LICENSEE EVENT REPORT

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CONTROL BLOCK: []] [] (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)
0 1 A L B R F 2 2 0 0 - 0 0 0 0 - 0 0 3 4 1 1 1 1 1 1 5 57 CAT 58
CON'T O 1 SOURCE L G O 5 O 0 0 2 6 O 7 0 7 3 0 8 2 8 0 8 2 6 8 2 0 O 1 O 1 O 1 O 1 O 1 O 1 O 1 O
During steady-state operation at 63-percent power, it was discovered that the
[0]3 computer heat balance calculated reactor thermal power non-conservatively due to
0 4 an erroneous feedwater flow signal (T.S. 2.1). There was no danger to the health
o 5 or safety of the public. Thermal limits and calculated scram points were not
0 6 exceeded. There are no redundant systems.
07
0 B 1 7 B 9
SYSTEM CODE CODE SUBCODE COMPONENT CODE SUBCODE SUBCOD
17 REPORT NUMBER 21 23 23 24 C 0 2 4 C 0 3 L C 0
ACTION FUTURE EFFECT SHUTDOWN HOURS (22) ATTACHMENT FORM SUB. PRIME COMPONENT MANUFACTURER SUBMITTED FORM SUB. SUPPLIER MANUFACTURER MANUFACTURER (25) D 2 3 12
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)
[1]0] LCause was an instrument sensing line root valve leak. Valve was a Dragon Model
[1]1 500-F. APRMs were recalibrated using manual heat balance. Valve leak will be
[1]2 repaired during refuel outage. This is a random occurrence and no recurrence
[1] Control is necessary. Procedures will be revised to ensure early detection of
[1]4 Lsimilar problems.
FACILITY STATUS STATUS OTHER STATUS OF DISCOVERY DESCRIPTION (32) NA A A STATUS DISCOVERY DESCRIPTION (32) NA B A STATUS DISCOVERY DESCRIPTION (32) Engineer observation
ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY 35 1 6 Z 33 Z 34 NA NA NA
7 8 9 10 11 44 45 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (29)
1 7 0 0 0 0 37 Z 38 NA
NUMBER DESCRIPTION (41) NA
LOSS OF OR DAMAGE TO FACILITY (43)
1 9 Z 42 NA
PUBLICITY SSUED DESCRIPTION (45) PDR ADOCK 05000260 NRC USE ONLY
NAME OF PREPARER Earl Nave PHONE(205) 729-0845

LER SUPPLEMENTAL INFORMATION

BFRO-50- 260 / 82024 Technical Specification Involved 2.1

Reported Under Technical Specification 6.7.2.b.(1)* Date Due NRC 08/29/82

Event Narrative:

Unit 1 was operating at 80-percent power and unit 3 at 86-percent power. These units were unaffected by this event. While operating unit 2 at 63-percent power, it was determined that the computer heat balance (OD-3) was calculating an incorrect thermal power (T.S. 2.1.A). After recalibration of the feedwater flow rate instrumentation, the cause was determined to be a leaking root valve to the "A" feedwater line flow transmitter. The valve was a Dragon model 500-F needle valve. The correct thermal power was determined by manual heat balance. The total feedwater flow rate value for this calculation was obtained by doubling the "B" feedwater lines flow rate value. The power calculated by OD-3 was 1632 MWe, and 2209 by manual heat balance. Core thermal limits were evaluated using the offline computer method (CLIP) and the APRMs were recalibrated per SI 4.1.B-2 using the power level obtained from the manual heat balance. A review of thermal limit calculations and APRM calibration data for the period affected by this problem indicated that no thermal limits were exceeded as a result of this problem and that the correct scram setpoints were not exceeded. The unit was shutdown for refueling on July 30, 1982. The feedwater flow rate indication will be corrected prior to startup. The manual heat balance technical instructions (TI1) will be revised to monitor individual feedwater line flow rate and plant efficiency to provide early detection for similar problems. There was no danger to the health or safety of the public. Thermal limits and calculated scram points were not exceeded. There are no redundant systems.

* Previous Similar Events:

NONE

Retention: Period - Lifetime, Responsibility - Document Control Supervisor

*Revision: