Docket Nos. 50-313
and 50-368

February 9, 1994

LICENSEE: Entergy Operations, Inc.

FACILITY: Arkansas Nuclear One, Units 1&2 (ANO-1&2)

SUBJECT: MEETING SUMMARY - LICENSING ACTIONS AND ACTIVITIES

On February 2, 1994, representatives of Entergy Operations, Inc. met with the NRC staff to discuss the status of licensing actions and activities. Meeting attendees are listed in Enclosure 1. A booklet handed out by the licensee is included as Enclosure 2.

The bulk of the discussion focused on who had the next action (NRC or Entergy).

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Enclosures:
1. Meeting Attendees
2. Handout

cc w/enclosure:
See next page

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DATE	2/7/94	2/8/94	2/8/94	2/9/94
COPY	YES/NO	YES/NO	YES/NO	YES/NO

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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ENCLOSURE 1

LICENSING ACTION AND ACTIVITY MEETING

ARKANSAS NUCLEAR ONE, UNITS 1&2

ATTENDEES

NAME	ORGANIZATION
T. Alexion	NRC/NRR
G. Kalman	NRC/NRR
R. Bevan	NRC/NRR
D. Mims	Director, Licensing
D. James	Supervisor, Licensing

ENCLOSURE 2

ARKANSAS NUCLEAR ONE

NRC TAC MEETING

February 2, 1994

TAC ITEM SUMMARY

February 1994	Open 40		Action ANO 14	Action NRC 26
		Closed 9	Added 1	Reopened 3 (~~IPE~~, Thermolag) 1
November 1993	Open 45		Action ANO 9	Action NRC 36
		Closed 15	Added 20	
July 1993	Open 38		Action ANO 5	Action NRC 33
		Closed 14	Added 17	
March 1993	Open 35		Action ANO 8	Action NRC 27
July 1992	Open 40			
October 199[illegible]	Open 43			
April 1991	Open 46			
September 1990	Open 95			
February 1990	Open 69			

NRC OPEN TAC NUMBERS JANUARY 1994

TAC NO	NAME	LICENSING ENGINEER	ACTION
M69426	Generic Letter 87-02 ANO-1 Seismic Qualification of Mechanical and Electric Equipment (SQUG)	Natalie Mosher	ANO
M69427	Generic Letter 87-02 ANO-2 Seismic Qualification of Mechanical and Electric Equipment (SQUG)	Natalie Mosher	ANO
M72108	NRC Bulletin 88-11, ANO-1 Pressurizer Surge Line Thermal Stratification	Jim Haley	NRC
M74376	Generic Letter 88-20 Supplement 1, ANO-1 IPE	Natalie Mosher	NRC
M74377	Generic Letter 88-20 Supplement 1, ANO-2 IPE	Natalie Mosher	NRC
M74906	GL 89-19 ANO-1 Safety Implication of Control Systems	Natalie Mosher	NRC
M74907	GL 89-19 ANO-2 Safety Implication of Control Systems	Natalie Mosher	NRC
M77326	GL 90-06 ANO-1 PORV and Block Valve Reliability	Clint Szabo	NRC
M77327	GL 90-06 ANO-2 PORV and Block Valve Reliability	Clint Szabo	NRC
M77399	Generic Letter 90-06 ANO-2 Low Temperature Overpressure Protection	Clint Szabo	ANO
M82550	ANO-2 Electrical Circuitry Isolation Requirements	David McNeil	NRC
M83430	GL 92-01 ANO-2 Reactor Vessel Structural Integrity	Jim Haley	NRC
M83588	ANO-1 IPEEE for External Events (GL 88-20 Supplement 4)	Natalie Mosher	ANO
M83589	ANO-2 IPEEE for External Events (GL 88-20 Supplement 4)	Natalie Mosher	ANO
M83730	GL 92-01 ANO-1 Reactor Vessel Structural Integrity	Jim Haley	NRC
M85352	ANO-1 Response to NRCB 90-01 Supplement 1, Loss of Fill Oil in Rosemounts Transmitters	Jim Haley	ANO
M85353	ANO-2 Response to NRCB 90-01 Supplement 1, Loss of Fill Oil in Rosemounts Transmitters	Jim Haley	ANO
M85515	ANO-2 Thermo-Lag (GL 92-08)	John Dosa	ANO
M85871	ANO-2 Containment Internal Pressure Revision	Natalie Mosher	NRC
M86005	ANO-1 Change to Correct Typographical Errors Throughout TS	David McNeil	NRC
M86264	ANO-1 Technical Specification Change Concerning C-3 Inspections of Steam Generator Tubes	John Dosa	NRC
M86468	ANO-1 Review of ODCM	John Dosa	NRC
M86469	ANO-2 Review of ODCM	John Dosa	NRC
M86946	ANO-1 ERV Cycling During Plant Runback	Natalie Mosher	ANO

M87066	ANO-2 Remove Cycle Specific Parameters per GL 88-16	Natalie Mosher	NRC
M87147	ANO-2 Indefinite Bypass of PPS Channel	Glenn Ashley	NRC
M87243	ANO-2 Shutdown Cooling Line Thermal Stratification	Jim Haley	ANO
M87263	ANO-1 Exemption from 10CFR26 - Fitness For Duty	Natalie Mosher	ANO
M87264	ANO-2 Exemption from 10CFR26 - Fitness For Duty	Natalie Mosher	ANO
M87445	ANO-1 Containment Net Free Volume	Clint Szabo	NRC
M87718	ANO-1 Proposed Revision to ISP	Natalie Mosher	ANO
M87719	ANO-2 Proposed Revision to ISP	Natalie Mosher	ANO
M87829	ANO-1 Review One Year Response to NRC SE for Second 10-Year IST Program	John Dosa	NRC
M87912	ANO-1 Inaccuracy of MOV Diagnostic Equipment	Jim Haley	NRC
M87913	ANO-2 Inaccuracy of MOV Diagnostic Equipment	Jim Haley	NRC
M88074	ANO-1 Prop. Alt. to 10CFR50.55A 10-Yr. ISI/IST Updat[illegible]	John Dosa	NRC
M88075	ANO-2 Prop. Alt. to 10CFR50.55A 10-Yr. ISI/IST Updat[illegible]	John Dosa	NRC
M88080	ANO-1 Relief Req. for Successive Inspections of RCP Weld Flaws	John Dosa	NRC
M88110	ANO-2 Incore Detector System TS Change Request	Clint Szabo	NRC
M88190	ANO-2 ISI Relief Request	John Dosa	NRC
	ANO-2 TS Change RPS Operation with Inoperable Functions	Clint Szabo	NRC
	ANO-2 Review One Year Response to NRC SE for Second 10-Year IST Program	John Dosa	NRC
	ANO-1 TS Change to Allow Handling of Spent Fuel Shipping Casks Using the Auxiliary Building Crane	David McNeil	ANO

ANO/NRC RELATED ISSUES JANUARY 1994

TAC NO M69426 **UNIT** 1 **PRIORITY** MED **ACTION** ANO

NAME Generic Letter 87-02 Seismic Qualification of Mechanical and Electric Equipment (SQUG)

RELATED TAC # M69427 (ANO-2), M83588 (ANO-1 IPEEE)

STATUS In February, 1987, the Nuclear Regulatory Commission (NRC) issued Generic Letter #87-02. In the generic letter, the NRC concludes, "...that the seismic adequacy of certain equipment in operating nuclear power plants must be reviewed against seismic criteria not in use when these plants were licensed." Arkansas Nuclear One - Units 1 and 2 (ANO-1 & 2) are subject to the requirements of this generic letter because both nuclear units were built and licensed prior to the use of more current and "state-of-the-art" seismic criteria. The impact of the generic letter on ANO is that it will be necessary to reevaluate the seismic adequacy of certain equipment identified as "essential" per USI A-46 criteria.

An owner's group, Seismic Qualification Utility Group (SQUG) was formed during the initial stages of USI A-46 development to demonstrate that past earthquake experience data can be used to verify the seismic adequacy of equipment at nuclear power plants. The efforts of the SQUG resulted in the NRC agreeing (in Generic Letter #87-02) that the use of past earthquake experience data is an acceptable method of verifying the seismic adequacy of equipment in operating nuclear power plants and can be used as a basis for resolving USI A-46. The NRC has also agreed upon a generic resolution process as the most viable and cost-effective approach to implementing the requirements of USI A-46.

With NRC endorsement and consent, the SQUG has developed generic resolution criteria and guidelines to be used in the implementation of the requirements of Generic Letter #87-02 (Generic Implementing Procedures, GIP). These procedures will have NRC acceptance (i.e. Safety Evaluation Report, SER) prior to use by the SQUG membership in resolving USI A-46. The GIP, was submitted in Spring, 1988. In letter dated March 11, 1991, the NRC provided approximately 70 comments on Revision 2 of the GIP to SQUG.

In letter OCAN039108 (dated March 21, 1991) Entergy Operations stated that due to the SER schedule slippage, the anticipated SQUG plant walkdowns would be deferred at least one refueling outage. However, final schedules can only be determined after issuance of the SER.

By NRC letter dated May 22, 1992 (OCNA059214) Supplement 1 to GL 87-02 was issued, transmitting the SSER-2 on SQUG GIP-2 which requested licensees to provide a schedule for implementation of GIP-2 within 120 days (due September 18, 1992).

Entergy Operations responded on September 18, 1992 (0CAN099201) and committed to implement GIP-2 during 1R11 with completion by May '995. The NRC responded on November 16, 1992 (0CNA119210) with a Safety Evaluation and a request for additional information. ANO responded to the Staff's request on January 28, 1993 (0CAN019307). The Staff had an additional request via telecon on the ANO-1 response spectra and requested Entergy Operations to submit additional information within 60 days.

Entergy Operations submitted the requested information on March 26, 1993 (1CAN039306). By NRC letter dated August 19, 1993 (1CNA089303), the Staff issued the final safety evaluation for ANO-1. The Staff considered the ANO-1 floor response spectra as "median centered" rather than "conservative design".

ACTION NEEDED ANO to complete walkdowns and provide submittal by May 1995.

LICENSING ENGINEER Natalie Mosher

TAC NO M69427 **UNIT** 2 **PRIORITY** MED **ACTION** ANO

NAME Generic Letter 87-02 Seismic Qualification of Mechanical and Electric Equipment (SQUG)

RELATED TAC # M69426 (ANO-1), M83589 (ANO-2 IPEEE)

STATUS In February, 1987, the Nuclear Regulatory Commission (NRC) issued Generic Letter #87-02. In the generic letter, the NRC concludes, "...that the seismic adequacy of certain equipment in operating nuclear power plants must be reviewed against seismic criteria not in use when these plants were licensed." Arkansas Nuclear One - Units 1 and 2 (ANO-1 & 2) are subject to the requirements of this generic letter because both nuclear units were built and licensed prior to the use of more current and "state-of-the-art" seismic criteria. The impact of the generic letter on ANO is that it will be necessary to reevaluate the seismic adequacy of certain equipment identified as "essential" per USI A-46 criteria.

An owner's group, Seismic Qualification Utility Group (SQUG) was formed during the initial stages of USI A-46 development to demonstrate that past earthquake experience data can be used to verify the seismic adequacy of equipment at nuclear power plants. The efforts of the SQUG resulted in the NRC agreeing (in Generic Letter #87-02) that the use of past earthquake experience data is an acceptable method of verifying the seismic adequacy of equipment in operating nuclear power plants and can be used as a basis for resolving USI A-46. The NRC has also agreed upon a generic resolution process as the most viable and cost-effective approach to implementing the requirements of USI A-46.

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In letter OCAN039108 (dated March 21, 1991) Entergy Operations stated that due to the SER schedule slippage, the anticipated SQUG plant walkdowns would be deferred at least one refueling outage. However, final schedules can only be determined after issuance of the SER.

By NRC letter dated May 22, 1992 (OCNA059214). Supplement 1 to GL 87-02 transmitting the SSER-2 on SQUG GIP-2 which requested licensees to provide a schedule for implementation of GIP-2 within 120 days (due September 18, 1992).

Entergy Operations responded on September 18, 1992 (0CAN099201) and committed to implement GIP-2 during 2R10 with completion by May 1995. The Staff responded on November 16, 1992 (0CNA119210) with a Safety Evaluation for ANO-2 and a request for additional information for ANO-1.

ACTION NEEDED ANO to complete walkdowns and provide submittal by May 1995.

LICENSING ENGINEER Natalie Mosher

TAC NO M72108 **UNIT 1** **PRIORITY** LOW **ACTION** NRC

NAME NRC Bulletin 88-11, Pressurizer Surge Line Thermal Stratification

RELATED TAC # M72109 (ANO-2)

STATUS ANO is responding to this bulletin as part of an Owners Group effort for both units. By letter dated June 2, 1989 (1CAN068906), ANO-1 submitted BAW-2085 which addressed Bulletin 88-11. On August 17, 1989, the NRC requested that the BWOG respond to questions on this report. A partial response was provided on September 29, 1989, with the remaining response due November 30, 1989. The BWOG response to the RAI was docketed on ANO-1's docket (1CAN108914).

NRC transmitted its evaluation of the BWOG bounding analysis on July 31, 1991 (1CNA079009). This analysis was part of Entergy Operations' response of June 2, 1989. The NRC found that the analysis provided sufficient information to justify continued plant operation until the final report is complete. However, issues have been identified that need to be resolved before they can conclude that the surge line meets all appropriate Code limits for a 40-year plant life.

In letter dated August 26, 1991 (1CNA089102), the NRC transmitted the SER for the generic final analysis of the surge line thermal stratification. The NRC found the methodology used to be generally acceptable, except for one major issue. This issue involves the stress index used in calculating thermal expansion stress for surge line elbows. This issue may have significant impact on acceptability of stress levels and fatigue life of the surge line. The Staff proposed several options for resolution of the issue.

In letter dated June 11, 1992, the BWOG transmitted the results of the re-evaluation of the surge line elbows. In letter dated July 16, 1992 (1CAN079201), ANO-1 provided the verification of the plant-specific applicability of the program and its results.

ACTION NEEDED NRC review and closure.

LICENSING ENGINEER Jim Haley

TAC NO M74376 **UNIT** 1 **PRIORITY** LOW **ACTION** NRC

NAME Generic Letter 88-20 Supplement 1, ANO-1 IPE

RELATED TAC # M74377

STATUS Supplement 1 to Generic 88-20 required a schedule for actions to comply with the Generic Letter.

ANO submitted correspondence on November 1, 1989, to address IPE approach, methodology, and schedule. The NRC responded on January 12, 1990 to indicate acceptance of the submitted schedule.

Entergy Operations submitted 0CAN079101, dated July 17,1991, to extend the submittal date to August 31, 1992.

Entergy Operations submitted 0CAN039204 on March 13, 1992 to extend the submittal date to April 30, 1993.

Entergy Operations submitted the ANO-1 IPE on April 29, 1993 (1CAN049301).

ACTION NEEDED NRC to issue Safety Evaluation.

LICENSING ENGINEER Natalie Mosher

TAC NO M74377 **UNIT** 2 **PRIORITY** LOW **ACTION** NRC

NAME Generic Letter 88-20 Supplement 1, ANO-2 IPE

RELATED TAC # M74376

STATUS

STATUS Supplement 1 to Generic 88-20 required a schedule for actions to comply with the Generic Letter.

ANO submitted correspondence on November 1, 1989, to address IPE approach, methodology, and schedule. The NRC responded on January 12, 1990 to indicate acceptance of the submitted schedule.

Entergy Operations submitted 0CAN079101, dated July 17,1991, to extend the submittal date to August 31, 1992.

Entergy Operations submitted the ANO-2 IPE on August 28, 1992 (2CAN089201).

ACTION NEEDED NRC to issue Safety Evaluation.

LICENSING ENGINEER Natalie Mosher

TAC NO M74906 **UNIT** 1 **PRIORITY** LOW **ACTION** NRC

NAME Generic Letter 89-19 Safety Implication of Control Systems

RELATED TAC # M74907 (ANO-2)

STATUS Generic Letter 89-19 describes the results of the NRC staff's contractors review of the safety implications of failures of non-safety grade control systems and recommends implementation of safety grade overfill protection for all plants.

ANO submitted its response on March 19, 1990 (1CAN039001) which concluded that no immediate plant changes were necessary. However, this response committed to include the steam generator overfill issue in the evaluations associated with the IPE.

In the NRC letters dated October 1, 1990 (1CNA109002) and 2CNA109002 the Staff disagreed with our responses and requested revised submittals within 45 days of receipt.

The Combustion Engineering Owners Group (CEOG) contacted Mr. Scott Newberry of the NRC and obtained his agreement for a meeting between the NRC and representatives of the CEOG specifically to discuss approaches to further address GL 89-19.

In our letter of November 19, 1990 (OCAN119003), we requested a time extension until after the November 20 meeting.

At the meeting the CEOG discussed the analysis used by the Staff in the determination of the need for the addition of the Main Feedwater Pump (MFWP) trip. The staff agreed to the following:

To stop the 45 day clock on the utility commitments to install a MFWP trip until the Staff can review the CEOG presentation.

After the Staff has reviewed this information, they will provide utilities with a response and allow an additional four weeks for the utilities to respond.

Although this was a CEOG effort, the same conclusion also apply to ANO-1. This is discussed in our letter of November 19, 1990 (OCAN119003).

Per NRC verbal request, Entergy Operations was asked to document ANO-1's (B&W's design) similarity on SGOP to that of CE design.

In letter dated May 29, 1992 (OCAN059207), Entergy Operations provided a discussion of the applicability of the CEOG presentation to ANO-1

ACTION NEEDED NRC review and closure.

LICENSING ENGINEER Natalie Mosher

TAC NO M74907 **UNIT** 2 **PRIORITY** LOW **ACTION** NRC

NAME Generic Letter 89-19 Safety Implication of Control Systems

RELATED TAC # M74906 (ANO-1)

STATUS Generic Letter 89-19 describes the results of the NRC staff's contractors review of the safety implications of failures of non-safety grade control systems and recommends implementation of safety grade overfill protection for all plants.

ANO submitted its response on March 19, 1990 (1CAN039001) which concluded that no immediate plant changes were necessary. However, this response committed to include the steam generator overfill issue in the evaluations associated with the IPE.

In the NRC letters dated October 1, 1990 (1CNA109002 and 2CNA109002) the Staff disagreed with our responses and requested revised submittals within 45 days of receipt.

The Combustion Engineering Owners Group (CEOG) contacted Mr. Scott Newberry of the NRC and obtained his agreement for a meeting between the NRC and representatives of the CEOG specifically to discuss approaches to further address GL 89-19.

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At the meeting the CEOG discussed the analysis used by the Staff in their determination of the need for the addition of a Main Feedwater Pump (MFWP) trip. The staff agreed to the following:

To stop the 45 day clock on utility commitments to install a MFWP trip until the Staff can review the CEOG presentation.

After the Staff has reviewed this information, they will provide utilities with a response and allow an additional four weeks for the utilities to respond.

ACTION NEEDED NRC to review CEOG approach and respond

LICENSING ENGINEER Natalie Mosher

TAC NO M77326 **UNIT 1** **PRIORITY** LOW **ACTION** NRC

NAME Generic Letter 90-06 PORV and Block Valve Reliability

RELATED TAC # M77327 (ANO-2), M77399 (ANO-2)

STATUS Generic Letter 90-06, Resolution of Generic Issue 70, " Power-Operated Relief Valve and Block Valve Reliability," and Generic Issue 94, "Additional Low-Temperature Overpressure Protection for Light-Water Reactors" was issued on June 25, 1990 (OCNA069027) to advise licensees of the NRC Staff positions with respect to resolution of these Generic Issues. The Staff positions are delineated in Enclosures A and B to the Generic Letter. A 180 day response was requested.

Entergy Operations letter dated December 21, 1990 (1CAN129013), provided the ANO-1 response to Generic Letter 90-06. Actions taken for the ERVs and block valves were considered to be in compliance with Generic Letter 90-06 except for Technical Specification changes. Since the ERVs do not perform a safety function, no changes to the specifications were considered appropriate.

By letter dated November 24, 1992 (0CNA119222), the NRC requested that Entergy Operations provide additional information regarding the ANO-1 GL 90-06 responses within 90 days. Other B&W utilities received similar requests for additional information. The B&W Owner's Group Technical Specifications Subcommittee determined that a generic response to the NRC request was appropriate. This generic approach, with additional plant specific responses, was accepted by the NRC Technical Reviewer in a conference call between the B&W Owner's Group Technical Specification Subcommittee and the NRC.

The B&W Owner's Group Generic response to the NRC, dated January 18,1993 (OG-1128), provided the generic responses to the NRC request for additional information. Entergy Operations letter dated February 5, 1993 (1CAN029301), provided the plant specific responses to the NRC request for additional information.

ACTION NEEDED NRC review and closure.

LICENSING ENGINEER Clint Szabo

TAC NO M77327 **UNIT** 2 **PRIORITY** LOW **ACTION** NRC

NAME Generic Letter 90-06 PORV and Block Valve Reliability

RELATED TAC # M77399 (ANO-2), M77326 (ANO-1)

STATUS Generic Letter 90-06, Resolution of Generic Issue 70, "Power-Operated Relief valve and Block Valve Reliability," and Generic Issue 94, "Additional Low-Temperature Overpressure Protection for Light-Water Reactors" was issued on June 25, 1990 (OCNA069027) to advise licensees of the NRC Staff positions with respect to resolution of these Generic Issues. The Staff positions are delineated in Enclosures A and B to the Generic Letter. A 180 day response was requested.

Entergy Operations letter dated December 21, 1990 (2CAN129013), provided the ANO-2 response to Generic Letter 90-06. Actions taken and proposed are considered to comply with Generic Letter 90-06. Proposed Technical Specification changes, based on the unique design of ANO-2, were proposed to be submitted by June 1991.

In letter dated June 18, 1991 (2CAN069107), ANO proposed Technical Specification changes for LTOP in Modes 5 and 6. PORV Technical Specifications for Modes 1, 2, and 3 were determined to be not applicable for ANO-2.

By letter dated November 24, 1992 (0CNA119222), the NRC requested that Entergy Operations provide additional information regarding the ANO-1 GL 90-06 responses within 90 days. The NRC requested that Entergy Operations evaluate the applicability of including the Emergency Core Cooling System (ECCS) Vent Valves in the ANO-2 Technical Specifications.

Entergy Operations letter dated February 5, 1993 (2CAN029301), provided the additional information requested by the NRC. Entergy Operations re-iterated its previous positions on LTOP, PORV applicability, and the design of ANO-2, and in addition, adopted the position that placing the ECCS Vent Valves in the Technical Specifications is not appropriate based on plant design, safety analysis, and procedural use of these valves.

ACTION NEEDED NRC review and closure.

LICENSING ENGINEER Clint Szabo

TAC NO M77399 **UNIT** 2 **PRIORITY** LOW **ACTION** ANO

NAME Generic Letter 90-06 - Low Temperature Overpressure Protection

RELATED TAC # M77327 (ANO-2), M77326 (ANO-1)

STATUS Generic Letter 90-06, Resolution of Generic Issue 70, "Power-Operated Relief valve and Block Valve Reliability," and Generic Issue 94, "Additional Low-Temperature Overpressure Protection for Light-Water Reactors" was issued on June 25, 1990 (OCNA069027) to advise licensees of the NRC Staff positions with respect to resolution of these Generic Issues. The Staff positions are delineated in Enclosures A and B to the Generic Letter. A 180 day response was requested.

Entergy Operations letter dated December 21, 1990 (2CAN129013), provided the ANO-2 response to Generic Letter 90-06. Actions taken and proposed are considered to comply with Generic Letter 90-06. Proposed Technical Specification changes, based on the unique design of ANO-2, were proposed to be submitted by June 1991.

In letter dated June 18, 1991 (2CAN069107), ANO proposed Technical Specification changes for LTOP in Modes 5 and 6. PORV Technical Specifications for Modes 1, 2, and 3 were determined to be not applicable for ANO-2.

A conference call was conducted on December 17, 1991, to discuss initial comments on the TSCR. The specific setpoints have been requested to be placed in the Technical Specifications section instead of the bases. However, no action is currently planned until NRC review of CE Topical Report is complete.

In Entergy Operations' letter dated June 30, 1992 (2CAN069206) ANO also deferred changes to Tech Spec 3.4.2 on Mode 4 overpressure protection until resubmittal of the new LTOP specification.

In January 1994, the NRC advised ANO that the CE Methodology will not be evaluated, as ASME Code Case N-514 methodology is acceptable to the Staff. ANO is evaluating acceptability of code case methodology for possible resubmittal.

ACTION NEEDED ANO to evaluate ASME Code Case N-514 methodology for acceptability.

LICENSING ENGINEER Clint Szabo

TAC NO M82550 **UNIT** 2 **PRIORITY** HIGH **ACTION** NRC

NAME Electrical Circuitry Isolation Requirements

RELATED TAC #

STATUS During the EDSFI at ANO-2, the inspection team raised a concern regarding circuitry isolation, The NRC Staff has reviewed this concern and safety evaluation in letter dated June 18, 1992 (2CNA069201).

The Staff concluded that the existing design does provide for adequate protection against credible faults in non-Class 1E loads from affecting Class 1E station battery power supplies.

The Staff also concluded that the design basis calculation for the coordination of containment penetration overcurrent protection devices to protect the electrical conductor from damage indicates several instances in which no protection exists for overload conditions.

Therefore, as part of the June 18, 1992, letter, the Staff requested Entergy Operations to provide its plans and schedules for the resolution of two items.

Entergy Operations provided the ANO-2 response to the NRC request for additional information on September 4, 1992 (2CAN099202). In this response, the ANO-2 design basis for overload protection was discussed and potential enhancements (beyond design basis) were discussed.

Additional information was also provided informally to the NRC for overload protection of the 125 VDC and 120 VAC circuits where complete coordination could not be provided.

In letter dated February 5, 1993 (2CNA029304), the NRC requested additional information regarding changes to the ANO-2 Reactor Building calculations and detailed primary and secondary overload protection.

On April 30, 1993 Entergy Operations responded by stating that the specific information requested by the Staff could not be readily provided. A revised scope was proposed which would be submitted by August 20, 1993.

The revised calculation was submitted by Entergy Operations (2CAN089304) on August 20, 1993.

ACTION NEEDED NRC review and closure.

LICENSING ENGINEER David McNeil

TAC NO M83430 **UNIT** 2 **PRIORITY** LOW **ACTION** NRC

NAME GL 92-01 Reactor Vessel Structural Integrity

RELATED TAC # M83730 (ANO-1)

STATUS Due to concerns raised during Staff's review of reactor vessel integrity for the Yankee Nuclear Power Station, the NRC issued Generic Letter (GL) 92-01, dated February 28, 1992. The purpose of the GL is for the Staff to obtain information needed to assess compliance with requirements and commitments regarding reactor vessel integrity.

GL 92-01 specifically requires each licensee to provide certain requested information for evaluation to ensure continued compliance with 10CFR50.60 and 50.61, as well as continued fulfillment of commitments made in response to GL 88-11. The requested information for Arkansas Nuclear One, Units 1 and 2 (ANO-1 & 2) was submitted to the Staff on July 1, 1992, in letter 0CAN079201.

Combustion Engineering (CE) was contracted by Entergy Operations to compile the information requested by GL 92-01 for ANO-2. As a result, CE prepared report A-MECH-ER-005 that provides the ANO-2 responses to the GL-92-01 requests.

The information contained in A-MECH-ER-005 is considered by Entergy Operations to be a complete and satisfactory response to the required information section of GL 92-01.

ANO received RAIs concerning our response to GL 92-01 in NRC letter 0CNA089319 on August 31, 1993. ANO responded to these RAIs in letter 0CAN109305 dated October 29, 1993.

ACTION NEEDED NRC review and closure.

LICENSING ENGINEER Jim Haley

TAC NO M83588 **UNIT** 1 **PRIORITY** LOW **ACTION** ANO

NAME IPEEE for External Events (GL 88-20 Supplement 4)

RELATED TAC # M69426 (ANO-1 A-46)

STATUS Supplement 4 (IPEEE) to the Generic Letter was issued on June 28, 1991. This supplement required a response that describes the ANO proposed program within 180 days of receipt.

In letter OCAN129101. dated December 19, 1991, ANO committed to submit a schedule within 120 days following receipt of the SQUG GIP SSER.

The GIP-2 SSER was received on May 22, 1992.

Entergy Operations submitted letter 0CAN099201, dated September 18, 1992 committing to perform our IPEEE during 1R11 with completion by May 1995.

By NRC letter dated August 24, 1993 (0CNA089313), the Staff accepted ANO's schedule for completion of the IPEEE effort by May 1995.

ACTION NEEDED Entergy Operations to submit ANO-1 IPEEE by May 1995.

LICENSING ENGINEER Natalie Mosher

TAC NO M83589 **UNIT** 2 **PRIORITY** LOW **ACTION** ANO

NAME IPEEE for External Events (GL 88-20 Supplement 4)

RELATED TAC # M69427 (ANO-2 A-46)

STATUS Supplement 4 (IPEEE) to the Generic Letter was issued on June 28, 1991. This supplement required a response that describes the ANO proposed IPEEE program within 180 days of receipt.

In letter OCAN129101, dated December 19, 1991, ANO committed to submit a schedule within 120 days following receipt of the SQUG GIP SSER.

The GIP-2 SSER was received on May 22, 1992.

Entergy Operations submitted letter 0CAN099201, dated September 18, 1992 committing to perform our IPEEE during 2R10 with completion by May 1995.

By NRC letter dated August 24, 1993 (0CNA089313), the Staff accepted ANO's schedule for completion of the IPEEE effort by May 1995.

ACTION NEEDED Entergy Operations to submit ANO-2 IPEEE by May 1995.

LICENSING ENGINEER Natalie Mosher

TAC NO M83730 **UNIT** 1 **PRIORITY** LOW **ACTION** NRC

NAME GL 92-01 Reactor Vessel Structural Integrity

RELATED TAC # M83430 (ANO-2)

STATUS Due to concerns raised during Staff's review of reactor vessel integrity for the Yankee Nuclear Power Station, the NRC issued Generic Letter (GL) 92-01 dated February 28, 1992. The purpose of the GL is for the Staff to obtain information needed to assess compliance with requirements and commitments regarding reactor vessel integrity.

GL 92-01 specifically requires each licensee to provide certain requested information for evaluation to ensure continued compliance with 10CFR5O.60 and 50.61, as well as continued fulfillment of commitments made in response to GL 88-11. The requested information for Arkansas Nuclear One, Units 1 and 2 (ANO-1 & 2) was submitted to the Staff on July 1, 1992 in letter OCAN079201.

ANO-1 is a charter member of the Babcock & Wilcox (B&W) Owners Group Reactor Vessel Working Group. This group has prepared a single document for NRC review that provides responses to the GL requests for all members of the Reactor Vessel Working Group, including ANO-1. That document, BAW-2166 dated June 1992, was submitted directly to the NRC by B&W, on behalf of the Owners Group, via letter OG-1036 dated June 17, 1992. Entergy Operations confirms that the ANO-1 related information contained in BAW-2166 is correct.

The information contained in BAW-2166 for ANO-1 is considered by Entergy Operations to be a complete and satisfactory response to the required Information section of GL 92-01.

ANO received RAIs concerning our response to GL 92-01 in NRC letter 0CNA089319 on August 31, 1993. ANO responded to these RAIs in letter 0CAN109305 dated October 29, 1993.

ACTION NEEDED NRC review and closure.

LICENSING ENGINEER Jim Haley

TAC NO M85352 **UNIT** 1 **PRIORITY** LOW **ACTION** ANO

NAME Response to NRCB 90-01, Supplement 1, Loss of Fill-Oil in Rosemount Transmitters

RELATED TAC # M85353 (ANO-2)

STATUS In the original NRCB 90-01, the Staff requested that licensees take action to replace or monitor Rosemount Model 1153B, 1153D and 1154 Transmitters of certain high failure rate lots manufactured with a date code prior to July 11, 1989. Supplement 1 to the Bulletin expanded the scope to consider the same model transmitters and date code but to monitor or replace based on pressure and time in service.

Letter 0CAN039302, dated March 5, 1993, provided the ANO response to NRCB 90-01, Supplement 1, which included a schedule for actions to be completed by ANO. Actions are currently being performed to identify and evaluate transmitters which may require enhanced monitoring frequency to comply with NRCB 90-01 Supplement 1. Those transmitters which are not accessible for enhanced monitoring during normal operations will be considered for replacement.

Actions discussed in 0CAN039302 are being completed. Several transmitters have been identified which cannot meet the supplement's monthly or quarterly surveillance frequency. These transmitters are being evaluated for replacement. Data from 1R11 are being included in this evaluation.

ACTION NEEDED ANO to provide response when actions are completed following 1R12.

LICENSING ENGINEER Jim Haley

TAC NO M85353 **UNIT** 2 **PRIORITY** MED **ACTION** ANO

NAME Response to NRCB 90-01, Supplement 1, Loss of Fill-Oil in Rosemount Transmitters

RELATED TAC # M85352 (ANO-1)

STATUS In the original NRCB 90-01, the Staff requested that licensees take action to replace or monitor Rosemount Model 1153B, 1153D and 1154 Transmitters of certain high failure rate lots manufactured with a date code prior to July 11, 1989. Supplement 1 to the Bulietin expanded the scope to consider the same model transmitters and date code but to monitor or replace based on pressure and time in service.

Letter 0CAN039302, dated March 5, 1993, provided the ANO response to NRCB 90-01, Supplement 1, which included a schedule for actions to be completed by ANO. Actions are currently being performed to identify and evaluate transmitters which may require enhanced monitoring frequency to comply with NRCB 90-01 Supplement 1. Those transmitters which are not accessible for enhanced monitoring during normal operations will be considered for replacement.

Several transmitters have been identified for replacement during the next refueling outage.

ACTION NEEDED ANO to provide response when actions are completed following 1R12.

LICENSING ENGINEER Jim Haley

TAC NO M85515 **UNIT** 2 **PRIORITY** HIGH **ACTION** ANO

NAME Thermo-Lag (GL 92-08)

RELATED TAC #

STATUS On December 17, 1992, the NRC issued GL 92-08. The generic letter was issued to obtain additional information from licensees to verify that Thermo-Lag 330-1 fire barrier systems comply with NRC's requirements. The NRC requested licensees to confirm that the Thermo-Lag 330-1 barrier systems have been qualified by representative fire endurance tests, that the ampacity derating factors have been derived by valid tests, and that these qualified barriers have been installed with appropriated procedures and quality controls to ensures that they comply with the NRC's requirements.

Entergy Operations responded on April 16, 1993.

By letter dated December 21, 1993 (2CNA129304), the NRC requested additional information to supplement ANO-2's response to the GL.

Entergy Operations has reviewed the initial determination of the use of thermo-lag and concluded that none of the installed material is required for Appendix R compliance for ANO-2.

ACTION NEEDED Entergy Operations to submit 50.54 ANO-2 response by February 19, 1994.

LICENSING ENGINEER John Dosa

TAC NO M85871 **UNIT** 2 **PRIORITY** MED **ACTION** NRC

NAME Containment Internal Pressure Revision

RELATED TAC #

STATUS Entergy Operations submitted 2CAN029305 on February 24, 1993 to revise containment temperature/pressure curve to be consistent with new large break LOCA analysis which supports 10% steam generator tube plugging.

ACTION NEEDED NRC review and approval.

LICENSING ENGINEER Natalie Mosher

TAC NO M86005 **UNIT** 1 **PRIORITY** LOW **ACTION** NRC

NAME Change to Correct Typographical Errors Throughout TS

RELATED TAC #

STATUS Entergy Operations submitted 1CAN029303 on February 24, 1993 to correct typographical errors in the Unit 1 TS.

ACTION NEEDED NRC review and approval.

LICENSING ENGINEER David McNeil

TAC NO M86264 **UNIT** 1 **PRIORITY** LOW **ACTION** NRC

NAME Technical Specification Change Concerning C-3 Inspections of Steam Generator Tubes

RELATED TAC #

STATUS Footnote 2 to ANO-1 TS Table 4.18-2, Steam Generator Tube Inspection, requires, in part, that for C-3 inspection results, NRC approval of a remedial action plan must be obtained. This requirement was discussed with the ANO-1 NRR Project Manager and it was recommended that Entergy Operations should propose deleting this requirement. The requirement is not a condition of the Standardized Technical Specifications and apparently is no longer considered to be necessary by the NRC Staff. Entergy Operations proposed this change in letter 1CAN039302 dated March 19, 1993.

ACTION NEEDED NRC review and approval.

LICENSING ENGINEER John Dosa

TAC NO M86468 **UNIT** 1 **PRIORITY** LOW **ACTION** NRC

NAME Review of ODCM

RELATED TAC #

STATUS The Semi-Annual Radiological Effluent Release Report for the third and fourth quarters of 1992 submitted on February 26, 1993 (0CAN029308) included Revision 3 to the Offsite Dose Calculation Manual (ODCM). This revision involved changes to two sampling stations.

ACTION NEEDED NRC review.

LICENSING ENGINEER John Dosa

TAC NO M86469 **UNIT** 2 **PRIORITY** LOW **ACTION** NRC

NAME Review of ODCM

RELATED TAC #

STATUS The Semi-Annual Radiological Effluent Release Report for the third and fourth quarters of 1992 submitted on February 26, 1993 (0CAN029308) included Revision 3 to the Offsite Dose Calculation Manual (ODCM). This revision involved changes to two sampling stations.

ACTION NEEDED NRC review.

LICENSING ENGINEER John Dosa

TAC NO M86946 **UNIT** 1 **PRIORITY** LOW **ACTION** ANO

NAME ERV Cycling During Plant Runback

RELATED TAC #

STATUS Entergy Operations submitted 1CAN089304 on August 16, 1993 to document ANO's evaluation of the concern regarding the manual use of the ERV to avert a high pressure trip given the increased potential for a SBLOCA caused by a stuck open ERV.

ANO to make presentation to NRC.

ACTION NEEDED ANO presentation to NRC on February 8, 1994.

LICENSING ENGINEER Natalie Mosher

TAC NO M87066 **UNIT** 2 **PRIORITY** MED **ACTION** NRC

NAME Remove Cycle Specific Parameters per GL 88-16

RELATED TAC #

STATUS Entergy Operations submitted 2CAN079301 on July 22, 1993 to remove cycle-specific parameters from TS and place them into a Core Operating Limits Report (COLR) per Generic Letter 88-16. The definition of shutdown margin is also being revised.

ANO provided additional information on October 20, 1993 (2CAN109305).

ACTION NEEDED NRC approval prior to 2R10 completion.

LICENSING ENGINEER Natalie Mosher

TAC NO M87147 **UNIT** 2 **PRIORITY** LOW **ACTION** NRC

NAME Indefinite Bypass of PPS Channel

RELATED TAC #

STATUS Entergy Operations submitted a TS change on July 22, 1993 to allow reactor protection system or engineered safety features actuation system channels to be bypassed until the next cold shutdown condition. The change requires the decision to leave a channel in bypass to be reviewed a the next scheduled meeting of the Plant Safety Committee. The Engineering Report which provides the justification for the TS change was included with the submittal. The NRC reviewer is making an initial review to identify questions which can be addressed in a meeting sometime in February- March 1994 time frame.

ACTION NEEDED NRC technical reviewer to supply questions on submittal to focus meeting subject.

LICENSING ENGINEER Glenn Ashley

TAC NO M87243 **UNIT** 2 **PRIORITY** LOW **ACTION** ANO

NAME Shutdown Cooling Line Thermal Stratification

RELATED TAC #

STATUS Entergy Operations submitted a voluntary response on August 17, 1993 (2CAN089301), concerning thermal stratification in the shutdown cooling line. ANO will provide a format revision to the stress report for the shutdown cooling line once the thermal transients are better understood.

ACTION NEEDED Entergy Operations to provide revised stress report.

LICENSING ENGINEER Jim Haley

TAC NO M87263 **UNIT** 1 **PRIORITY** LOW **ACTION** ANO

NAME Exemption from 10CFR26 - Fitness For Duty

RELATED TAC # M87264

STATUS Entergy Operations submitted CNRO-93/00027 on August 20, 1993 to request exemption from 10CFR26, "Fitness For Duty".

Rule approved by Commission on December 21, 1993 to be effective January 1, 1994.

ACTION NEEDED ANO to withdraw request.

LICENSING ENGINEER Natalie Mosher

TAC NO M87264 **UNIT** 2 **PRIORITY** LOW **ACTION** ANO

NAME Exemption from 10CFR26 - Fitness For Duty

RELATED TAC # M87263

STATUS Entergy Operations submitted CNRO-93/00027 on August 20, 1993 to request exemption from 10CFR26, "Fitness For Duty".

Rule approved by Commission on December 21, 1993 to be effective January 1, 1994.

ACTION NEEDED ANO to withdraw request.

LICENSING ENGINEER Natalie Mosher

TAC NO M87445 **UNIT** 1 **PRIORITY** LOW **ACTION** NRC

NAME Containment Net Free Volume Correction

RELATED TAC #

STATUS Entergy Operations submitted change on July 22, 1993 to correct the value for "net free volume" given in the ANO-1 TS 5.3.1 to reflect the appropriate value.

ACTION NEEDED NRC review and approval.

LICENSING ENGINEER Clint Szabo

TAC NO M87718 **UNIT** 1 **PRIORITY** LOW **ACTION** ANO

NAME Proposed Revision to the ISP

RELATED TAC # M87719

STATUS Entergy Operations submitted a proposed revision to the ISP on September 4, 1993 (OCAN099301), as a part of the regulatory burden reduction program to add flexibility to determine the optimal security compensatory measures based on plant activities and the status of the overall security program. Subsequent conversations with Entergy Operations has identified the need to supplement the original submittal to provide additional clarification. This supplement will be transmitted to the NRC by early February 1994.

ACTION NEEDED Entergy Operations to supplement original response.

LICENSING ENGINEER Natalie Mosher

TAC NO M87719 **UNIT** 2 **PRIORITY** LOW **ACTION** ANO

NAME Proposed Revision to the ISP

RELATED TAC # M87718

STATUS Entergy Operations submitted a proposed revision to the ISP on September 4, 1993 (OCAN099301), as a part of the regulatory burden reduction program to add flexibility to determine the optimal security compensatory measures based on plant activities and the status of the overall security program. Subsequent conversations with Entergy Operations has identified the need to supplement the original submittal to provide additional clarification. This supplement will be transmitted to the NRC by early February 1994.

ACTION NEEDED Entergy Operations to supplement original response.

LICENSING ENGINEER Natalie Mosher

TAC NO M87829 **UNIT** 1 **PRIORITY** LOW **ACTION** NRC

NAME Review One Year Response to NRC SE for Second 10-Year IST Program

RELATED TAC #

STATUS Entergy Operations received the NRC SE for the second 10 year pump and valve IST program at ANO-1 by letter dated September 21, 1992 (1CNA099203). The SE and the attached TER documented the review of the relief requests submitted by Entergy Operations and requested a 90 day response providing additional justification for the requests. This information was submitted by letter dated December 18, 1992 (1CAN129202). The SE and the TER also requested follow-up responses within one year to describe the processes used in developing the ANO-1 IST program and to address anomalies in the program. The one year response and a complete program resubmittal were made on September 21, 1993 (1CAN099304).

ACTION NEEDED NRC review and closure.

LICENSING ENGINEER John Dosa

TAC NO M87912 **UNIT** 1 **PRIORITY** MED **ACTION** NRC

NAME Inaccuracy of MOV Diagnostic Equipment

RELATED TAC # M87913

STATUS Entergy Operations received GL 89-10 Supplement 5 on July 7, 1993 (0CNA069319). The ANO response (0CAN099303) dated September 30, 1993 contained detailed descriptions of how ANO had addressed previously identified inaccuracies in MOV diagnostic equipment and identified several planned actions which are in the process of being completed.

ACTION NEEDED NRC review and closure.

LICENSING ENGINEER Jim Haley

TAC NO M87913 **UNIT** 2 **PRIORITY** MED **ACTION** NRC

NAME Inaccuracy of MOV Diagnostic Equipment

RELATED TAC # M87912

STATUS Entergy Operations received GL 89-10 Supplement 5 on July 7, 1993 (0CNA069319). The ANO response (0CAN099303) dated September 30, 1993 contained detailed descriptions of how ANO had addressed previously identified inaccuracies in MOV diagnostic equipment and identified several planned actions which are in the process of being completed.

ACTION NEEDED NRC review and closure.

LICENSING ENGINEER Jim Haley

TAC NO M88074 **UNIT** 1 **PRIORITY** MED **ACTION** NRC

NAME Proposed Alternative to 10CFR50.55A 10-Year IST/IST Program

RELATED TAC # M88075

STATUS Entergy Operations submitted CNRO-93/00032 on October 21, 1993, to request an alternative to the mandatory 10-year program updates for ISI/IST programs to the most recent NRC approved version of the ASME Section XI Code. The alternative would require updates only when Code revisions were determined to be safety significant.

ACTION NEEDED NRC review and acceptance.

LICENSING ENGINEER John Dosa

TAC NO M88075 **UNIT** 2 **PRIORITY** MED **ACTION** NRC

NAME Proposed Alternative to 10CFR50.55A 10-Year IST/IST Program

RELATED TAC # M88074

STATUS Entergy Operations submitted CNRO-93/00032 on October 21, 1993, to request an alternative to the mandatory 10-year program updates for ISI/IST programs to the most recent NRC approved version of the ASME Section XI Code. The alternative would require updates only when Code revisions were determined to be safety significant.

ACTION NEEDED NRC review and acceptance.

LICENSING ENGINEER John Dosa

TAC NO M88080 **UNIT** 1 **PRIORITY** MED **ACTION** NRC

NAME Relief Request for Successive Inspections of RCP Weld Flaws

RELATED TAC #

STATUS Entergy Operations submitted 1CAN099302 on September 28, 1993 to request relief from the requirement of IWB-2420(b) of the ASME Boiler and Pressure Vessel Code, Section XI, 1980 Edition through Winter 1981 Addenda, Rules for ISI of Nuclear Power Plant Components. The specific request is for relief (per 10CFR50.55a(g)(5)) from the requirement to perform additional successive weld examinations on the ANO-1 "A" and "B" RCP casing welds.

ACTION NEEDED NRC review and approval.

LICENSING ENGINEER John Dosa

TAC NO M88110 **UNIT** 2 **PRIORITY** LOW **ACTION** NRC

NAME Incore Detector System TS Change Request

RELATED TAC #

STATUS Due to an abnormally high incore detector failure rate during ANO-2 fuel cycle 10, Entergy Operations requested and was granted an exigent TS amendment (Amendment No. 151 dated October 20, 1993) reducing the number of incore detectors and detector locations for the remainder of the fuel cycle. Entergy Operations submitted 2CAN109303 on October 27, 1993 to relocate the requirements for the incore detector system from the TS to the SAR, in order to prevent a need for a similar TS change in the future. This change is in accordance with NUREG 1432 "CE Restructured Standard Technical Specifications".

ACTION NEEDED NRC review and approval.

LICENSING ENGINEER Clint Szabo

TAC NO M88190 **UNIT** 2 **PRIORITY** LOW **ACTION** NRC

NAME ISI Relief Request B-B/3.1

RELATED TAC #

STATUS By letter dated December 30, 1992 (2CNA129205), the Staff provided its evaluation of the ANO-2 first 10-year interval ISI relief requests. The NRC determined that relief was appropriate or not necessary for 12 of 15 relief requests. However, in 3 cases (B-B/3.1, C-E-1/C2.5, and part of B-J/B4.5) relief was denied. For these 3 cases, ANO was given the option of performing the appropriate inspections at the next scheduled outage or resubmitting the requests with additional information.

By letter dated November 3, 1993 (2CAN119302), Entergy Operations resubmitted B-B/3.1 with additional supporting information.

ACTION NEEDED NRC review and acceptance prior to 2R10.

LICENSING ENGINEER John Dosa

TAC NO **UNIT** 1 **PRIORITY** **ACTION** NRC

NAME TS Change Request Allowing Operation of the RPS with Inoperable Functions in One Channel in the Untripped and Unbypassed State

RELATED TAC #

STATUS Entergy Operations submitted this change request on January 13, 1994 to allow flexibility in the operation of the ANO-1 RPS. This change allows operation of the RPS with inoperable untripped functions in the unbypassed state as equivalent to placing the RPS channel in "Channel Bypass" for only the failed functions.

ACTION NEEDED NRC review and approval.

LICENSING ENGINEER Clint Szabo

TAC NO **UNIT** 2 **PRIORITY** **ACTION** NRC

NAME Review One Year Response to NRC SE for Second 10-Year IST Program

RELATED TAC #

STATUS Entergy Operations received the NRC SE for the second 10 year pump and valve IST program at ANO-2 by letter dated January 22, 1993 (2CNA019304). The SE and the attached TER documented the review of the relief requests submitted by Entergy Operations and requested follow-up responses within one year to describe the processes used in developing the ANO-2 IST program and to address anomalies in the program. The one year response and a complete program resubmittal (including 5 new relief requests) were made on January 21, 1994 (2CAN019402).

ACTION NEEDED NRC review and closure.

LICENSING ENGINEER John Dosa

TAC NO **UNIT** 1 **PRIORITY** **ACTION** ANO

NAME TS Change to Allow Handling of Spent Fuel Shipping Casks Using the Auxiliary Building Crane

RELATED TAC #

STATUS ANO-1 Technical Specification (TS) 3.8.15 currently prohibits handling of spent fuel shipping casks using the auxiliary building crane. ANO-1 implemented this requirement in TS amendment 17 with the stipulation that the requirement be deleted following NRC approval of the crane design and the spent fuel cask drop analysis. This industry issue was originally addressed by the NRC in Generic Issue A-36 and NUREG-0612 "Control of Heavy Loads at Nuclear Power Plants". NUREG -0612 required licensees to review this issue and submit their findings in two phases. ANO-1 completion of these requirements is documented in NRC correspondence dated October 11, 1984 (0CNA108406) and generic letter 85-11. A Technical Specification Change Request has been drafted to delete ANO-1 TS 3.8.15. This requests has been distributed for concurrence review and is expected to be submitted to the NRC immediately following a special SRC meeting to be held sometime in February.

ACTION NEEDED ANO to submit in February 1994.

LICENSING ENGINEER David McNeil