



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA ST., N.W., SUITE 3100
ATLANTA, GEORGIA 30303

Report No. 50-416/82-54

Licensee: Mississippi Power and Light Company
Jackson, MS 39205

Facility Name: Grand Gulf Nuclear Station

Docket No. 50-416

License No. NPF-13

Inspection at Grand Gulf site near Port Gibson, MS

Inspector: Frank Jape
for M. Thomas

8/6/82
Date Signed

Accompanying Personnel: F. Jape

Approved by: Frank Jape

F. Jape, Section Chief
Engineering Inspection Branch
Division of Engineering and Technical Programs

8/6/82
Date Signed

SUMMARY

Inspection on July 20-23, 1982

Areas Inspected

This routine, unannounced inspection involved 32 inspector-hours on site in the areas of initial fuel loading activities and followup on a NRR question concerning RCIC testing.

Results

Of the two areas inspected, no violations or deviations were identified.

REPORT DETAILS

1. Persons Contacted

Licensee Employees

- *C. K. McCoy, Plant Manager
- *R. A. Ambrosino, Assistant Plant Manager
- *L. R. Miller, Operations Assistant
- *S. F. Tanner, QA Supervisor
- J. C. Roberts, Startup Supervisor
- G. A. Zinke, Technical Engineering Supervisor

Other licensee employees contacted included health physics and startup test personnel, technicians, operators, security force members, and office personnel.

Other Organizations

M. G. Farschon, G. E. Site Operations Manager

*Attended exit interview

2. Exit Interview

The inspection scope and findings were summarized on July 23, 1982, with those persons indicated in paragraph 1 above. The licensee acknowledged the inspector's findings with no additional questions or comments.

3. Licensee Action on Previous Inspection Findings

Not inspected.

4. Unresolved Items

Unresolved items were not identified during this inspection.

5. Independent Inspection Effort (92706)

The inspector toured Unit 1 reactor building, auxiliary building, control building, and control room to verify that activities were being performed in accordance with NRC requirements.

No violations or deviations were identified.

6. Initial Fuel Loading Activities (72524)

The inspector observed fuel loading activities in the control room, auxiliary building, and on the containment refueling floor, and reviewed procedures pertaining to fuel loading to verify that activities were being

accomplished in accordance with Technical Specifications, licensee approved procedures, and NRC requirements. The inspector reviewed the fuel loading procedure and accompanying data to verify the following:

- a. Direct communication was established between the control room and the containment refueling floor.
- b. Instruments monitoring fuel loading were properly calibrated and operating with a measurable count rate.
- c. Staffing requirements were in accordance with Technical Specifications.
- d. Proper revision of the procedure was in use.
- e. Control of personnel access to the refueling floor.
- f. Changes to the procedure were made in accordance with administrative procedures.
- g. Data sheet entries for legibility and traceability.

In addition, the inspector reviewed test exceptions, control room logs, and test supervisor logs for records of deficiencies and difficulties encountered during fuel loading.

The inspector observed fuel handling crew performance for compliance with procedures during movement of fuel from the auxiliary building to the upper containment storage, from upper containment into the reactor vessel, and movement of the Fuel Loading Chambers (used to monitor fuel loading) about the core.

The inspector observed movement of the following fuel bundles:

LJS268	LJS453	LJS470
LJS324	LJS601	LJS611
LJS388	LJS461	LJS223
LJS396	LJN557	LJS215
LJS316	LJS471	LJS436
LJP941	LJS610	LJS380

Fuel loading was temporarily suspended on July 22, 1982, when fuel bundle LJS509 would not go into its designated core location (41-60) which was located next to the local power range monitor (LPRM) in core location (42-59). After closer examination, licensee personnel concluded that the LPRM was slightly bent, thus restricting insertion of the fuel bundle, and the LPRM would have to be removed from the core and replaced. Licensee

personnel stated that several LPRM's had been bent and required replacement thus far during fuel loading. The inspector observed crew activities during preparations to remove the LPRM and reviewed control room logs to verify documentation of the problem.

No violations or deviations were identified.

7. Followup on NRR Question Concerning Reactor Core Isolation Cooling (RCIC) System Testing (92704)

During the inspection, the inspector reviewed correspondence between MP&L and NRC Division of Licensing regarding RCIC testing. In a letter to MP&L dated February 19, 1982, NRC Licensing Branch No. 2 stated that certain tests are within the scope of preoperational testing described in Regulatory Guide 1.68, Appendix A.1.d. The tests included the following:

- a. Startup of the RCIC system after a loss of AC power to the system.
- b. Operation of the RCIC system with a sustained loss of AC power to the system.
- c. RCIC operation to demonstrate DC separation.

As discussed in a letter from MP&L to NRC Licensing Branch No.2 (letter AECM-82/107, April 2, 1982) and also discussed with the inspector during this inspection, licensee personnel stated that Item c. above satisfies the requirements of Regulatory Guide 1.68, Appendix A.1.d, which requires demonstration of electrical independence for residual and decay heat removal systems. This has been verified by Region II inspectors during the review of test results for preoperational test 1E71-PT01, Integrated ECCS and Loss of Offsite Power Test. Licensee personnel also stated that items a. and b. will be satisfied during performance of the special test, Simulated Loss of Onsite and Offsite AC Power Test, as described in MP&L letter AECM-81/281, dated August 18, 1981. Prior to performance of the special test, MP&L will provide a safety analysis and a copy of the test procedure for NRC review and approval. The special test is also discussed in Grand Gulf's operating license under License Condition C. (44(b)).