

JUL 22 1982

Certified By Cardyn J. Wilson

MEMORANDUM FOR: Gus C. Lainas, Assistant Director
for Safety Assessment
Division of Licensing

FROM: Thomas A. Ippolito, Chief
Operating Reactors Assessment Branch
Division of Licensing

SUBJECT: FORTHCOMING MEETING WITH STEAM GENERATOR OWNERS
GROUP - PROPOSED STEAM GENERATOR GENERIC
REQUIREMENTS

Date & Time: July 29, 1982
10:30 a.m. - 5:00 p.m.

Location: Phillips Building, P-118
Bethesda, MD

Purpose: To discuss the value/impact assessment of the draft
NRC report "NRC Requirements Concerning Steam
Generator Tube Degradation and Rupture Events
(Including resolution of USI's A-3, A-4, and A-5)"
in accordance with the attached agenda.

Participants: NRC
D. Eisenhut, R. Mattson, S. Hanauer, R. Vollmer,
R. Baer, R. Ramirez, G. Lainas, T. Ippolito,
T. Marsh, J. Strosnider, J. Moorehouse(SAI).

SGOG
A. Schmidt, L. White, P. Santoro, R. Acosta,
R. McCredy, B. Snow, A. Curtis

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EPRI
S. Green

151

Thomas A. Ippolito, Chief
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RD-7-2B

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Attachment:
As Stated

OFFICE	DL:ORAB	DL:ORAB	DL:ORAB				
SURNAME	RMartin:cw	Tippolito	DEisenhut				
DATE	7/22/82	7/22/82	7/ /82				

MEETING NOTICE DISTRIBUTION

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Central Files
NRC PDR
Local PDR
ORAB Reading
NSIC
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OI&E
ACRS (10)
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R. Romirez
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W. Johnston
E. Jordan
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K. Wichman
R. Martin
E. Murphy
V. Benaroya
P. Norian
W. Mills
Receptionist (Phillips Bldg.)
B. Scharf

AGENDA

- I. Introduction
- II. Proposed Licensee Requirements
 1. Steam Generator Integrity
 - Prevention and Detection of Loose Parts or Foreign Objects
 - Stabilization and Monitoring of Degraded Tubes
 - Inservice Inspection Program
 - Improved Eddy Current Techniques
 - Primary to Secondary Leakage
 - Secondary Water Chemistry Program
 - Condenser Inservice Inspection Program
 - Upper Inspection Ports
 2. Plant Systems Response
 - Reactor Coolant System Pressure Control During A SGTR
 - Safety Injection Signal Reset
 - Containment Isolation and Reset
 3. Human Factors Consideration
None
 4. Radiological Consequences
 - Standard Technical Specification Limit for Coolant Iodine Activity
 5. Organizational Response
None
- III. NRC Proposed Actions
 1. Steam Generator Integrity
 - Steam Generator Tube Sleeves
 2. Plant Systems Response
 - Steam Generator Overfill
 - PORV Operability
 - Pressurized Thermal Shock
 - Improved Accident Monitoring
 - Reactor Vessel Inventory Measurement

3. Human Factors Consideration

- Requirement on Reactor Coolant Pump Trip
- Rotary Switch Functional Identification
- Indicator Lights Burned Out
- Inconsistent Terminology
- Emergency Operation Procedures Improvement

4. Radiological Consequences

- Reassessment of Radiological Consequences Following a Postulated Steam Generator Tube Rupture Event
- Reevaluation of SGTR Design Basis Event
- Secondary System Isolation
- Review of Ventilation Intakes
- Collection of Snow Samples

5. Organizational Response

- Interactions with Regional Base Teams by the NRC Executive Team
- NRC Site Team
- Familiarization with NRC Response Plan
- Alternate Evacuation Routes and Sites
- Deescalation of Emergency Classification
- Offsite Dose Assessment