Commonwealth Edison
Dresden Nuclear Power Station
6500 North Dresden Road
Morris, Illinois 60450
Telephone 815/942-2920

February 14, 1994

GFS LTR #94-0053

Director, Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Washington, DC 20555

Attention: Document Control Desk

Gentlemen:

Subject: Monthly Operating Data Report
Dresden Nuclear Power Station
Commonwealth Edison Company
Docket Nos. 50-010, 50-237, and 50-249

Reference: Gary F. Spedl to Director of Nuclear Reactor Regulation (GFS LTR #93-0101), Monthly Operating Data Report, dated October 13, 1993.

Enclosed is the Dresden Nuclear Power Station Monthly Operating Summary Report for January 1994. No 10 CFR 50.59 evaluations are included in this report. As stated in last month's report, changes implemented in July 1993 and thereafter were to have been reported along with the FSAR updates. The FSAR update, however, was submitted as part of the re-baseline effort in mid-December 1993. Due to the change in 10 CFR 50.59 reporting requirements, the next FSAR update will not be due for Dresden until 1995; therefore, the evaluations for July 1993 through December 1993 will be reported in June 1994, which is when the FSAR updates would have been due under the previous 10 CFR 50.59 guidance. This change in reporting frequency remains within the guidance provided in 10 CFR 50.59(b)(2), which states that the report on changes, tests and experiments, with the accompanying safety evaluation summaries, may be submitted annually or along with the FSAR updates as required by 50.71(e), or at shorter intervals as may be specified by the licensee.

Please note that the format of the enclosed report has been revised to omit the sections which previously addressed the 10 CFR 50.59 evaluations.

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This information is supplied to your office in accordance with the instructions set forth in Regulatory Guide 1.16.

Sincerely,

Gary F. Spedl
Station Manager
Dresden Station

Enclosure

cc: NRC Region III Office
Illinois Dept. of Nuclear Safety, State of Illinois
U.S. NRC, Document Management Branch
Nuclear Licensing Administrator
Site Vice President
Regulatory Assurance
NRC Senior Resident Inspector
Site Quality Verification - Dresden
Site Engineering and Construction Manager
Nuclear Oversight Manager/ R. Janecek
General Electric
Comptroller's Office
INPO Records Center
UDI, Inc.- Wash., D.C.
File/Numerical

MONTHLY NRC

SUMMARY OF OPERATING EXPERIENCE,

CHANGES, TESTS, AND EXPERIMENTS

PER REGULATORY GUIDE 1.16 AND 10 CFR 50.59

FOR

DRESDEN NUCLEAR POWER STATION

COMMONWEALTH EDISON COMPANY

FOR January, 1994

UNIT	DOCKET	LICENSE
1	050-010	DPR-2
2	050-237	DPR-19
3	050-249	DPR-25

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January, 1994 NRC REPORT

1.0 Introduction

Dresden Nuclear Power Station is a three reactor generating facility owned and operated by the Commonwealth Edison Company of Chicago, Illinois. Dresden Station is located at the confluence of the Kankakee and Des Plaines Rivers, in Grundy County, near Morris, Illinois.

Dresden Unit 1 is a General Electric Boiling Water Reactor with a design net electrical output rating of 200 megawatts electrical (MWe). The unit is retired in place with all nuclear fuel removed from the reactor vessel. Therefore, no Unit 1 operating data is provided in this report.

Dresden Units 2 and 3 are General Electric Boiling Water Reactors with design net electrical output ratings of 794 MWe each.

Waste heat is rejected to a man-made cooling lake using the Kankakee River for make-up and the Illinois River for blowdown.

The Architect-Engineer for Dresden Units 2 and 3 was Sargent and Lundy of Chicago, Illinois.

This report for January, 1993, was compiled by Kevin W. Sykes of the Dresden Regulatory Assurance Staff, telephone number (815) 942-2920, extension 2704.

2.0 SUMMARY OF OPERATING EXPERIENCE FOR January, 1994

2.1 UNIT 2 MONTHLY OPERATING EXPERIENCE SUMMARY

01/01/94 Unit 2 entered the month on line and critical. Core thermal power has been administratively limited to 99% since 09-24-93 due to the feedwater flow nozzle calibration concern identified at Quad Cities. This was indicated in the Monthly Operating Report for September 1993 (Reference), but was inadvertently omitted from subsequent reports. The nozzle concern is being investigated under Problem Investigation Report (PIR) 12-2-93-117 (NTS #237-200-93-11700).

01/21/94 At 2300 hours, the reactor load was reduced from 795 MWe to approximately 295 MWe in order to make a drywell entry to repair a Rod Position Indication System (RPIS) probe. The drywell entry was made at 0800 hours on 1-22-94.

01/23/94 At 0125 hours, during the performance of Dresden Operating Surveillance (DOS) 0250-01, Partial Closure Operability Test of Main Steam Isolation Valves (MSIVs), relay 590-102H failed to pick up indicating that the limit switch on the 2-0203-1D MSIV had not reset to the fully open position. The reactor load was reduced to make a second drywell entry to replace the limit switch on the 2-0203-1D MSIV (NWR D23876).

The unit continued on line through the end of the month.

2.0 SUMMARY OF OPERATING EXPERIENCE FOR January, 1994

2.2 UNIT 3 MONTHLY OPERATING EXPERIENCE SUMMARY

01/01/94

Unit 3 entered the month critical and on line. Core thermal power has been administratively limited to 99% since 09-24-93 due to the feedwater flow nozzle calibration concern identified at Quad Cities. This was indicated in the Monthly Operating Report for September 1993 (Reference), but was inadvertently omitted from subsequent reports. The nozzle concern is being investigated under Problem Investigation Report (PIR) 12-2-93-117 (NTS #237-200-93-11700).

At approximately 1244 hours, transmission pole 292 failed, causing 138 kV line 1207 to trip, which in turn resulted in a loss of power to the Lift Station pumps (this event is being investigated under Problem Investigation Report (PIR) 12-2-94-001, NTS #237-200-94-00100). At 1500 hours the unit load was decreased from 625 MWe to approximately 416 MWe to reduce the discharge canal temperature to comply with the station's NPDES permit. At 2132 hours, Feedwater level disturbances were observed. Unit load was maintained at approximately 450 MWe while repair work was performed on the Feedwater low flow Regulating Valve (NWR D23576).

01/26/94

At 0145 hours, Unit load was reduced from 578 MWe to approximately 380 MWe to perform DOS 0250-02, Full Closure Timing and Exercising of Main Steam Isolation Valves (MSIVs). Problems were observed with the 3-0203-1A MSIV. At 1205 hours, a load drop was initiated from 372 MWe to approximately 290 MWe in order to make a drywell entry to adjust the timing on the 3-0203-1A MSIV.

Unit 3 remained critical and on line through the end of the month.

3.0 OPERATING DATA REPORT

3.1 OPERATING DATA REPORT - DRESDEN UNIT TWO

DOCKET No. 050-237
DATE February 1, 1994
COMPLETED BY K. W. Sykes
TELEPHONE (815) 942-2920

OPERATING STATUS

1. REPORTING PERIOD: **January, 1994**
2. CURRENTLY AUTHORIZED POWER LEVEL (MWth): 2,527
   MAXIMUM DEPENDABLE CAPACITY (MWe NET): 772
   DESIGN ELECTRICAL RATING (MWe Net): 794
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe Net): 99% of thermal output (2502 MWth)
4. REASONS FOR RESTRICTIONS (IF ANY: Feedwater flow nozzle calibration concern (see Section 2.1)

REPORTING PERIOD DATA

	PARAMETER	THIS MONTH	YEAR TO DATE	CUMULATIVE
5.	HOURS IN PERIOD	744	744	207,192
6.	TIME REACTOR CRITICAL (Hours)	744	744	154,458
7.	TIME REACTOR RESERVE SHUTDOWN (Hours)	0	0	0
8.	TIME GENERATOR ON-LINE (Hours)	744	744	148,021
9.	TIME GENERATOR RESERVE SHUTDOWN (Hours)	0	0	0
10.	THERMAL ENERGY GENERATED (MWHt Gross)	1,763,352	1,763,352	306,081,222
11.	ELECTRICAL ENERGY GENERATED (MWHe Gross)	555,803	555,803	97,662,340
12.	ELECTRICAL ENERGY GENERATED (MWHe Net)	529,819	529,819	93,341,010
13.	REACTOR SERVICE FACTOR (%)	100	100	74.5
14.	REACTOR AVAILABILITY FACTOR (%)	100	100	74.5
15.	GENERATOR SERVICE FACTOR (%)	100	100	71.4
16.	GENERATOR AVAILABILITY FACTOR (%)	100	100	71.4
17.	CAPACITY FACTOR (USING MDC Net) (%)	92.2	92.2	58.4
18.	CAPACITY FACTOR (USING DER Net) (%)	89.6	89.6	56.7
19.	FORCED OUTAGE FACTOR (%)	0	0	12.2

20. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (Type, Date and Duration of Each)

21. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP. N/A

3.0 OPERATING DATA REPORT

3.2 OPERATING DATA REPORT - DRESDEN UNIT THREE

DOCKET No. 050-249
DATE February 1, 1994
COMPLETED BY K. W. Sykes
TELEPHONE (815) 942-2920

OPERATING STATUS

1. REPORTING PERIOD: **January, 1994**
2. CURRENTLY AUTHORIZED POWER LEVEL (MWth): 2,527
   MAXIMUM DEPENDABLE CAPACITY (MWe Net): 773
   DESIGN ELECTRICAL RATING (MWe Net): 794
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe Net): 99% of thermal output (2502 MWth)
4. REASONS FOR RESTRICTIONS (IF ANY): Feedwater flow nozzle calibration concern (see Section 2.2)

REPORTING PERIOD DATA

5.	HOURS IN PERIOD	744	744	197,521
6.	TIME REACTOR CRITICAL (Hours)	744	744	143,450
7.	TIME REACTOR RESERVE SHUTDOWN (Hours)	0	0	0
8.	TIME GENERATOR ON-LINE (Hours)	744	744	138,059
9.	TIME GENERATOR RESERVE SHUTDOWN (Hours)	0	0	0
10.	THERMAL ENERGY GENERATED (MWHt Gross)	1,373,542	1,373,542	285,108,359
11.	ELECTRICAL ENERGY GENERATED(MWHe Gross)	438,229	438,229	91,614,740
12.	ELECTRICAL ENERGY GENERATED (MWHe Net)	416,486	416,486	87,001,196
13.	REACTOR SERVICE FACTOR (%)	100	100	72.6
14.	REACTOR AVAILABILITY FACTOR (%)	100	100	72.6
15.	GENERATOR SERVICE FACTOR (%)	100	100	69.9
16.	GENERATOR AVAILABILITY FACTOR (%)	100	100	69.9
17.	CAPACITY FACTOR (USING MDC Net) (%)	72.4	72.4	57.0
18.	CAPACITY FACTOR (USING DER Net) (%)	70.5	70.5	55.5
19.	FORCED OUTAGE FACTOR (%)	0	0	11.7

20. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (Type, Date and Duration of Each)
Refuel Outage 13, D3R13, is scheduled for March 1994. The scheduled duration for this refuel outage is 13 weeks.

21. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP
N/A

3.3 AVERAGE DAILY UNIT 2 POWER LEVEL

DOCKET No. 050-237
UNIT Dresden 2
DATE February 1, 1994
COMPLETED BY K. W. Sykes
TELEPHONE (815) 942-2920

MONTH: **January, 1994**

DAY	AVERAGE DAILY NET POWER LEVEL (MWe)	DAY	AVERAGE DAILY NET POWER LEVEL (MWe)
1	659	18	748
2	570	19	749
3	696	20	750
4	743	21	749
5	745	22	391
6	743	23	414
7	740	24	666
8	744	25	747
9	729	26	764
10	748	27	764
11	747	28	764
12	746	29	749
13	745	30	726
14	747	31	749
15	749		
16	746		
17	736		

3.4 AVERAGE DAILY UNIT 3 POWER LEVEL

DOCKET No. 050-249
UNIT Dresden 3
DATE February 1, 1994
COMPLETED BY K. W. Sykes
TELEPHONE (815) 942-2920

MONTH: **January, 1994**

DAY	AVERAGE DAILY NET POWER LEVEL (MWe)	DAY	AVERAGE DAILY NET POWER LEVEL (MWe)
1	533	18	572
2	421	19	570
3	534	20	567
4	617	21	564
5	600	22	561
6	599	23	557
7	598	24	553
8	596	25	549
9	594	26	356
10	590	27	587
11	587	28	545
12	583	29	541
13	581	30	540
14	579	31	538
15	579		
16	577		
17	574		

3.5 UNIT 2 SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH OF January, 1994

NO.	DATE	TYPE(1)	DURATION (HOURS)*	REASON(2)	METHOD OF SHUTTING DOWN REACTOR (3)	LICENSEE EVENT REPORT #	SYSTEM CODE(4)	COMPO-NENT CODE (5)	CORREC-TIVE ACTIONS/ COM-MENTS
15	940121	F	0	B	5	NA	NA	NA	SEE NOTE 1 BELOW

* Year-to-date forced outage hours = 896; Cumulative forced outage hours = 20,620

NOTE 1: Load reduced to approximately 295 MWe for drywell entry to repair Rod Position Indication System (RPIS) probe

TABLE KEY:

(1)
F: Forced
S: Scheduled

(2)
Reason:
A Equipment Failure (Explain)
B Maintenance or Test
C Refueling
D Regulatory Restriction

E Operator Training & Licensing Exam
F Administrative
G Operational Error
H Other (Explain)

(3)
Method:
1. Manual
2. Manual Scram
3. Automatic Scram
4. Other (Explain)
5. Load Reduction

(4)
Exhibit G Instructions for Preparation of Data Entry Sheets for Licensee Event Reports (LER) File (NUREG-0161)

(5)
Exhibit I Same Source as above.

3.6 UNIT 3 SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH January, 1994

NO.	DATE	TYPE(1)	DURATION (HOURS)*	REASON(2)	METHOD OF SHUTTING DOWN REACTOR (3)	LICENSEE EVENT REPORT #	SYSTEM CODE(4)	COMPO-NENT CODE (5)	CORREC-TIVE ACTIONS/ COM-MENTS
9	940101	F	0	B	5	NA	NA	NA	SEE NOTE 1 BELOW
10	940126	F	0	B	5	NA	NA	NA	SEE NOTE 2 BELOW

* Year-to-date forced outage hours = 1707; Cumulative forced outage hours = 18,239

Note 1 - Load reduced to approximately 416 MWe to reduce the discharge canal temperature

Note 2 - Load was reduced to approximately 380 MWe to perform DOS 0250-02

TABLE KEY:

(1)
F: Forced
S: Scheduled

(2)
Reason:
A Equipment Failure (Explain)
B Maintenance or Test
C Refueling
D Regulatory Restriction
E Operator Training & Licensing Exam
F Administrative
H Other (Explain)

(3)
Method:
1. Manual
2. Manual Scram
3. Automatic Scram
4. Other (Explain)
5. Load Reduction

(4)
Exhibit G Instructions for Preparation of Data Entry Sheets for Licensee Event Reports (LER) File (NUREG-0161)

(5)
Exhibit I Same Source as above.

3.7 UNIT 2 MAXIMUM DAILY ELECTRICAL LOAD FOR THE MONTH OF January, 1994.

Day	Hour Ending	UNIT 2 MAXIMUM DAILY ELECTRICAL LOAD (KWe)
1	1900	758,000
2	0100	722,000
3	1500	791,000
4	1700	791,000
5	0100	791,000
6	0800	791,000
7	0700	791,000
8	0100	791,000
9	0700	794,000
10	0100	793,000
11	0400	791,000
12	0100	791,000
13	1900	791,000
14	0200	792,000
15	1900	794,000
16	0300	796,000
17	0800	797,000
18	1700	796,000
19	0700	798,000
20	2000	797,000
21	0100	796,000
22	0100	663,000
23	2400	598,000
24	2100	778,000
25	0900	809,000
26	1700	814,000
27	1100	812,000
28	1000	812,000
29	0400	808,000
30	0900	779,000
31	1200	808,000

3.8 UNIT 3 MAXIMUM DAILY ELECTRICAL LOAD FOR January, 1994

Day	Hour Ending	UNIT 3 MAXIMUM DAILY ELECTRICAL LOAD (KWe)
1	1500	636,000
2	0900	504,000
3	2400	655,000
4	0200	671,000
5	1000	634,000
6	0100	632,000
7	1200	632,000
8	0100	629,000
9	0100	627,000
10	0100	624,000
11	0100	621,000
12	0100	617,000
13	0100	614,000
14	0100	612,000
15	0100	611,000
16	0100	610,000
17	0100	607,000
18	0100	605,000
19	0100	602,000
20	0100	600,000
21	0100	597,000
22	0300	594,000
23	0100	590,000
24	0100	588,000
25	0100	583,000
26	0100	578,000
27	0500	650,000
28	0100	585,000
29	0600	573,000
30	0100	571,000
31	0100	569,000

4.0 UNIQUE REPORTING REQUIREMENTS

4.1 MAIN STEAM RELIEF VALVE OPERATIONS

None

4.2 OFF-SITE DOSE CALCULATION MANUAL (ODCM) CHANGES

Offsite Dose Calculation Manual (ODCM) Revision 1.0, Effective Date 1-1-94

The Offsite Dose Calculation Manual (ODCM) was revised primarily to address the revised 10CFR20, which became effective 1-1-94, and non-10CFR20 related areas. The 10CFR20 revisions include the addition of the restricted area boundary map and distance values, whole body dose factors which are now calculated at 1 cm depth, addition of equations to calculate TEDE, DDE, CEDE and CDE, expansion of the nuclide database, and the liquid effluent concentration limit term was revised from MPC to DWC (derived water concentration). Non-10CFR20 changes included the incorporation of land use census data, the revision of the ground shine term which is now part of the DDE equation, the addition of joint frequency distribution summary tables for reference purposes and the incorporation of Unit 1 information into Chapter Twelve.

The changes made within this revision do not reduce the accuracy or reliability of dose calculations. Additionally, setpoint calculations were reviewed and found to be valid post 1-1-94.

4.3 MAJOR CHANGES TO THE RADIOACTIVE WASTE TREATMENT SYSTEMS DURING January, 1994

None.

4.4 FAILED FUEL ELEMENT INDICATIONS

4.4.1 Unit 2

Unit 2 fuel performance during January, 1994, continued to show no indications of leaking fuel. This is based on the sum of the activities of the six (6) Noble Gases as measured at the Recombiner. Therefore, Unit 2 had excellent fuel performance.

4.4.2 Unit 3

Unit 3 fuel performance during January, 1994, continued to show no indications of leaking fuel. This is based on the sum of the activities of the six (6) Noble Gases as measured at the Recombiner. Therefore, Unit 3 had excellent fuel performance.

5.0 PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

5.1 Amendments to Facility License or Technical Specifications implemented during January, 1994.

None.

5.2 Safety Related Maintenance (Unit 2 and 3)

5.2.1 Safety related preventive maintenance activities for January, 1994 are summarized in the attached computer printout. Unit 2/3 items are designated as "00" in the "Unit" column.

SELECTION CRITERIA: MAINT COMP DATE IN RANGE OF '01/01/94' THRU '01/31/94' NUC SAFE EQ 'Y' PREV MAIN EQ 'Y'

OBS	UNIT	EPN	EID DESCRIPTION	MAINT COMP DATE
1	00	2/3-0001	ALL INSTRUMENT MAINT EID/EPN APPLICATIONS	01/03/94
2	00	2/3-0001	ROSEMOUNT TRANSMITTER MODEL 1152, U2 AND 3	01/03/94
3	00	2/3-305-CRA4661	DRIVE CONTROL ROD #A4661	01/21/94
4	00	2/3-305-CRA8504	DRIVE CONTROL ROD #A8504	01/21/94
5	00	2/3-305-CR1043	DRIVE CONTROL ROD #1043	01/21/94
6	00	2/3-305-CR5710	DRIVE CONTROL ROD #5710	01/21/94
7	00	2/3-305-CR6310	DRIVE CONTROL ROD #6310	01/21/94
8	00	2/3-305-CR6391	DRIVE CONTROL ROD #6391	01/21/94
9	00	2/3-305-CR679C	DRIVE CONTROL ROD #679C	01/21/94
10	00	2/3-305-CR751	DRIVE CONTROL ROD #751	01/21/94
11	00	2/3-305-CR8522	DRIVE CONTROL ROD #8522	01/21/94
12	00	2/3-305-CR929	DRIVE CONTROL ROD #929	01/21/94
13	00	2/3-305-CR957	DRIVE CONTROL ROD #957	01/21/94
14	00	2/3-305-CR977	DRIVE CONTROL ROD #977	01/21/94
15	02	2-300	SURVEILLANCE HYD CONTROL UNIT ANCHOR BOLTS	01/10/94
16	02	2-1600-DOOR	DOOR PERSONNEL INTERLOCK DRYWELL	01/15/94
17	02	2-1600-DOOR	DOOR PERSONNEL INTERLOCK DRYWELL	01/23/94
18	02	2-2301-28	VALVE GLOBE AO HPCI STEAM LINE DRAIN ISOL	01/20/94
19	03	3-0300	MISC SYSTEM CONTROL ROD DRIVE HYDRAULICS	01/13/94
20	03	3-0305-101	VALVE MAN HCV ISOLATION #101	01/06/94
21	03	3-0500	MISC SYSTEM REACTOR PROTECTION	01/17/94
22	03	3-1501-51A	SWITCH A LPCI DISCHARGE HEADER HIGH PRESSURE	01/26/94

OBS	DESCRIPTION
1	REQUESTED--- REPLACE DEFECTIVE INSTRUMENT TEST FITTINGS DURING PERFORM OF ROUTINE MAINT------
2	REQUESTED--- REPLACE ROSEMONT E.Q. TRANSMITTER COVER O-RINGS WHILE PERFORMING SURVEILLANCES--
3	REQUESTED--- FLUSH + REBUILD CRD REMOVED FROM D2 CORE POSITION WR 10409----------
4	REQUESTED--- FLUSH AND REBUILD CRD S/N A8504 WHICH WAS REMOVED FROM U-2 LOCATION J-8---------
5	REQUESTED--- FLUSH AND REBUILD CRD S/N 1043 WHICH WAS REMOVED FROM U-2 LOCATION D-4---------
6	REQUESTED--- FLUSH AND REBUILD CRD S/N 5710 WHICH WAS REMOVED FROM U-2 LOCATION G-10--------
7	REQUESTED--- FLUSH + REBUILD CRD REMOVED FROM D2 CORE POSITION L-13 WR 10422----------
8	REQUESTED--- FLUSH + REBUILD CRD REMOVED FROM D2 CORE POSITION F-9 WR 10397----------
9	REQUESTED--- FLUSH AND REBUILD CRD S/N 679C WHICH WAS REMOVED FROM U-2 LOCATION K-7---------
10	REQUESTED--- FLUSH + REBUILD CRD REMOVED FROM D2 CORE POSITION E-13 WR 10398.----------
11	REQUESTED--- FLUSH + REBUILD CRD REMOVED FROM D2 CORE POSITION G-11 WR 10411----------
12	REQUESTED--- FLUSH AND REBUILD CRD S/N 929 WHICH WAS REMOVED FROM U-2 LOCATION H-11--------
13	REQUESTED--- FLUSH AND REBUILD CRD 957 REMOVED FROM D2 CORE POSITION F-15 WR 10425----------
14	REQUESTED--- FLUSH + REBUILD CRD REMOVED FROM D2 CORE POSITION F-13 WR 10412----------
15	REQUESTED--- PERFORM HCU ANCHOR BOLT TORQUE CHECK OF WEST BANK.----------
16	REQUESTED--- CHECK STRONGBACK TORQUE VALUES - RETORQUE TO SPECIFIED VALUES.----------
17	REQUESTED--- REMOVE STRONGBACK IN PREP FOR DW ENTRY, RE-INSTALL PRIOR TO LLRT----------
18	REQUESTED--- TEST OF 2-2301-28 PER SYSTEM ENGINEER AND SITE ENGINEER.----------
19	REQUESTED--- PROVIDE LIST OF SERIAL NUMBERS OF DRIVES REMOVE AND REPLACED----------
20	REQUESTED--- FABRICATE REPLACEMENT VALVE ASSEMBLIES FOR THE 2(3)-305-101 VALVES\----------
21	REQUESTED--- TAKE AMPERAGE MEASUREMENTSAT EPA BREAKERS 3A-1,3A-2,3B-1,3B-2,3AB-1 AND 3AB-2.--
22	REQUESTED--- REFUEL LPCI LOOP ALARM SW CALIB-D3R13----------

SELECTION CRITERIA: MAINT COMP DATE IN RANGE OF '01/01/94' THRU '01/31/94' NUC SAFE EQ 'Y' PREV MAIN EQ 'Y'

OBS	UNIT	EPN	EID DESCRIPTION	MAINT COMP DATE
23	03	3-1600-DOOR	DOOR PERSONNEL INTERLOCK DRYWELL	01/26/94
24	03	3-1641-101	XMITTER PRIMARY CONT LEVEL DRYWELL PRESSURE	01/05/94

OBS	DESCRIPTION
23	REQUESTED--- U3 DRYWELL PERSONNEL INTERLOCKS. PERFORM INSPECTION----------
24	REQUESTED--- REPLACE XMITTER 3-1641-101.----------

5.2 Safety Related Maintenance (Unit 2 and 3)

5.2.2 Safety related corrective maintenance activities for January, 1994 are summarized in the attached computer printout. Unit 2/3 items are designated as "00" in the "Unit" column.

SELECTION CRITERIA: MAINT COMP DATE IN RANGE OF '01/01/94' THRU '01/31/94' NUC SAFE EQ 'Y' CORR MAIN EQ 'Y'

OBS	UNIT	EPN	EID DESCRIPTION	MAINT COMP DATE
1	00	2/3-203-BK6296	VALVE SAFETY MAIN STEAM SPARE #BK6296	01/17/94
2	00	2/3-0203-BK6525	VALVE SAFETY MAIN STEAM SPARE #BK6525	01/17/94
3	02	2-203-1D	SWITCH LIMIT OPEN INBOARD MSIV 1D	01/23/94
4	02	2-0700-APRM2	MONITOR AVERAGE POWER RANGE CHANNEL 2	01/13/94
5	02	2-1401-A	MOTOR A CORE SPRAY PUMP	01/17/94
6	02	2-1466C	SWITCH A CORE SPRAY PUMP DISCHARGE PRESSURE	01/14/94
7	02	2-2420B	RECORDER DRYWELL HIGH RANGE MONITOR RADIATION	01/11/94
8	03	3-261-34A	SWITCH RECIRC LOOP RISER DIFF PRESSURE	01/05/94
9	03	3-1149	SWITCH SBLC SUCTION HEADER TEMPERATURE	01/19/94
10	03	3-1402-36B	VALVE STOP CHECK B CORE SPRAY	01/07/94
11	03	3-1500	SYSTEM LPCI HANGERS/SUPPORTS CLASS I	01/10/94
12	03	3-2301-28	VALVE GLOBE AO HPCI STEAM LINE DRAIN ISOL	01/15/94
13	03	3-5742A	OPERATOR AO VALVE	01/30/94
14	03	3-6600	MISC SYSTEM DIESEL GENERATOR	01/31/94

OBS	DESCRIPTION
1	REQUESTED--- REBUILD,REPAIR AND TEST VLV REMOVED FROM POSIT B-SET PRESS AT 1260 ,PSI---------
2	REQUESTED--- REBUILD,REPAIR,AND TEST SPARE SAFETY-REMOVED FROM POSIT D----------
3	REQUESTED--- 1 D MSIV LIM SW APPEARS TO BE HANGING UP-WHENTHE 1 D MSIV WAS 10% TESTED THE 590
4	REQUESTED--- THE OUTPUT VOLTAGE OF PS26 WAS FOUND AT ZERO VOLTAGE, REPLACED FUSE - FUSE BLEW.
5	REQUESTED--- LEAK AT MOTOR COOLING WATER LINE UNION.----------
6	REQUESTED--- TEST EQUIP. CONNECTION FOUND BADLY DAMAGED, TUBING NUT ON SENSING LINE DAMAGED.-
7	REQUESTED--- RED PEN SEEMS TO BE READING HARD DOWNSCALE. INVESTIGATE AND REPAIR.----------
8	REQUESTED--- LPCI LOOP BREAK DIFFERENTIAL PRESSURE SWITCH 3-261-34A IS READING STEADY AT 3.9-
9	REQUESTED--- SUCTION LINE HEAT TRACE TEMP CONTROLLER NOT CONTROLLING BELOW ALARM POINT ALARM-
10	REQUESTED--- HANDWHEEL FELL OFF. TAPED TO PIPE AT VALVE, REPLACE.----------
11	REQUESTED--- MIN-FLOW LINE HANGER HAS IMPROPER BOLTING. (BOLTS LOOSE) PLEASE REPAIR.--------
12	REQUESTED--- VALVE LEAKING THRU - CONTACT M. CHURILLA FOR GUIDANCE.----------
13	REQUESTED--- DAMPER FAILED CLOSED DURING NORMAL OPS. WOULDNOT OPEN ON 4 ATTEMPTS. NO AIR LEA
14	REQUESTED--- INSTALL TACH DRIVE CAP AT BASE OF U3 DG GOVERNOR----------