

CONTROL SYSTEMS FAILURES
EVALUATION REPORT

AUGUST 1982

PREPARED

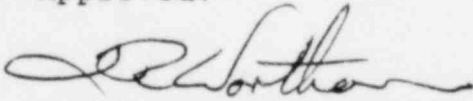
FOR

LONG ISLAND LIGHTING COMPANY
SHOREHAM NUCLEAR POWER STATION

PREPARED
BY

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CONTROL SYSTEMS FAILURES
EVALUATION REPORT
FOR THE SHOREHAM NUCLEAR POWER STATION

1.0 OBJECT

This document constitutes:

- An analysis in response to the NRC concern that the failures of power sources or sensors which provide power or electrical signals to multiple control systems could result in consequences outside the bounds of the Shoreham Final Safety Analysis Report (FSAR) Chapter 15 analyses and beyond the capability of operators or safety systems.
- A positive demonstration that adequate review and analysis has been performed to ensure that despite such failure the Shoreham FSAR Chapter 15 analyses are bounding, and no consequence beyond the capability of operators on safety systems would result.

A comprehensive approach was developed to analyze the control systems capable of affecting reactor water level, pressure or power in the Shoreham plant.

This report with its attachments was prepared by the General Electric Company for the Long Island Lighting Company (LILCO) with a significant technical contribution from the Stone & Webster Engineering Corporation (SWEC).

2.0 CONCLUSIONS

This report, supplemented by the existing FSAR Chapter 15 transient analyses, documents an evaluation of the Shoreham Nuclear Power Station for system interaction by electrical means. The conclusion of this evaluation is that previously reported limits of minimum critical power ratio (MCPR), peak vessel and main steamline pressures, and peak fuel cladding temperature for the expected operational occurrence category of events would not be exceeded as a result of common power source or sensor failures. Although new transient category events have been postulated as a result of this study, the net effects have been positively determined to be less severe than those of the original, conservative, Chapter 15 events. It should be noted that this study uses the event - consequence logic of the Chapter 15 analysis, but starts the logic chain from a specific source (e.g., a single bus failure) rather than a system condition (e.g., feedwater runout). By approaching the study in this manner, a great deal of confidence can be placed in the study conclusions. The approach validated itself by uncovering previously unanalyzed interactions. The soundness of the total plant design is demonstrated by its being tolerant of these interactions.

3.0 ANALYSIS METHODOLOGY

The electrical control systems failure analysis was conducted in the following manner by GE and the SWEC:

<u>ACTIVITY</u>	<u>ASSIGNED TO</u>
• DEFINE BUS STRUCTURE	SWEC
• DEFINE CONTROL SYSTEMS	SWEC & GE
• IDENTIFY LOADS	SWEC & GE
• DETERMINE CRITICAL LOADS	SWEC & GE
• SUMMARIZE CRITICAL LOADS	GE
• ANALYZE COMBINED EFFECTS	GE
• COMPARE RESULTS TO CHAPTER 15	GE
• ANALYZE EXCEPTIONS	GE
• MODIFY/AUGMENT CHAPTER 15 IF NECESSARY	GE

3.1 DEFINE BUS STRUCTURE

This step established the potential sources for system interaction by electrical means. Bus trees (see Figures 1 and 2) were constructed using one-line diagram information to show power distribution from the highest level not previously analyzed (the highest level previously analyzed is the loss of offsite power) down to the lowest level of plant distribution (Motor Control Center's, instrument busses, etc.).

3.2 DEFINE CONTROL SYSTEMS

This step established the scope of control systems to be analyzed. A complete list of Shoreham plant systems and subsystems was compiled. This list was then reviewed to confine the analysis to only those systems with the potential to affect reactor pressure, water level, or power.

To ensure that all necessary systems were considered, certain elimination criteria were established that documented the justification for not analyzing that system further. If there was any uncertainty as to whether or not a system met the criteria, it was retained for further analysis. Those systems that met the criteria for elimination were removed from the complete system list to produce the final list of control systems for analysis. This final list, reviewed by GE and SWEC, is shown as follows:

3.2 DEFINE CONTROL SYSTEMS (Continued)

<u>MPL</u>	<u>SYSTEMS</u>	<u>MPL</u>	<u>SYSTEMS</u>
B21	Nuclear Boiler System	N42	Hydrogen Seal System
B31	Reactor Recirculation	N43	Generator Cooling
C11	CRD Hydraulic Control System	N44	Air Removal
C32	Feedwater Turbine	N45	Generator Hydrogen & CO2 Purge
C51	Neutron Monitoring	N51	Main Generator Excitation
D11	Process Radiation Monitor System	N62	Off Gas
D21	Area Radiation Monitor System	N71	Circulating Water
G33	Reactor Water Cleanup	P41	Service Water
N11	Main Steam	P42	RB Closed Cooling Water System
N21	Condensate	P43	TB Closed Cooling Water System
N32	Turbine Control	P50	Compressed Air
N34	Lube Oil	P71	Low Conductivity Drains
N35	Moisture Extraction	Z93	Primary Containment Instrumentation

3.3 IDENTIFY LOADS

This step provided the data base necessary to determine which electrical loads were to be analyzed. A set of load tables comprised of all electrical loads of the control systems in Paragraph 3.2 was assembled by GE and SWEC, each providing information on the loads within their respective scope of supply.

Each load was listed with its power bus source, its unique Master Parts List number, circuit description, and failure mode on power loss with primary and secondary effects. A sample of a load table is included in Appendix C.

3.4 DETERMINE CRITICAL LOADS

This step constituted the first analytical step in sorting out the loads with the potential for initiating events affecting reactor pressure, water level and power. The elimination criteria established earlier for the system list was refined in Appendix B for use in the component review for determining which individual loads were worthy of further consideration or could be deleted from the analysis. If there was any uncertainty as to whether or not a load met the elimination criteria it was retained for further analysis. The code associated with an elimination criterion was assigned to each eliminated load in the load tables discussed in the previous step.

3.5 SUMMARIZE CRITICAL LOADS

The non-critical loads were deleted from the load tables, and the remaining loads are grouped together by their common power busses. These tables are shown in Appendix A.

3.6 ANALYZE COMBINED EFFECTS

This step provided the basis for determining the worst case combinations of load and system failures that are credible events considering the interconnection by power distribution. Using the combined effects at the lowest level bus as a starting point, the next higher bus was postulated to fail and the total effects at that level analyzed. This process was continued up to the highest bus level. The combined effects at the lowest bus level are included in the Appendix A tables. Worst case effects at the higher levels are summarized in Section 4. The combined effects at intermediate bus levels less severe than their associated higher bus combined effects were analyzed but not included in Section 4. The intermediate level combined effect analysis is already represented in the higher bus analysis.

3.7 COMPARE RESULTS TO CHAPTER 15

This step evaluated the consequences of all potential system interaction events initiated by electrical means. A review of the information in the Appendix A tables was conducted in the course of developing the bus summaries of Section 4. At each bus level of the combined effects analysis, the review evaluated the effects as being bounded by a specific Chapter 15 transient analysis or not. Section 4 includes these evaluations considering the worst case effects.

3.8 ANALYZE EXCEPTIONS

The purpose of this step was to determine if a failure scenario not directly covered by a Chapter 15 transient analysis would be bounded by one with more severe effects. The cases of this type are included in the Section 4 descriptions of worst case failures.

3.9 MODIFY/AUGMENT CHAPTER 15 IF NECESSARY

This step was not necessary in the Shoreham analysis.

4.0 BUS LOSS SUMMARY RESULTS AND CHAPTER 15 COMPARISONS

AC Bus

1A
(4.16KV) Loss of this bus causes the loss of power to condensate booster pump A and reactor recirculation pump A. There is also a potential main turbine trip due to the circulating water pump A loss and its subsequent effect on condenser vacuum. Since a reduction of reactor recirculation flow would immediately start reducing reactor power, an immediate or delayed turbine trip would produce an equal or less severe transient than the turbine trip event of Chapter 15. Therefore this event is bounded.

1B
(4.16KV) The effects of the loss of this bus are similar to those of the loss of Bus 1A.

11
(4.16KV) Loss of this bus will cause condensate pump A and circulating water pump C to be inoperative. The loss of the condensate pump will initiate reactor recirculation flow to run back and reduce reactor power corresponding to 67 percent of rated feedwater flow. In addition, a loss of feedwater heating of less than 10°F will occur, but this effect will be nullified by the recirculation runback. In the event that circulating water pump A or B is in the backwash operation, the loss of circulating water pump C may cause pump D to flow back and effectively reduce the circulating water flow to a one-pump operation; and the condenser back pressure may rise rapidly leading to a main turbine trip. The ensuing pressure excursion may even reach the bypass closure trip setpoint. However, this event will take place at reduced reactor power and it is bounded by the turbine trip without bypass transient already analyzed in FSAR Chapter 15.

Loss of the associated lower busses fed by Bus 11 will produce some or all of the following effects: Decrease in condenser vacuum, delayed main turbine trip, reduction in feedwater flow, and reduction in reactor recirculation flow.

The worst case reduction in feedwater temperature has been determined to be no more than 10°F. This reduction in feedwater heating will increase reactor power by less than three percent nuclear boiler rated (NBR) power.

The worst case scenario is the unlikely event of a loss of feedwater heating and a delayed turbine trip. A computer analysis was performed to determine the reactor parameters as a consequence of a turbine trip at approximately 103 percent of initial power. The results yielded a minimum critical power ratio (MCPR) of 1.10 and a maximum dome pressure of 1197 psia which is less severe than the most limiting transient analyzed in FSAR Chapter 15. This event is then, although previously not analyzed for the Shoreham plant, still bounded by existing analyses.

4.0 BUS LOSS SUMMARY RESULTS AND CHAPTER 15 COMPARISONS (Continued)

AC Bus

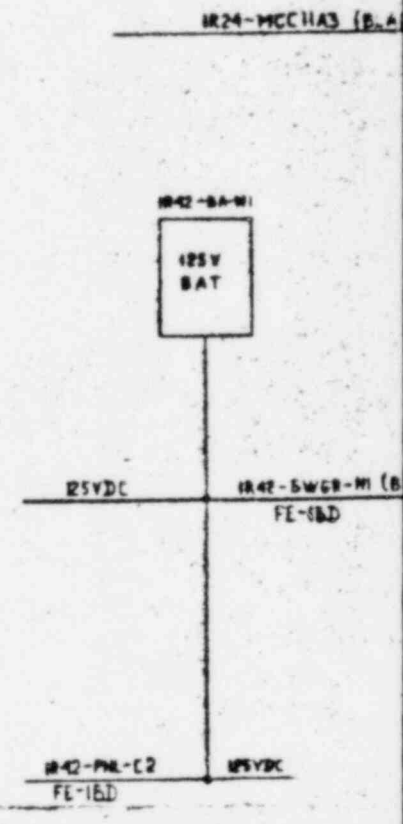
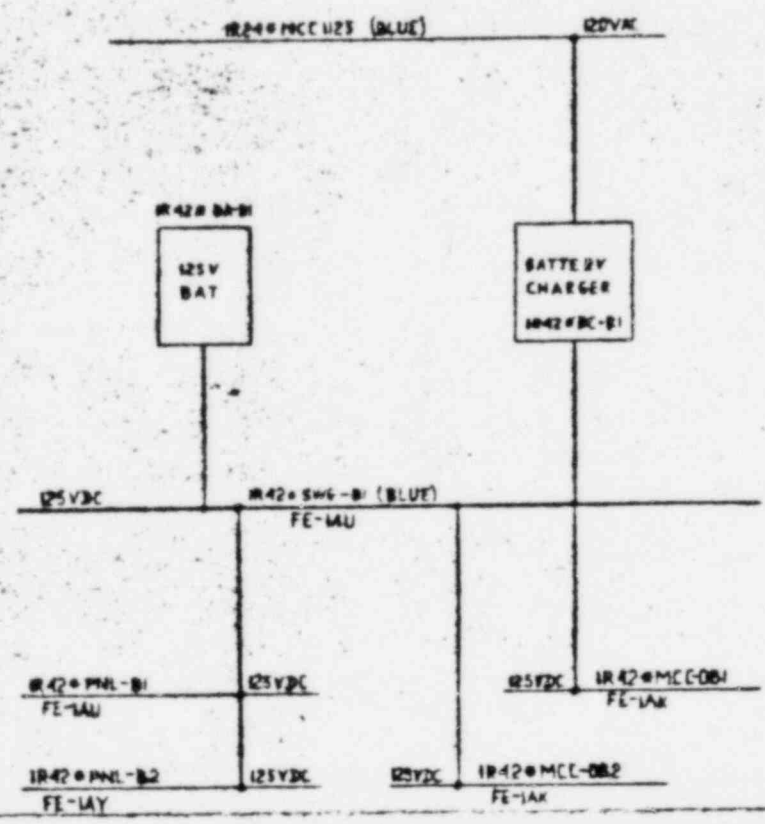
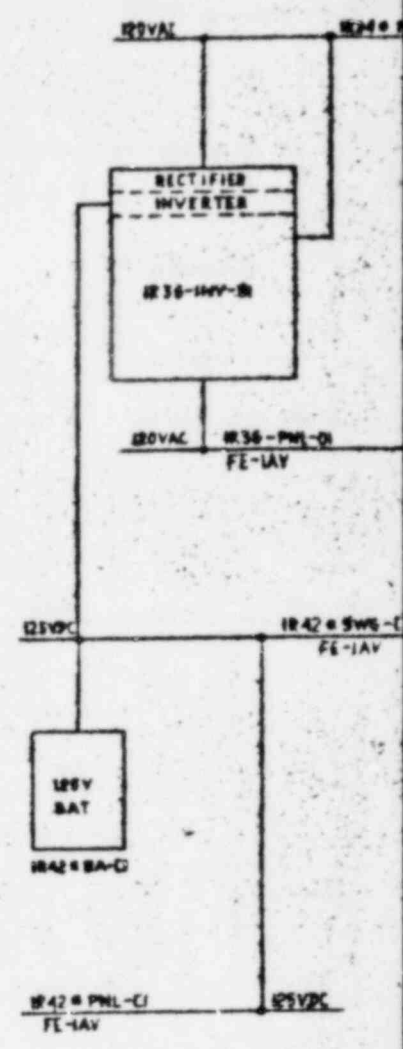
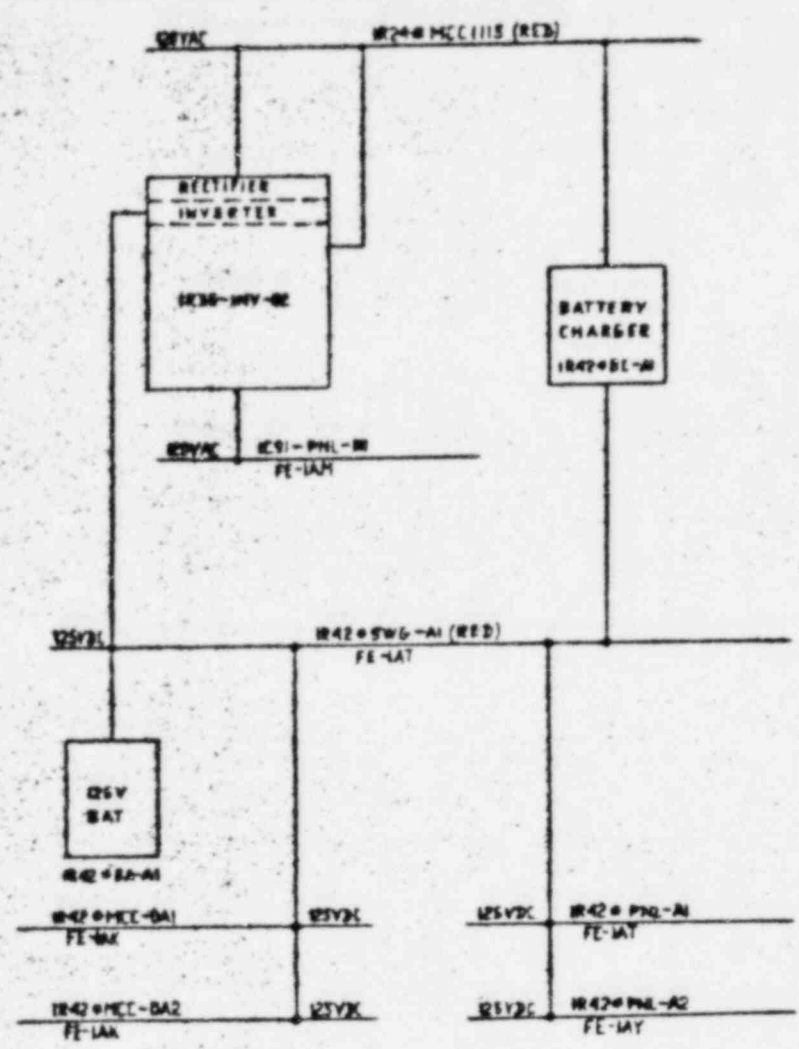
12 The effects of the loss of this bus are similar to those of the
(4.16KV) loss of Bus 11.

101/102 The loss of either of these busses will cause a single channel
(4.16KV) trip from the APRM circuitry to the reactor protection system
Emergency which produces no transient.

103 Loss of this bus will cause a decrease in reactor recirculation
(4.16KV) flow and a lock of the feedwater pumps at-last-speed setting. An
Emergency increase in level would ensue terminated by the level 8 feedwater
pump and main turbine trip. This event is similar to and bounded
by the feedwater runout event analyzed in Chapter 15.

DC Bus

1R42- The worst case effect of the loss of either of these battery
BA N1 buses is a main turbine trip with no additional complications
& N2 which is bounded by Chapter 15 load rejection analysis.



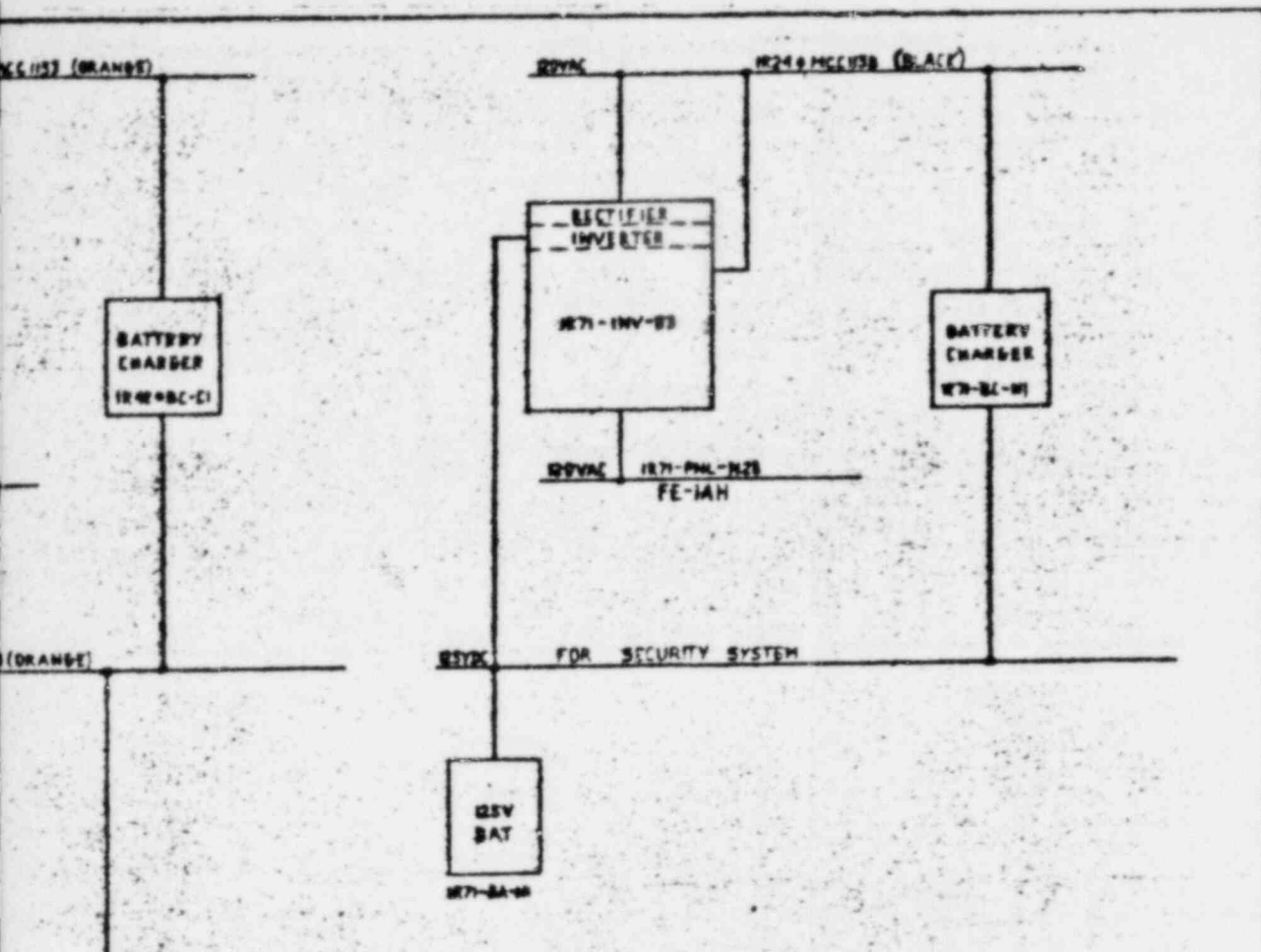
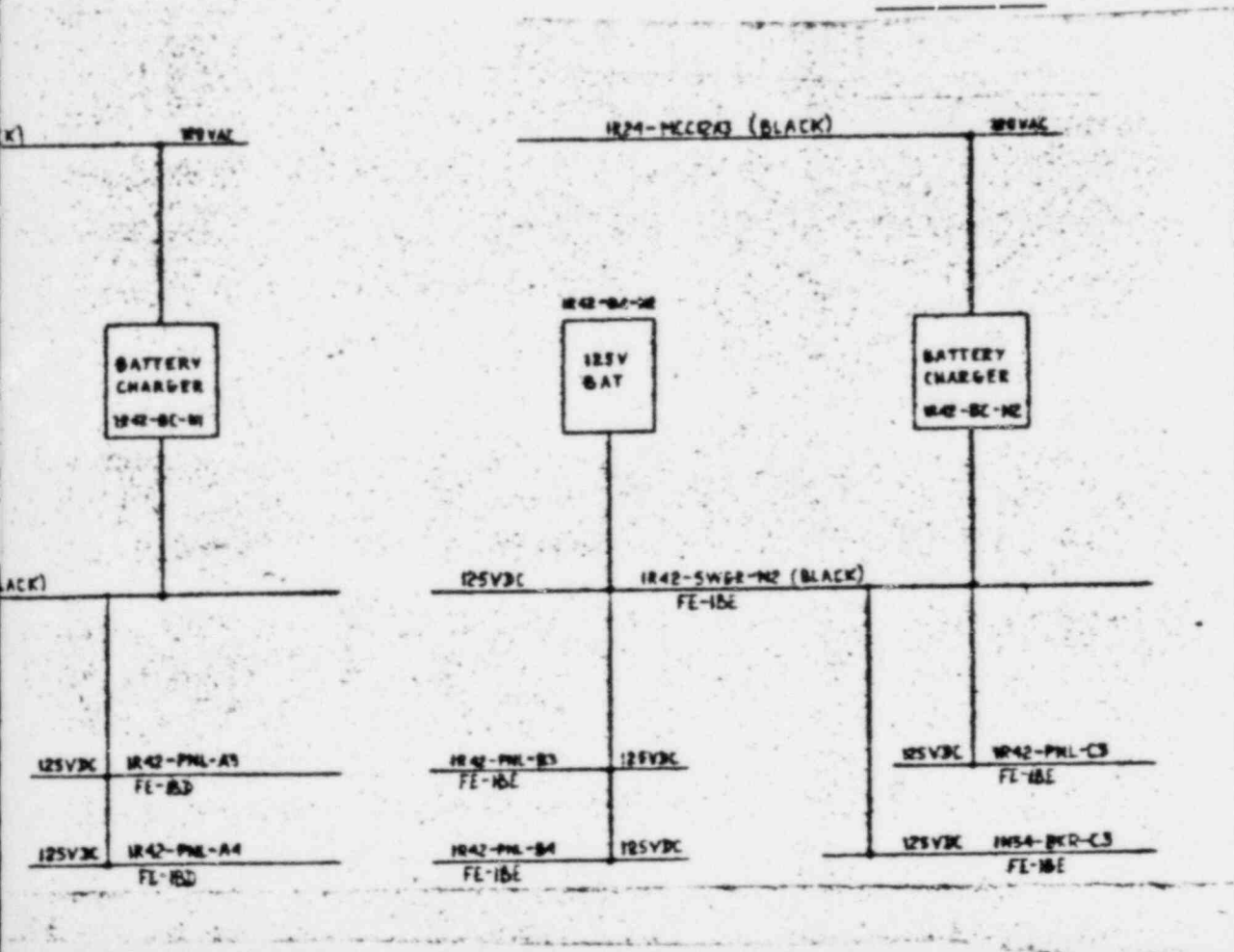


FIGURE 1 SHOREHAM CONTROL SYSTEMS FAILURES ANALYSIS
DC BUS TREE



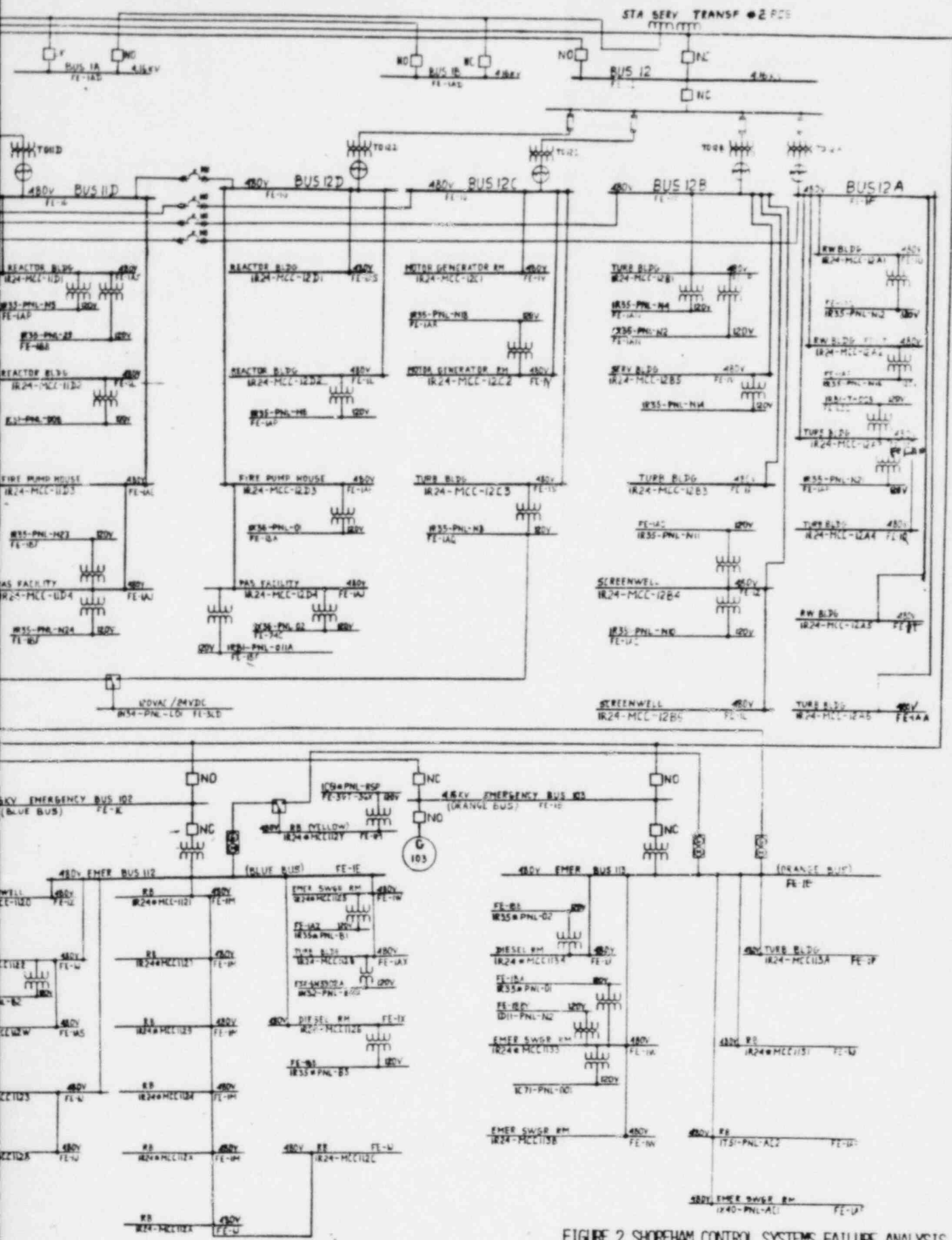


FIGURE 2 SHOREHAM CONTROL SYSTEMS FAILURE ANALYSIS

AC BUS TREE

APPENDIX A - BUS TABLES

SHOREHAM CONTROL SYSTEM FAILURE ANALYSIS

	SYSTEM	COMPONENT DESCRIPTION	PRIMARY EFFECT	SECONDARY EFFECT	COMBINED EFFECTS
AC BUS 1A	RECIRC	GENERATOR DRIVE MOTOR - S001A	LOSE GENERATOR DRIVE MOTOR - S001A	RUN BACK TO 65% POWER	REDUCTION OF FEEDWATER FLOW TO 67% OF RATED
	CIRC. WTR	CIRCULATING WATER PUMP A (PUMP B - BUS 18) (PUMP C - BUS 11) (PUMP D - BUS 12)	DECREASE CONDENSER VACUUM	SLIGHT DECREASE IN CONDENSER VACUUM	RECIRCULATION RUNBACK TO 65% REACTOR POWER SLIGHT DECREASE IN CONDENSER VACUUM
	CONDENSATE	CONDENSATE BOOSTER PUMP A (PUMP B - BUS 18)	REDUCTION OF FEEDWATER TO 67% OF RATED	RUNBACK TO 65% REACTOR POWER	

APPENDIX A - BUS TABLES

SHOREHAM CONTROL SYSTEM FAILURE ANALYSIS

	SYSTEM	COMPONENT DESCRIPTION	PRIMARY EFFECT	SECONDARY EFFECT	COMBINED EFFECTS
AC BUS 1B	RECIRC	GENERATOR DRIVE MOTOR - S001B	LOSE GENERATOR DRIVE MOTOR - S001B	RUN BACK TO 65% POWER	REDUCTION OF FEEDWATER FLOW TO 67% OF RATED
	CIRC. WTR	CIRCULATING WATER PUMP B (PUMP A - BUS 1A) (PUMP C - BUS 11) (PUMP D - BUS 12)	DECREASE CONDENSER VACUUM	SLIGHT DECREASE IN CONDENSER VACUUM	RECIRCULATION RUNBACK TO 65% REACTOR POWER
	CONDENSATE	CONDENSATE BOOSTER PUMP B (PUMP A - BUS 1A)	REDUCTION OF FEEDWATER FLOW TO 67% OF RATED	RUNBACK TO 65% REACTOR POWER	SLIGHT DECREASE IN CONDENSER VACUUM

APPENDIX A - BUS TABLES

AC
BUS 11

SHOREHAM CONTROL SYSTEM FAILURE ANALYSIS

	SYSTEM	COMPONENT DESCRIPTION	PRIMARY EFFECT	SECONDARY EFFECT	COMBINED EFFECTS	
BUS 11A	CONDENSATE	CONDENSATE PUMP A (PUMP B-BUS 12)	FEEDWATER REDUCED TO 67% OF RATED	REACTOR PRESSURE VESSEL WATER LEVEL LOWER AND AT 65% POWER	(SEE SECTION 4)	
	COMP. AIR	AIR COMPRESSOR A (COMPRESSOR B & C - BUS 12)	COMPRESSOR A INOPERATIVE	NONE - BACKED UP BY COMPRESSORS B & C		
	CIRC. WTR	CIRCULATION WATER PUMP C (PUMP A - BUS 1A) (PUMP B - BUS 1B) (PUMP D - BUS 12)	PUMP INOPERATIVE	DECREASE CONDENSER VACUUM. MAIN TURBINE TRIP.		
	SERVICE WATER TURBINE BUILDING	SERVICE WATER PUMP A (PUMPS B & C - BUS 12)	PUMP INOPERATIVE	NONE - BACKED UP BY PUMPS B & C		
	1R24 MCC11A1	COMP. AIR	WASTE NEUTRAL TANK INLET AIR MOV-81	IF OPEN, WILL LOSE INSTRUMENT AIR (COMPRESSORS B & C BUS 12 AND 1R35 PNL-N8)	NONE	NONE
	1R24 MCC11A2	OFFGAS	DRYER TRAIN A (TRAIN B - MCC12A2)	LOSS OF DRYER TRAIN A	NONE - BACKED UP BY DRYER TRAIN B	NONE
	1R24 MCC11A3	CONDENSATE	HEATER TRAIN BYPASS MOV-30	MOV FAILS AS IS	NONE	WORST CASE - DECREASE IN CONDENSER VACUUM
		GEN. COOLING	STATOR COOLING WATER PUMP A (PUMP B - BUS 12)	IF PUMP B NOT AVAILABLE, MAIN TURBINE TRIP	NONE - BACKED UP BY PUMP B	
		AIR REMOVAL	AIR EJECTOR ISOLATION MOV-45A AIR EJECTOR ISOLATION MOV-46A	MOV'S FAIL AS IS	IF IN BACKWASH, SLIGHT DECREASE IN CONDENSER VACUUM	
	CIRC. WATER	CONDENSER INLET MOV-32A CONDENSER DISCHARGE MOV-33A CONDENSER BACKWASH VALVE MOV-34A	IF IN BACKWASH, REDUCE FLOW TO 2 QUADRANTS IF IN BACKWASH, REDUCE FLOW TO 2 QUADRANTS	DECREASE CONDENSER VACUUM DECREASE CONDENSER VACUUM		

APPENDIX A - BUS TABLES

AC
BUS 11

SHOREHAM CONTROL SYSTEM FAILURE ANALYSIS

	SYSTEM	COMPONENT DESCRIPTION	PRIMARY EFFECT	SECONDARY EFFECT	COMBINED EFFECTS
BUS 11A 1R24 MCC11A4 1R35-PNL N7	COMP. AIR	COMPRESSOR START UP AUXILIARY OIL PUMP A	AUX OIL PUMP INOPERATIVE	UNABLE TO START AIR COMPRESSOR A	DECREASE CONDENSER VACUUM AND FEEDWATER TEMPERATURE
	OFFGAS	HOT GAS BYPASS LIQUID FREON SOV-49A SOV-31A	SOV'S DE ENERGIZED	NONE - BACKED UP BY TRAIN B (TRAIN B - BUS 12A)	
	CONDENSATE	REACTOR FEEDWATER PUMP A DISCHARGE SOV-42A	SOV DE-ENERGIZED	SLIGHT INCREASE IN REACTOR FEEDWATER PUMP TURBINE SPEED	DECREASE CONDENSER VACUUM AND FEEDWATER TEMPERATURE
	MOISTURE EXTRACTION	GLAND STEAM EVAPORATOR DRAIN SOV-10L	SOV-10L VALVE CLOSES	BYPASS HEATER STEAM TO CONDENSER	DECREASE IN FEEDWATER TEMPERATURE AND CONDENSER VACUUM
		STEAM SEAL EVAPORATOR DRAIN SOV-10H	SOV-10H VALVE OPENS		
RADWASTE STEAM GENERATOR DRAIN SOV-11L		SOV-11L VALVE CLOSES			
RADWASTE STEAM GENERATOR DRAIN SOV-11H		SOV-11H VALVE OPENS			
1ST POINT HEATER SOV-01AH		SOV-01 AH VALVE OPENS	DECREASE FEEDWATER TEMPERATURE AND CONDENSER VACUUM		
1ST POINT HEATER SOV-01AN		SOV-01 AN VALVE CLOSES			
2ND POINT HEATER SOV-02AH		SOV-02 AH VALVE OPENS			
2ND POINT HEATER SOV-02AN		SOV-02 AN VALVE CLOSES			
3RD POINT HEATER SOV-03AH		SOV-03 AH VALVE OPENS			
3RD POINT HEATER SOV-03AN		SOV-03 AN VALVE CLOSES			
4TH POINT HEATER SOV-04AH	SOV-04 AH VALVE OPENS				
4TH POINT HEATER SOV-04AN	SOV-04 AN VALVE CLOSES				
5TH POINT HEATER SOV-05AH	SOV-05 AH VALVE OPENS				
COMP. AIR	AIR COMPRESSOR CONTROL CIRCUIT A (CIRCUITS B & C- 1R35-PNL N8)	LOSS OF INSTRUMENT AIR COMPRESSOR A	NONE - BACKED UP BY AIR COMPRESSORS B & C		

APPENDIX A - BUS TABLES

AC
BUS 11B

SHOREHAM CONTROL SYSTEM FAILURE ANALYSIS

	SYSTEM	COMPONENT DESCRIPTION	PRIMARY EFFECT	SECONDARY EFFECT	COMBINED EFFECTS
1R24 MCC11B1	CIRC. WATER	CIRCULATION WATER CONDENSER INLET MOV-32C CIRCULATION WATER CONDENSER DISCHARGE MOV-33D CONDENSER BACKWASH VALVE MOV-34C	IF IN BACKWASH, REDUCE FLOW TO 2 QUADRANTS IF IN BACKWASH, REDUCE FLOW TO 2 QUADRANTS IF IN BACKWASH, REDUCE FLOW TO 2 QUADRANTS	DECREASE CONDENSER VACUUM DECREASE CONDENSER VACUUM DECREASE CONDENSER VACUUM	WORST CASE - DECREASE CONDENSER VACUUM. RFT A TRIP. FEEDWATER FLOW REDUCTION TO 67% OF RATED. MINIMUM SPEED ON RECIRC A & B PUMPS - 50% REACTOR POWER
	MAIN TURBINE CONTROL	MAIN TURBINE EHC FLUID PUMP A (PUMP B - MCC12B1)	PUMP A INOPERATIVE	NONE - BACKED UP BY PUMP B	
	FEEDWATER	CONTROL SIGNAL FAIL INITIATING CONTACT	LOSS OF REACTOR FEEDWATER PUMP'S CONTROL SIGNAL WILL NOT SET AT LAST SPEED	SCRAM IF REACTOR FEEDWATER PUMP'S CONTROL SIGNAL LOST	MINIMUM SPEED ON RECIRC A & B PUMPS - 50% POWER
1R35 PNL N1	RECIRC	RECIRCULATION DIVISION 1 SPEED CONTROL	RECIRCULATION PUMPS A & B MINIMUM SPEED IF IN AUTO MODE	REACTOR AT 50% POWER	
1R35 PNL N3	CONDENSATE	MINIMUM FLOW BYPASS SOV-28A	SOV DE-ENERGIZED FCV FAILS OPEN	RFP TURBINE A TRIP. FEEDWATER FLOW REDUCTION TO 67% OF RATED. RECIRC RUNBACK TO 65% POWER	RFP TURBINE A TRIP. FEEDWATER FLOW REDUCTION TO 67% OF RATED. RECIRC RUNBACK TO 65% POWER
1R24 MCC11B4	CIRC. WATER	CIRCULATION WATER PUMP A DISCHARGE MOV-31A CIRCULATION WATER PUMP C DISCHARGE MOV-31C	MOV'S FAIL AS IS MOV'S FAIL AS IS	FAILED CLOSED - UNABLE TO START PUMP(S) FAILED OPEN - NO EFFECT ON PUMP(S)	LOSS OF TURBINE BUILDING SERVICE WATER STRAINER BACKWASH CAPABILITY IF CIRC WATER DISCHARGE VALVES FAIL OPEN AND PUMPS STOP, UNABLE TO PREVENT BACK FLOW, DECREASE CONDENSER VACUUM - MAIN TURBINE TRIP
1R24 MCC11B6	SERVICE WATER	TURBINE BUILDING SERVICE WATER INLET MOV-113A	MOV FAILS AS IS - NORMALLY CLOSED. LOSS OF STRAINER BACKWASH CAPABILITY	WORST CASE - MAIN TURBINE AND RFP TURBINE TRIP AFTER MANY HOURS	WORST CASE - MAIN TURBINE AND RFP TURBINE TRIP AFTER MANY HOURS

APPENDIX A - BUS TABLES

AC
BUS 12

SHOREHAM CONTROL SYSTEM FAILURE ANALYSIS

	SYSTEM	COMPONENT DESCRIPTION	PRIMARY EFFECT	SECONDARY EFFECT	COMBINED EFFECTS
	CONDENSATE	CONDENSATE PUMP B (PUMP A - BUS 11)	FEEDWATER REDUCED TO 67% OF RATED	REACTOR PRESSURE VESSEL WATER LEVEL LOWER AND AT 65% POWER	(SEE SECTION 4)
	COMP. AIR	INSTRUMENT AIR COMPRESSOR B INSTRUMENT AIR COMPRESSOR C (COMPRESSOR A ON BUS 11)	AIR COMPRESSORS B & C INOPERATIVE	NONE - BACKED UP BY COMPRESSOR A	
	CIRC. WATER	CIRCULATION WATER PUMP D	PUMP INOPERATIVE	DECREASE CONDENSER VACUUM. MAIN TURBINE TRIP	
	SERVICE WATER TURBINE BUILDING	SERVICE WATER PUMP B SERVICE WATER PUMP C	PUMPS INOPERATIVE	REDUCED TURBINE COOLING WATER. MAIN TURBINE TRIP AFTER SOME TIME	
BUS 12A 1R24 MCC12A2	OFFGAS	DRYER TRAIN B (TRAIN A - MCC11A2)	LOSS OF DRYER TRAIN B	NONE - BACKED UP BY TRAIN A	NONE
1R24 MCC12A3	GEN. COOLING	STATOR COOLING WATER PUMP B (PUMP A - BUS 11)	PUMP INOPERATIVE IF PUMP A NOT AVAILABLE MAIN TURBINE TRIP	NONE - BACKED UP BY PUMP A	WORST CASE - DECREASE CONDENSER VACUUM
	CIRC. WATER	CIRCULATION WATER CONDENSATE INLET MOV-32B CIRCULATION WATER CONDENSATE DISCHARGE MOV-33B CONDENSER BACKWASH VALVE MOV-34B	IF IN BACKWASH, REDUCE FLOW TO 2 QUADRANTS IF IN BACKWASH, REDUCE FLOW TO 2 QUADRANTS IF IN BACKWASH, REDUCE FLOW TO 2 QUADRANTS	DECREASE IN CONDENSER VACUUM DECREASE IN CONDENSER VACUUM DECREASE IN CONDENSER VACUUM	
1R24 MCC12A4	COMP. AIR	COMPRESSOR AUXILIARY OIL PUMP B COMPRESSOR AUXILIARY OIL PUMP C	PUMPS INOPERATIVE	COMPRESSORS B & C WILL NOT START IF DEMAND ON AIR SYSTEM REQUIRES	WORST CASE - DECREASE CONDENSER VACUUM
	AIR REMOVAL	AIR EJECTOR ISOLATION MOV-45B AIR EJECTOR ISOLATION MOV-46B	MOV'S FAIL AS IS	IF IN BACKWASH, SLIGHT DECREASE IN CONDENSER	
	OFFGAS	HOT GAS BYPASS SOV-49B LIQUID FREON SOV-31B	SOV'S DE-ENERGIZE	NONE - BACKED UP BY TRAIN A (TRAIN A - BUS 11A)	

APPENDIX A – BUS TABLES

AC
BUS 12

SHOREHAM CONTROL SYSTEM FAILURE ANALYSIS

	SYSTEM	COMPONENT DESCRIPTION	PRIMARY EFFECT	SECONDARY EFFECT	COMBINED EFFECTS
BUS 12B	MAIN TURBINE CONTROL	MAIN TURBINE EHC FLUID PUMP B (PUMP A – MCC11B1)	PUMP INOPERATIVE	NONE – BACKED UP BY PUMP A	WORST CASE – DECREASE IN CONDENSER VACUUM RFP TURBINE TRIP. FEED-WATER FLOW REDUCTION TO 67% OF RATED, MINIMUM SPEED ON RECIRC A & B PUMPS – 50% REACTOR POWER
1R24 MCC12B1	CIRC. WATER	CONDENSER BACKWASH VALVE MOV-34D CIRCULATION WATER CONDENSATE INLET MOV-32D CIRCULATION WATER CONDENSATE DISCHARGE MOV-33C	IF IN BACKWASH, REDUCE FLOW TO 2 QUADRANTS IF IN BACKWASH, REDUCE FLOW TO 2 QUADRANTS IF IN BACKWASH, REDUCE FLOW TO 2 QUADRANTS	DECREASE CONDENSER VACUUM DECREASE CONDENSER VACUUM DECREASE CONDENSER VACUUM	
1R35 PNL N2	RECIRC	RECIRCULATION DIVISION II SPEED CONTROL	RECIRCULATION A & B MINIMUM SPEED IF IN AUTO MODE	RUN BACK TO 50% POWER	
1R35 PNL N4	CONDENSATE	MINIMUM FLOW BYPASS SOV-28B	SOV DE-ENERGIZED FCV FAILS OPEN	RFP TURBINE TRIP. FEED-WATER FLOW REDUCTION TO 67% OF RATED. RECIRC RUNBACK TO 65% REACTOR POWER	RFP TURBINE TRIP. FEED-WATER FLOW REDUCTION TO 67% OF RATED. RECIRC RUNBACK TO 65% REACTOR POWER
1R24 MCC12B4	CIRC. WATER	CIRCULATION WATER PUMP DISCHARGE MOV-31B CIRCULATION WATER PUMP DISCHARGE MOV-31D	MOV'S FAIL AS IS MOV'S FAIL AS IS	FAILED CLOSED – UNABLE TO START PUMP(S). FAILED OPEN – NO EFFECT ON PUMP(S)	LOSS OF TURBINE BUILDING SERVICE WATER STRAINER BACKWASH CAPABILITY IF CIRC WATER DISCHARGE VALVES FAIL OPEN AND PUMPS STOP, UNABLE TO PREVENT BACK FLOW, DECREASE CONDENSER VACUUM – MAIN TURBINE TRIP
1R35 PNL N10	CIRC. WATER	STRAINER – S 81A STRAINER – S 81B	STRAINER MOTOR INOPERATIVE	UNABLE TO BACKWASH – MAIN TURBINE TRIP AFTER SEVERAL HOURS	MAIN TURBINE TRIP
1R35 PNL N11	CIRC. WATER	CIRCULATION WATER PUMPS CONTROL CIRCUIT INTERLOCK	CIRCULATION WATER PUMPS A, B, C & D CANNOT RE-START IF ANY SHOULD STOP	NONE	NONE

APPENDIX A - BUS TABLES

AC
BUS 12

SHOREHAM CONTROL SYSTEM FAILURE ANALYSIS

	SYSTEM	COMPONENT DESCRIPTION	PRIMARY EFFECT	SECONDARY EFFECT	COMBINED EFFECTS
1R24 MCC12B4 1R24 MCC12B6 - - - -	SERVICE WATER	TURBINE SERVICE WATER STRAINER INLET MOV-113E	MOV FAILS AS IS - NORMALLY CLOSED. LOSS OF STRAINER BACKWASH CAPABILITY	WORST CASE - MAIN TUR- BINE AND RFP TURBINE TRIP AFTER MANY HOURS	WORST CASE - MAIN TUR- BINE AND RFP TURBINE TRIP AFTER MANY HOURS

APPENDIX A - BUS TABLES

AC
BUS 12

SHOREHAM CONTROL SYSTEM FAILURE ANALYSIS

SYSTEM	COMPONENT DESCRIPTION	PRIMARY EFFECT	SECONDARY EFFECT	COMBINED EFFECTS
BUS 12C 1R24 MCC12C3 1R35 PNL NB	COMP. AIR AIR COMPRESSOR CONTROL CIRCUIT PUMP B AIR COMPRESSOR CONTROL CIRCUIT PUMP C (CIRCUIT A - 1R35 PNL N7)	LOSE INSTRUMENT COMPRES- SORS B & C	NONE - BACKED UP BY COMPRESSOR A	DECREASE CONDENSER VACUUM AND FEEDWATER TEMPERATURE
	STEAM SYSTEM STEAM SUPPLY REACTOR FEED- WATER PUMP TURBINE SOV 30A STEAM SUPPLY REACTOR FEED- WATER PUMP TURBINE SOV 30B	VALVES INOPERABLE	NONE - STEAM FLOW TO RFPT'S AT NORMAL OPERATION UNAFFECTED	
	CONDENSATE OPERATES FEEDWATER DIS- CHARGE VALVE NRV-42B SOV 42B	SOV DE-ENERGIZED	SLIGHT INCREASE IN REACTOR FEEDWATER PUMP TURBINE SPEED	
	MOISTURE EXTRACTION 1ST STAGE DRAIN SOV 7AH TANK DRAINS TO SOV 7AL CONDENSER SOV 7BH SOV 7BL 2ND STAGE DRAIN TANK SOV 8AH DRAINS TO CONDENSER SOV 8AL SOV 8BH SOV 8BL MOISTURE SEPARATOR SOV 9AH & REHEATERS DRAIN SOV 9AL TANK DRAINS TO SOV 9BH CONDENSER SOV 9BL 1ST POINT HEATER DRAIN SOV 1BH 1ST POINT HEATER DRAIN SOV 1BN 2ND POINT HEATER DRAIN SOV 2BH 2ND POINT HEATER DRAIN SOV 2BN 3RD POINT HEATER DRAIN SOV 3BH 3RD POINT HEATER DRAIN SOV 3BN	DUMPS STEAM TO CONDENSER BYPASS HEATER STEAM TO CONDENSER	DECREASE CONDENSER VACUUM DECREASE CONDENSER VACUUM DECREASE FEEDWATER TEMPERATURE AND CONDENSER VACUUM	

APPENDIX A – BUS TABLES

AC
BUS 12

SHOREHAM CONTROL SYSTEM FAILURE ANALYSIS

	SYSTEM	COMPONENT DESCRIPTION	PRIMARY EFFECT	SECONDARY EFFECT	COMBINED EFFECTS
BUS 12C 1R24 MCC12C3 1R35 PNL NB -----	MOISTURE EXTRACTION	4TH POINT HEATER DRAIN SOV 4BH 4TH POINT HEATER DRAIN SOV 4BN 6TH POINT HEATER DRAIN SOV 5BH	BYPASS HEATER STEAM TO CONDENSER	DECREASE FEEDWATER TEMPERATURE AND CONDENSER VACUUM	

APPENDIX A - BUS TABLES

SHOREHAM CONTROL SYSTEM FAILURE ANALYSIS

AC
BUS 101

	SYSTEM	COMPONENT DESCRIPTION	PRIMARY EFFECT	SECONDARY EFFECT	COMBINED EFFECTS
BUS 111 1R24 MCC1115 (BACK UP TO MGS B)	NEUTRON MONITORING	AVERAGE POWER RANGE MONITOR CHANNELS A, C, E ROD BLOCK MONITOR CHANNEL A	HALF SCRAM TRIP	IF ALTERNATE CHANNELS ALSO TRIP, REACTOR SCRAM	½ AVERAGE POWER RANGE MONITOR SCRAM - DIV. I
AC BUS 102 BUS 112 1R24 MCC1125 (BACK UP TO MGS A)	NEUTRON MONITORING	AVERAGE POWER RANGE MONITOR CHANNELS B, D, F ROD BLOCK MONITOR CHANNEL B	HALF SCRAM TRIP	IF ALTERNATE CHANNELS ALSO TRIP, REACTOR SCRAM	½ AVERAGE POWER RANGE MONITOR SCRAM - DIV. II
AC BUS 103 BUS 113 1R24 MCC1133 1R36 INV-01 125 VDC FROM INVERTER 1R36 PNL-01 (NO BATTERY BACK UP)	FEEDWATER	REACTOR FEEDWATER PUMPS CONTROL SIGNAL CIRCUITRY	FEEDWATER PUMPS REMAIN AT LAST SPEED UNLESS 1R35 PNL-N1 IS LOST - THEN REACTOR FEEDWATER PUMPS RUN DOWN	LOAD FOLLOWING MIS- MATCH WILL CAUSE MAIN TURBINE TRIP	RECIRCULATION PUMPS A & B AT MINIMUM SPEED - 50% POWER
	REACTOR RECIRC	RECIRCULATION CONTROL SIGNAL CIRCUITRY	IF IN AUTO MODE, PUMPS WILL RUN BACK TO MINIMUM SPEED (APPROXIMATELY 50% POWER)	(SEE REACTOR FEEDWATER CONTROL CIRCUIT ABOVE)	

SHOREHAM CONTROL SYSTEM FAILURE ANALYSIS

	SYSTEM	COMPONENT DESCRIPTION	PRIMARY EFFECT	SECONDARY EFFECT	COMBINED EFFECTS
DC BUS 1R42 BA N1 125 VDC 1R42 PNL A4 1R42 PNL C2	RFPT CONTROL	REACTOR FEEDWATER PUMP TURBINE A PANEL 2 EHC (REACTOR FEEDWATER PUMP TURBINE B - PANEL 1R42 PNL B4)	CONTINUES IF AT SPEED	NONE, IF AT SPEED	MAIN TURBINE TRIP
	MAIN TURBINE CONTROL	MAIN TURBINE PANEL 1 EHC	MAIN TURBINE TRIP	MAIN TURBINE TRIP	
DC BUS 1R42 BA N2 125 VDC 1R42 PNL B4	FEEDWATER	HIGH LEVEL B TRIP CIRCUIT	2 OF 3 HIGH LEVEL TRIP INTACT, BUT B TRIPPED		NONE
	RFPT CONTROL	REACTOR FEEDWATER PUMP TURBINE B PANEL 3 EHC (REACTOR FEEDWATER PUMP TURBINE A - PANEL 1R42 PNL A4)	CONTINUES IF NOT AT SPEED	NONE, IF AT SPEED	

APPENDIX B
ELIMINATION CRITERIA

<u>Elimination Criterion *</u>	<u>Basis</u>
N1	Components whose failure effects are clearly bounded by a dominant failure effect on the same bus can be eliminated by inspection. An example would be the loss of several trips such as feedwater turbine overspeed trip on the same bus as the solenoid that controls all remote trips. The solenoid loss is clearly the dominant effect. Also in the case of identical components, only one of the components on that bus need be listed.
N2	Instrumentation with no direct or indirect controlling function or passive input (such as a permissive) into control logic. Instrumentation and other dedicated inputs to the process computer, as well as the computer itself, may be excluded. Operator actions as a result of indications are not considered control functions for the control systems failure analysis.
N3	Control systems and controlled components (pumps, valves) which have no direct or indirect interaction with reactor operation/parameters. Examples are communications, most unit heaters and controls, lighting controls, ventilation control systems for exterior buildings, machine shop equipment, refueling or maintenance equipment controls, etc.
N4	Control systems and controlled components (pumps, valves) that do interact or interface with reactor operating systems but which cannot affect the reactor parameters (water level, pressure or reactivity) either directly or indirectly. Examples are: some offgas components, area radiation monitors.
N5	Systems which are not used during normal power operation. For example, eliminate start-up, shutdown or refueling systems not used during normal operation.
N6	Some lube oil pumps are powered from AC busses but have a back-up pump powered from a DC source. Since a single electrical failure cannot disable the lube oil function these components can be eliminated from the analysis.
Y	Requires further analysis.

* In some cases more than one of these criteria may apply.

APPENDIX C
SAMPLE LOAD TABLE

Power Bus (NE Designation)	System (Functioning WPL #)	System Control and Instrumentation Loads on Bus	GENERAL EFFECTS				SPECIFIC EFFECTS OF BUS TO SYSTEM															
			Limitations on the system's capability to perform its principal function due to loss of bus	Effect on System Subsections	Effect on other Systems	Effects Inertia IG wired on REACTOR WATER LEVEL, PRESSURE or CTR	Injerts Required from other systems and effect of LOSS of Injert	Z	C	D	E											
												F	G	H	I							
BUS-4 (NE Designation) BUS-4 (NE Designation) BUS-4 (NE Designation) BUS-4 (NE Designation) BUS-4 (NE Designation) BUS-4 (NE Designation) BUS-4 (NE Designation) BUS-4 (NE Designation) BUS-4 (NE Designation) BUS-4 (NE Designation) BUS-4 (NE Designation) BUS-4 (NE Designation) BUS-4 (NE Designation) BUS-4 (NE Designation) BUS-4 (NE Designation) BUS-4 (NE Designation) BUS-4 (NE Designation)	BUS-4	BUS-4-1500Z	No loss of bus	None	None	None	None															
		BUS-4-1500B	Pump IN34-P-183 will be operable from BUS-4-15003																			
		BUS-4-1500A	IN34-TCY000A Falls Open	IN34-TCY000A	IN34-TCY000A Falls Open	IN34-TCY000A	IN34-TCY000A Falls Open	IN34-TCY000A	IN34-TCY000A Falls Open	IN34-TCY000A	IN34-TCY000A Falls Open	IN34-TCY000A	IN34-TCY000A Falls Open	IN34-TCY000A	IN34-TCY000A Falls Open	IN34-TCY000A	IN34-TCY000A Falls Open	IN34-TCY000A	IN34-TCY000A Falls Open			
		BUS-4-TC000B	IN34-TCY000B Falls Open	IN34-TCY000B	IN34-TCY000B Falls Open	IN34-TCY000B	IN34-TCY000B Falls Open	IN34-TCY000B	IN34-TCY000B Falls Open	IN34-TCY000B	IN34-TCY000B Falls Open	IN34-TCY000B	IN34-TCY000B Falls Open	IN34-TCY000B	IN34-TCY000B Falls Open	IN34-TCY000B	IN34-TCY000B Falls Open	IN34-TCY000B	IN34-TCY000B Falls Open			
		BUS-11 (NE Designation) BUS-11 (NE Designation) BUS-11 (NE Designation) BUS-11 (NE Designation) BUS-11 (NE Designation) BUS-11 (NE Designation) BUS-11 (NE Designation) BUS-11 (NE Designation) BUS-11 (NE Designation) BUS-11 (NE Designation)	BUS-11	BUS-11-PT001	Loss of Signal	None	None	None	None													
				BUS-11-PT003	Loss of Signal																	
				BUS-11-PT01A	Loss of Signal																	
				BUS-11-PT01B	Loss of Signal																	
				BUS-11-PT01A	Loss of Signal																	
				BUS-11-PT01B	Loss of Signal																	
				BUS-11-PT01A	Loss of Signal																	
				BUS-11-PT01B	Loss of Signal																	
				BUS-11-PT01A	Loss of Signal																	
				BUS-11-PT01B	Loss of Signal																	
		BUS-11 (NE Designation) BUS-11 (NE Designation) BUS-11 (NE Designation) BUS-11 (NE Designation) BUS-11 (NE Designation) BUS-11 (NE Designation) BUS-11 (NE Designation) BUS-11 (NE Designation) BUS-11 (NE Designation) BUS-11 (NE Designation)	BUS-11	BUS-11-PS01A	None																	
				BUS-11-PS01B	None																	
				BUS-11-PS01A	Loss of Signal																	
BUS-11-PT01A	Loss of Signal																					
BUS-11-PT01B	Loss of Signal																					
BUS-11-PT01A	Loss of Signal																					
BUS-11-PT01B	Loss of Signal																					
BUS-11-PS01A	None																					
BUS-11-PS01B	None																					
BUS-11-PS01A	Loss of Signal																					
BUS-12 (NE Designation) BUS-12 (NE Designation) BUS-12 (NE Designation) BUS-12 (NE Designation) BUS-12 (NE Designation) BUS-12 (NE Designation) BUS-12 (NE Designation) BUS-12 (NE Designation) BUS-12 (NE Designation) BUS-12 (NE Designation)	BUS-12	BUS-12-PS-009A	Loss of Signal																			
		BUS-12-PS-009B	Loss of Signal																			
		BUS-12-PT-005A																				
		BUS-12-PT-005B																				
		BUS-12-PT-007																				
		BUS-12-PT-008A																				
		BUS-12-PT-008B																				
		BUS-12-PT-009																				
		BUS-12-PT-009A																				
		BUS-12-PT-009B																				