At the present time, FitzPatrick's access to offsite low level waste disposal is provided through a contract between New York State and a compact commission in South Carolina. The contract, according to South Carolina law, calls for termination of our disposal privileges on June 30, 1994, or sooner upon sixty days' notice from the compact commission. The most recent schedule for the New York State disposal facility calls for operation in November 2001.

The FitzPatrick interim waste storage facility must be reevaluated, restored, and readied for use in anticipation of loss of offsite disposal in 1994 pending the operation of a New York State disposal facility.

IMPACT ON PLANT OPERATION

If FitzPatrick were to lose access to offsite disposal without the capability for onsite storage, the plant would begin to accumulate spent by-product materials from radioactive water collection, purification and processing systems in tanks and vessels. Short-term accumulation is acceptable, routine practice. Long-term accumulation in the plant would disturb plant chemistry, increase personnel exposures and degrade radiation protection programs. Excess accumulation of by-product materials without additional installed tank capacity might eventually result in the loss of ability to run the plant.

The purpose of the onsite storage facility is to minimize the adverse effects on plant operations as a result of a temporary lack of disposal capacity by providing an approved, controlled location where spent radioactive materials prepared for eventual disposal can be safely stored until offsite disposal resumes.

Shipment for offsite disposal was the original design basis for the plant, and offsite disposal has been in continuous practice since initial startup. The FitzPatrick Operating License and Technical Specifications do not prohibit onsite storage of low level waste.

The Nuclear Regulatory Commission has issued guidance that allows nuclear power licensees to add low level radioactive waste storage capacity in accordance with the provisions of 10 CFR 50.59 under certain conditions. The major regulatory obstacle in the NRC guidance is the limitation on waste storage to a period of not more than five years.

REFERENCE DOCUMENTS

1. NRC Generic Letter 81-38.
2. NRC IE Notice 90-09
3. NUREG-0800 Chapter 11.4 Appendix A
4. Nuclear Safety Evaluation JAF-SE-85-140 (PORC Mtg. No. 85-094)

**JAMES A. FITZPATRICK NUCLEAR POWER PLANT
TECHNICAL ISSUES
COMPLETE REPORT**

DATE 12/16/1993

TECHNICAL ISSUE NUMBER: 11

STATUS: OPEN

TITLE: INTERIM RADWASTE STORAGE FACILITY

LEAD INDIVIDUAL: DiRocco DEPARTMENT: RES - WPO

INTERDEPARTMENTAL INTERFACES:

The designated project organisation is attached at the end of this report. The major organisational interfaces are:

RES-WPO	A. DiRocco	718-6305
RES-JAF	J. Solini	6704
Tech Services	B. MacDonald	6566
Proj Engineering	F. Okas	718-6816
Engineering - JAF	J. Erkan	6504

ACTION PLAN:

A revised action plan is being prepared based on review by the designated project organisation.

WORK ACCOMPLISHED THIS PERIOD:

Since the engineering meeting of November 9, 1993, the following activities were accomplished:

- o RES-WPO reviewed the facility shielding evaluation to determine the effect of container models on doses to adjacent buildings, the new fence line and the site boundary.
- o RES-WPO recommended modification to the fire detection system in accordance with NED recommendations.
- o PE-WPO initiated modification package development for the fire detection improvement.

WORK TO BE ACCOMPLISHED NEXT PERIOD:

- o RES-JAF will update the status of the facility radiation protection monitoring system.
- o PE-WPO will propose modification package for fire detection system and will identify the status of other modifications. RES-WPO will identify funding source based on PE estimate.
- o RES-WPO will initiate revisions to the nuclear safety evaluation based on the final recommendations from the shielding evaluation.

FINAL TECHNICAL ISSUE DISPOSITION:

The project will culminate in the operability of the interim waste facility by April 1994.

JAF INTERIM WASTE STORAGE FACILITY - PROJECT ORGANIZATION

Organization	Technical Function	Cognizant Manager	Assigned Staff
Rad & Env - WPO	Project Management	J.Kahabka	A.DiRocco
	Shielding Analyses	G.Re	Anuah R.
	Waste Gas Generation	R. Grover	K.Neal
Rad & Env - JAF	Project Management Support	J.Solini	J.Solini
	Health Physics	M.McMahon	
	Rad Engineering	J.Solini	K.Peper
	Rad Shipping	J.Solini	A.Young
Operations - JAF	Waste Streams & Processing	S.Allen	D.Robert
Project Engineering - WPO	Project Engineer	T.Dougherty	P.Okas
Nuclear Engineering Division	Fire Protection Assessment	F.Biolee	J.Pechacek
			J.Sepahpur
Tech Services	Fire Protection Systems and Compliance	D.Wallace	B.MacDonald
	Rad & Effluent Monitoring Systems	D.Wallace	B.Burnsworth
	Sampling Systems	D.Wallace	B.Burnsworth
	Electrical & Crane	D.Wallace	T.Bostian
Site Engineering Division	Design & Engineering	D.Ruddy	J.Erkan
	Mod Packages	D.Ruddy	C.Patrickson
	Field Engineer	D.Ruddy	C.Patrickson
Nuclear Licensing	Licensing Review	J.Gray	J.Hoddy
Construction Services	Construction Management	J.Fitzgerald	K.Kilpeck
Safety & Security	Safety Assessment	T.Ambrose	J.Parsons
	Safety Compliance	T.Ambrose	J.Parsons
	Security	T.Telfer	J.Haley

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 11/11/1993

TECHNICAL ISSUE NUMBER: 4

STATUS: OPEN

TITLE: REACTOR VESSEL WATER LEVEL

LEAD INDIVIDUAL: LOMBARDOZZI

DEPARTMENT: I&C-WPO

INTERDEPARTMENTAL INTERFACES:

DEPARTMENT	INDIVIDUAL	PHONE
WPO Licensing	Fish	3230
TS	Carlson	6326
WPO	Lauman	6281
SED	Moore	6531
ORG	Fromm	6209

ACTION PLAN:

1. Install backfill modification (pre-outage work).
2. Develop schedule for design, installation and training.
3. Resolve all analytical issues that support the Nuclear Safety evaluation.
4. Prepare preoperational testing procedure and resolve all operability issues.

WORK ACCOMPLISHED THIS PERIOD:

ISSUE #1 - BACKFILL MODIFICATION

The backfill modification tie-ins to the reference legs and Control Rod Drive System (CRD) was initiated at the beginning of the 1993 maintenance outage on October 23, 1993.

To date, the refuel zone, 3A and 3B, reference leg tie-ins have been completed and hydro leak tested.

Work on reference leg 2A is in progress and should be tied in and hydro tested by days' end, October 30, 1993.

Reference leg 2B is scheduled to be tied-in and hydro tested on Saturday, October 30, 1993.

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 11/11/1993

TECHNICAL ISSUE NUMBER: 4

STATUS: OPEN

TITLE: REACTOR VESSEL WATER LEVEL

LEAD INDIVIDUAL: LOMBARDOZZI

DEPARTMENT: I&C-WPO

Pending Operations Department release of the CRD for testing, the entire backfill system will be system hydro leak tested on Sunday, October 31, 1993.

Joe Klevorn of Operations is doing a walkdown and validation of the preoperational testing (POT), October 29, 1993.

The POT is scheduled to go to PORC on Monday, November 1, 1993.

The backfill modification is on schedule.

The resident NRC inspector, Bill Cook has requested copies of the modification, NSE, and the Proto-Power thermo-hydraulic study.

ISSUE #2 - CONDENSING CHAMBER THERMOCOUPLES

Observed operation of all condensing chamber thermocouples since recent startup. All are functioning. The performance of the 3A and 3B chamber continued to show stability. Preliminary indications showed slightly degraded performance from the 2A and 2B chamber (temperatures were dropping). Some industry experience showed that condensing chambers temperatures can sometimes rise again after leaks are repaired. Some reference line leaks have been identified and need to be corrected soon. Thermocouple data is being tracked and evaluated on a weekly basis. The most recent data indicates a stabilizing trend and no degradation in performance.

WORK TO BE ACCOMPLISHED NEXT PERIOD:

BWROG ACTION ITEMS

Analytical Model Application to JAF

Receive training and insert JAF specific x, y, z coordinates of reference leg geometries into the BWROG supplied analytical model software Level Error (LEER) Design and Drafting has completed generating the X, Y, Z coordinates from isometrics.

JAF CONDENSING CHAMBER THERMOCOUPLES - Continue Monitoring

FINAL TECHNICAL ISSUE DISPOSITION: (i.e. mod initiated, procedure rev. etc.)

- Backfill mod is currently being installed on all reference legs.

- The project team has been established and responsibilities have been assigned.

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 11/11/1993

TECHNICAL ISSUE NUMBER: 4

STATUS: OPEN

TITLE: REACTOR VESSEL WATER LEVEL

LEAD INDIVIDUAL: LOMBARDOZZI

DEPARTMENT: I&C-WPO

- Procedures are in place and on schedule to meet the NRC Bulletin 93-03 short term requirements.
- The stress analysis needed to support the nuclear safety evaluation for system operability was issued on September 2, 1993.
- Appendix R issues in process of being resolved.
- Out of service time for reactor water level operability.

*Present plans are to close
this issue at the December
Engineering Mtg.*

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 5

STATUS: OPEN

TITLE: SHROUD CRACKING

LEAD INDIVIDUAL: LAFFERTY

DEPARTMENT: NUC MAINT-WPO

PROBLEM STATEMENT:

This issue first occurred in Europe. The first US report was at Brunswick in August of this year. The second report was at Peach Bottom 3 in September. GE has issued a SIL 572R1 to address the cracking. The SIL identifies 304 S/S as the common connection.

Our shroud is carbon 304 S/S made by Sun-Ship about the same time as Brunswick's. GE is drafting a white paper for safety implementation, but as a minimum we must be prepared to inspect, and if required, make repairs in the 1995 outage.

IMPACT ON PLANT OPERATION: (REGULATORY, OPERATING RESTRICTIONS, DESIGN ETC.)

REFERENCE DOCUMENTS:

SIL 572R1

IRC Info. Notice

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 5

STATUS: OPEN

TITLE: SHROUD CRACKING

LEAD INDIVIDUAL: LAFFERTY

DEPARTMENT: NUC MAINT-WPO

INTERDEPARTMENTAL INTERFACES:

DEPARTMENT	INDIVIDUAL	PHONE
NUC-Maint	Lafferty	6939
Proj Eng	Kohr	
I&C	Reno	6109
Chemistry	Goldstein	6285
Chemistry	Jarvis	6721

ACTION PLAN:

Engineering report to provide justification for continued operation. Completed 11/93.

Perform engineering evaluation to increase hydrogen injection (14 scfm to 20 scfm). Ongoing

Required reading to detect shroud cracking is complete. Shroud during the next refuel outage.

Meeting to discuss inspection and repair with GE scheduled for January 12, 1993.

Contingency mod to repair shroud. Still under evaluation.

WORK ACCOMPLISHED THIS PERIOD:

Met with the BWROG and the NRC to discuss the generic shroud inspection/repair guideline.

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 5

STATUS: OPEN

TITLE: SHROUD CRACKING

LEAD INDIVIDUAL: LAFFERTY

DEPARTMENT: NUC MAINT-WPO

WORK TO BE ACCOMPLISHED NEXT PERIOD:

Issue increased H2 report by 1/30/93.

Meet with GE.

FINAL TECHNICAL ISSUE DISPOSITION: (i.e. mod initiated, procedure rev. etc.)

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 6

STATUS: OPEN

TITLE: AUXILIARY BOILER SYSTEM

LEAD INDIVIDUAL: KOHR

DEPARTMENT: WPO-PES

PROBLEM STATEMENT:

Due to contamination event of March 1991, the Auxiliary Boilers are no longer available to supply steam to various systems and components (nitrogen vaporizer, radwaste system, plant heating, etc.). Compensatory measures have been taken to permanently install a hot water boiler for plant heating, a demineralizer system for radwaste processing, an electric CO2 vaporizer for generator purging, and a temporary boiler to supply steam for nitrogen vaporization during drywell inerting. Outstanding concerns include determining the disposition of the contaminated equipment in the Auxiliary Boiler Building, deciding on the need for a second hot water boiler, providing permanent storage for No. 2 fuel oil used by the hot water boiler, and providing a permanent method of nitrogen vaporization. Two modifications have been initiated - F1-92-146 conversion to No. 2 oil, and F1-92-145 nitrogen vaporizer replacement.

IMPACT ON PLANT OPERATION: (REGULATORY, OPERATING RESTRICTIONS, DESIGN ETC.)

The Auxiliary Boiler System is non-safety related QA Category II/III. Therefore, there are few regulatory issues involved. The capacity limitations of the current fuel oil storage arrangements limits the system to a one day supply of oil during peak load operation. The temporary boiler being utilized for supplying steam to the nitrogen vaporizer requires repeated use of the temporary mod process, which is undesirable from both the operations and regulatory standpoint. The single hot water boiler has no redundant backup source for plant heating, resulting in loss of plant heating in the event of inoperability of the hot water boiler.

REFERENCE DOCUMENTS:

None

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 6

STATUS: OPEN

TITLE: AUXILIARY BOILER SYSTEM

LEAD INDIVIDUAL: KOHR

DEPARTMENT: WPO-PES

INTERDEPARTMENTAL INTERFACES:

DEPARTMENT	INDIVIDUAL	PHONE
PES-WPO	KOHR	3320
SED	PATRICKSON	6950
CS	NEFF	6951
NED-WPO	STENNER	6411
TS-JAF	BURNSWORTH	6519

ACTION PLAN:

Establish a project team to evaluate outstanding issues and recommend modifications or maintenance.

Implement Mod. F1-92-145 Electric Nitrogen Vaporizer.

Implement Mod. F1-92-146 Conversion to No. 2 Fuel Oil.

Implement additional modifications or other actions as they are recommended by the project team and approved.

WORK ACCOMPLISHED THIS PERIOD:

F1-92-145 - NITROGEN VAPORIZER

NYP&A review of PEP and equipment purchase specification completed and comments returned to Profo-Power. Critical path to mod completion is through equipment delivery, forecast for August 1994.

F1-92-146 - CONVERSION TO NO. 2 FUEL OIL

No. 6 fuel oil removal by Clean Harbors initiated during week of December 19, with approximately 9,000 gallons removed. Oil removal to continue in January.

Cataract continued preparation of detailed engineering package, while

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 6

STATUS: OPEN

TITLE: AUXILIARY BOILER SYSTEM

LEAD INDIVIDUAL: KOHR

DEPARTMENT: WPO-PES

providing field engineering support to oil removal effort.

WORK TO BE ACCOMPLISHED NEXT PERIOD:

Mod. No. F1-92-145 - Proto-Power to incorporate comments on PEP and purchase specification and review with NYPA. Proto-Power to issue final documents upon NYPA concurrence with comment resolution.

Mod. No. F1-92-146 - Complete oil removal and tank cleaning by end of January. Cataract to inspect above-ground oil tank after completion of cleaning. Cataract to continue detailed engineering effort, including possible tank repairs resulting from inspection and purchase specification for removal of underground tank and piping.

Clear Engineering to initiate study of alternatives for redundant source plant heating.

FINAL TECHNICAL ISSUE DISPOSITION: (i.e. mod initiated, procedure rev. etc.)

Mod. F1-92-145 - Electric Nitrogen Vaporizer scheduled to be operational by October 1994.

Mod. F1-92-146 - Conversion to No. 2 Fuel Oil scheduled to be operational by October 1994.

Complete evaluation of outstanding issues and make recommendations for resolution by 2/28/94 (delayed due to other priority work).

Dissolve Project Team and turn over responsibility to System Engineer - Tech Services by 3/31/94.

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 7

STATUS: OPEN

TITLE: RADWASTE IMPROVEMENTS

LEAD INDIVIDUAL: HARTJEN

DEPARTMENT: WPO-RH&C

PROBLEM STATEMENT:

The waste concentrators in the Radwaste System are not being used due to cross contamination of the auxiliary boiler which provides steam to the units. Various alternatives are being evaluated for the long term operation of the Radwaste System.

Leased radwaste processing system is currently in use. Need to evaluate alternates to leased system to reduce high operating costs, increasing disposal costs, eliminate current operating problems, and improve system operability, reliability, and performance.

IMPACT ON PLANT OPERATION: (REGULATORY, OPERATING RESTRICTIONS, DESIGN ETC.)

High cost of operating leased ion exchange equipment which is being used in lieu of waste concentrators.

Modifications will reduce both operating and disposal costs with a reduction in solid wastes generated. Chemistry will improve and will allow us to achieve EPRI and INPO Chemistry Guidelines.

REFERENCE DOCUMENTS:

Vance Associates, "An Evaluation of Radwaste Processing Facility"

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 7

STATUS: OPEN

FILE: RADWASTE IMPROVEMENTS

LEAD INDIVIDUAL: HARTJEN

DEPARTMENT: WPO-RH&C

INTERDEPARTMENTAL INTERFACES:

DEPARTMENT	INDIVIDUAL	PHONE
PES-WPO	Salcedo	6295
Operations	D. Robert	6314
Chemistry	Jarvis	6721
Chemistry	McKeen	6703
T.S.	C. Burnsworth	6519

ACTION PLAN:

Determine scope of long term operation of Radwaste System. Presentation to JAF Management took place on June 10.

JAF Technical Services coordinating new phase of evaluation to reduce scope and cost of Radwaste System improvements, and recommend most cost effective system improvements.

Prepare preliminary engineering package for long term modifications. PEP preparation part of evaluation scope. Performed by Vance & Associates.

WORK ACCOMPLISHED THIS PERIOD:

Modification No. F1-93-163 assigned to this project. Modification proposal form issued for site concurrence.

Draft of project schedule complete. Project schedule milestones identified.

Preliminary Engineering Package complete and issued for comments. Comments due 1/22/94.

Funding (CEAR, Trustee Item) process continues. Target the March, 1994 Trustee Meeting.

JAMES A. FITZPATRICK NUCLEAR POWER PLANT
TECHNICAL ISSUES
COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 7

STATUS: OPEN

TITLE: RADWASTE IMPROVEMENTS

LEAD INDIVIDUAL: HARTJEN

DEPARTMENT: WPO-RH&C

WORK TO BE ACCOMPLISHED NEXT PERIOD:

Continue process to obtain funding for this modification.
Issue final revision to Radwaste Assessment Study.
Issue final project schedule.

FINAL TECHNICAL ISSUE DISPOSITION: (i.e. mod initiated, procedure rev. etc.)
Modification Initiated - No Schedule Yet

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 8

STATUS: OPEN

TITLE: MEL/GENERIC LETTER 83-28

LEAD INDIVIDUAL: ELLMERS/MOODY

DEPARTMENT: LIC/CONFIG

PROBLEM STATEMENT:

The NRC staff identified potential programmatic weaknesses in the Authority's implementation of Generic Letter 83-28 (Reference 1) during inspections at FitzPatrick in 1991 and 1992. These weaknesses were identified during inspections conducted in November 1991 (Reference 2), February 1992 (Reference 3) and the Diagnostic Evaluation Team (DET) inspections in September and October of 1991.

In response, the Authority conducted a Quality Assurance Audit (Reference 4), to quantify the extent of our compliance with Authority commitments associated with Generic Letter and staff guidance in the Generic Letter. This audit was performed by an independent contractor who identified three findings and twenty-six observations. The audit generally supported and confirmed the weaknesses found by the NRC staff during their inspections.

A second audit (Reference 5) was conducted to determine if the deficiencies identified in the first audit had resulted in actual hardware problems or deficiencies. This audit was also performed by an independent organization outside of the Authority. This assessment identified eight corrective actions and two recommendations. This audit confirmed the results of the first audit.

IMPACT ON PLANT OPERATION: (REGULATORY, OPERATING RESTRICTIONS, DESIGN ETC.)

Operability of structures, systems, and components are not affected by this issue.

Remarks: GL 83-28 was issued ten years ago. The NRC assessed the current level of compliance a year ago. The Authority presented preliminary plans and schedules for resolving this issue to the inspection team a year ago and committed to submit a status update at that time.

REFERENCE DOCUMENTS:

1. NRC Generic Letter 83-28, "Required Actions Based on Generic Implications of Salem Anticipated Transients Without Scram (ATWS) Events," issued July 8, 1993

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 8

STATUS: OPEN

TITLE: MEL/GENERIC LETTER 83-28

LEAD INDIVIDUAL: ELLMERS/MOODY DEPARTMENT: LIC/CONFIG

2. NRC Letter C.J. Cowgill to R.J. Converse, dated November 21, 1991, "NRC Region 1 Inspection Report No. 50-333/91-20."

3. NRC Letter C.W. Hehl to R.J. Converse, dated February 20, 1992, "NRC Region 1 Inspection Report NO. 50-333/91-22."

4. NYPA Quality Assurance Audit Report (QAAR) 92-07

5. Independent Assessment JTS-92-0904

Refer to the action plan for a complete list of related documents.

Design Basis Document Information: No design basis document information is related to this issue.

INTERDEPARTMENTAL INTERFACES:

DEPARTMENT	INDIVIDUAL	PHONE
Maint	DeRoy	6068
WPO-NED	Dougherty	6259
WPO-NED	Ettlinger	6560
Planning	Flaherty	6784
WPO-Licensing	Gray	6289

ACTION PLAN:

WORK ACCOMPLISHED THIS PERIOD:

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 8

STATUS: OPEN

TITLE: MEL/GENERIC LETTER 83-28

LEAD INDIVIDUAL: ELLMERS/MOODY DEPARTMENT: LIC/CONFIG

Salem ATWS - NRC Generic Letter 83-28

Comments on an updated response to Generic Letter 83-28 were requested from NYPA personnel responsible for implementing the updated plans by a memo dated February 8, 1993. Comments were requested not later than March, 1993.

If no significant comments are received, a revised response is scheduled for submittal to the NRC during March or early April.

(MARCH) Work was started on resolving the comments received to date. Judging from the number and depth of comments, neither the letter to the NRC or the supporting action plan received a thorough review. John Hoddy has asked those who have not reviewed these documents to do so at this time.

(APRIL) A meeting to discuss the plan and comments was held April 13th at FitzPatrick. Attending the April 13th meeting were P. Brozenich, K. Moody, J. Ellmers, D. Rieper, J. DeRoy, T. Landers, S. Pobutkiewicz, J. Hoddy, and J. Ellmers. Most attendees felt that the letter over committed and that some of the actions outlined in the letter were not necessary to comply with the GL. Ken Moody volunteered to act as Site liaison and mark-up both the NRC and the Action Plan letter to eliminate commitments not required. As of the end of April, a partial mark-up was received from Ken. Due to other work, he has stated that his time for future work may be limited and suggested that J. Hoddy complete the task.

(MAY) The draft action plan and letter to the NRC were revised to reflect comments from the April 13th meeting. The revised letter and plan are currently under review by K. Moody.

(JUNE) Draft letter to the NRC with "Status Update on NRC Generic Letter 83-28, Required Actions Based on Generic Implications of Salem ATWS Event" currently in formal review and concurrence cycle. Anticipate sending this report late June/early July 1993.

(JULY) Draft letter to the NRC with "Status Update on NRC Generic Letter 83-28, Required Actions Based on Generic Implications of Salem ATWS Event" has gone through the concurrence cycle and all comments have been incorporated or resolved. Update currently under final review by W. Josiger. Anticipate forwarding to R.E. Beedle for signature and transmittal to the NRC shortly.

The GL 83-28 Action Plan (which responds to individual audit findings and recommendations) will be issued by Ken Moody before July 30, 1993.

(AUGUST) Updated response to NRC on GL 83-28 was routed for concurrence starting the week of June 21st. Most comments were received and resolved by July 16, 1993. Currently working to resolve comments of W. Josiger.

The 83-28 Action Plan was not issued as scheduled due to delay associated with NRC letter. Action Plan will be issued by Ken Moody without waiting for NRC letter.

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 8

STATUS: OPEN

TITLE: MEL/GENERIC LETTER 83-28

LEAD INDIVIDUAL: ELLMERS/MOODY

DEPARTMENT: LIC/CONFIG

(OCTOBER) Generic Letter action plan issued October 1, 1993 by K. Moody. Letter to NRC reviewed by Licensing Engineer not previously involved to clarify the report and resolve outstanding comments. Revised letter and report will be sent to the NRC during November 1993.

(NOVEMBER) Updated response to GL 83-28 transmitted to NRC November 29, 1993 (JPN-93-080). A meeting was held 12/1/93 with WPO QA to discuss the GL Action Plan's responses to the associated QA Audit (May 1992). Based on that meeting, QA will accept the Action Plan as a satisfactory resolution of 2 of 3 findings and 25 of the 26 recommendations. The one outstanding finding is being held open pending additional information regarding training of WPO engineers. The one outstanding recommendation is scheduled for completion 12/31/93 (ACTS 4543, J. Kaucher) regarding the need for a WPO Procurement Engineering Group.

(DECEMBER) NYPA December 30, 1993 letter to the NRC "Revised Response to Generic Implication of Salem ATWS Events (Generic Letter 83-28) Revised Schedule for EPIC Related Modifications", (JPN-93-087), rescheduled five commitments regarding Generic Letter 83-28. See attached excerpt "Attachment I to JPN-93-087". The schedules in this letter supersedes those included with the Authority's prior response (JPN-93-080, dated November 29, 1993). Other commitments in the revised response are not affected.

Judd Ellmers and Ken Moody are continuing to assist Rich Norolan of WPO Quality Assurance to address in detail the three findings in QA Audit 92-07. R. Norolan will issue QA's report in January.

WORK TO BE ACCOMPLISHED NEXT PERIOD:

Submit updated response to NRC.

FINAL TECHNICAL ISSUE DISPOSITION: (i.e. mod initiated, procedure rev. etc.)

Attachment I to JPN-93-087

Revised Response to Generic Implications of Salem
 ATWS Events (Generic Letter 83-28)
 Revised Schedule for EPIC Related Modifications

Summary of Commitments

The schedules in Attachment I supersede those included with the Authority's prior response (JPN-93-080 dated November 29, 1993). Other commitments in the revised response are not affected.

Identification No.	Description	Date
JPN-93-087-01	Complete remainder of the EPIC upgrade, Phase 2 (modification F1-90-212). Both CPUs will be upgraded, tested and operable.	March 31, 1994
JPN-93-087-02	Review NRC draft Technical Evaluation Report Table 1.2-2 and make recommendations for changes to the EPIC database to assure that all of the required computer points have been included, unless specifically exempted.	March 31, 1994
JPN-93-087-03	Schedule modification to add additional EPIC points to the Sequence of Events (SOE) and Post-Trip Log (PTL) for proper recording and indication required to meet the guidance of Generic Letter 83-28, Table 1.2-2 "BWR Parameter List" and NRC's draft Technical Evaluation Report (TER) dated October 28, 1993.	June 30, 1994
JPN-93-087-04	Add the following data processing capabilities to the EPIC system: (1) Change the EPIC points for Emergency Diesel Generator status (D-1089 through D-1095) from regular digital points to faster resolution SOE points; (2) Change the EPIC point for Control Rod Position (all rods in/past full in, D-1086) from a digital point to a SOE point.	Thirty days after startup from the next scheduled refueling outage. (Estimated date January 31, 1995.)
JPN-93-087-05	Add SBM hand switches to MEL (PEDB) database	December 31, 1994

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 9

STATUS: OPEN

TITLE: TRANSFORMER REPLACEMENTS

LEAD INDIVIDUAL: STRANOVSKY

DEPARTMENT: MAINT-WPO

PROBLEM STATEMENT:

In response to two recent transformer failures, a gradual replacement or refurbishment of all normal and emergency 4kV/600V service station transformers is planned.

IMPACT ON PLANT OPERATION: (REGULATORY, OPERATING RESTRICTIONS, DESIGN ETC.)

Docket No. 50-333, LER 92-034

REFERENCE DOCUMENTS:

GS-92-273 and LER 92-034

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 9

STATUS: OPEN

TITLE: TRANSFORMER REPLACEMENTS

LEAD INDIVIDUAL: STRANOVSKY

DEPARTMENT: MAINT-WPO

INTERDEPARTMENTAL INTERFACES:

DEPARTMENT	INDIVIDUAL	PHONE
Maintenance	Hoy	6203
WPO-NED	Mavrikis	6273
Purchasing	Mantaro	6050
WPO-Projects	Salcedo	3293
WPO-Projects	Schmeck	6487

ACTION PLAN:

1. Replace 71T-14 and purchase spare for 71T-14 and 71T-13. (To be installed in April 1994 - Mod. F1-93-054)

2. Replace T-8, T-10, T-12 in April 1994 outage, replace T-6, T-7, T-9, T-11 in 1995 refueling outage. Mod. F1-92-295

WORK ACCOMPLISHED THIS PERIOD:

1. Modification Package F1-93-054 for replacement of transformer 71-T14 was issued for comments. Comments are due by January 15, 1994. Work on this modification continues.

2. The bid was awarded to purchase 7 normal station transformers. Delivery is scheduled for April and October of 1994.

3. Work on modification package F1-92-295 for replacement of transformers T-3 and T-10 in April 1994 continues. Revised schedule is needed.

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 01/07/1994

TECHNICAL ISSUE NUMBER: 9

STATUS: OPEN

TITLE: TRANSFORMER REPLACEMENTS

LEAD INDIVIDUAL: STRANOVSKY

DEPARTMENT: MAINT-WPO

WORK TO BE ACCOMPLISHED NEXT PERIOD:

1. Resolve all comments on modification package F1-93-054 for replacement of transformer 71-T14.
2. The transformers 71-T13 and 71-T14 were placed on QA hold pending resolution of DER-93-0899. WPO QA should disposition this DER (Action #9613).
3. In order to replace transformer T-8 and T-10 in April 1994 outage, the modification package F1-92-295 must be completed next month.

FINAL TECHNICAL ISSUE DISPOSITION: (i.e. mod initiated, procedure rev. etc.)

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 10/30/1993

TECHNICAL ISSUE NUMBER: 10

STATUS: OPEN

TITLE: EHC

LEAD INDIVIDUAL: TALLENTS

DEPARTMENT: MAINT

PROBLEM STATEMENT:

ITEM #1

General Electric made several recommendations to improve the EHC System after an investigation of a Reactor Scram caused by the EHC System. System noise was suspected to have contributed to the event. An investigation within the EHC cabinet revealed several potential sources of noise such as; DC Op Amps, 1 and 3 KHZ oscillators and speed signal filtration. High and low frequency noise tests performed at the time confirmed the noise problem. General Electric provided a list of circuit boards that they recommended be replaced. This list was pared down by M. Tallents and M. Reno, then requested through a modification proposal. The modification is presently with SED.

ITEM #2

General Electric recognized the industry (EHC) noise problem and has recommended a modification to minimize the effects of the problem in Technical Information Letter (TIL) 1123. The modification would install a hydraulic accumulator on each main turbine control valve hydraulic actuator. The accumulator would dampen the hydraulic pulsations without compromising the system loop gain. This modification shall be implemented if necessary. The hydraulic system shall be tested to determine the extent of the pressure pulsations. The testing shall be conducted before and after the EHC circuit board modification (Item #1). A recommendation shall be made after the test data has been reviewed.

ITEM #3

The EHC hydraulic power unit pumps, 94P-7A,B, should be replaced for the reasons described in the referenced documents.

IMPACT ON PLANT OPERATION: (REGULATORY, OPERATING RESTRICTIONS, DESIGN ETC.)

later

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

TECHNICAL ISSUES

COMPLETE REPORT

DATE 10/30/1993

TECHNICAL ISSUE NUMBER: 10

STATUS: OPEN

TITLE: EHC

LEAD INDIVIDUAL: TALLENTS

DEPARTMENT: MAINT

REFERENCE DOCUMENTS:

Item #1 - JMD-90-154, LER 89-020-01, WR 097459

Item #2 - GE TIL 1123, ACTS 6929

Item #3 - GE TIL 1115-3, F1-89-107, JMD-92-333, JEP-92-38, OER 9-0114

INTERDEPARTMENTAL INTERFACES:

DEPARTMENT	INDIVIDUAL	PHONE
SED	Romm	6536
SED	Savory	6522
MD	Tallents	6208
I&C	Reno	6109
SED	Ottman	6548

ACTION PLAN:

ITEM #1

Item 1: EHC Circuit Board Modification M1-92-260

Meeting was held with GE during week of 10/16/93. As a result of discussion, GE will present a modified proposal which will include start-up cost and recommendation for spare parts. SED will provide engineering assistance in review of purchase order.

Modification assigned to: Ed Romm
Responsible Department: SED

- Purchase Order for materials issued by 12/31/93

February 14, 1994

licensing action and activity. Future licensing actions planned by PASNY include those associated with a 24-month operating cycle and those associated with removing fire protection requirements from the TS.

Mr. William Cook, the NRC Senior Resident Inspector at FitzPatrick, outlined upcoming inspection activities planned for FitzPatrick. The meeting concluded with an agreement among the attendees to conduct the next quarterly licensing meeting at the FitzPatrick site late in the month of March 1994.

Original signed by:

Brian C. McCabe, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. List of Attendees
- 2. Licensee Handout Material

cc w/enclosures:
See next page

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