

APPENDIX

U.S. NUCLEAR REGULATORY COMMISSION

REGION IV

Report: 50-298/82-13

Docket: 50-298

Licensee: Nebraska Public Power District
P. O. Box 499
Columbus, Nebraska 68601

Facility Name: Cooper Nuclear Station

Inspection At: Cooper Nuclear Station, Nemaha County, Nebraska
May 1-31, 1982

Inspector:

D. L. DuBois
D. L. DuBois, Resident Reactor
Inspector, Reactor Project Section A

6/18/82
Date

Approved:

T. F. Westerman
T. F. Westerman, Chief
Reactor Project Section A

6/28/82
Date

Inspection Summary

Inspection on May 1-31, 1982 (Report 50-298/82-13)

Areas Inspected: Routine, announced inspection of operational safety verifications, monthly equipment surveillance and maintenance observations, followup on licensee events, and preparation for refueling. This inspection involved 66 inspector-hours onsite by one NRC inspector.

Results: Within the areas inspected no violations or deviations were identified.

DETAILS1. Persons Contacted

*L. Lessor, Plant Superintendent
P. Thomason, Acting Operations Supervisor
L. Lawrence, Maintenance Supervisor
R. McDonald, Acting C & HP Supervisor
H. Jantzen, I & C Supervisor
V. Wolstenholm, QA Supervisor
D. Majeres, Maintenance Planner
R. Peterson, Reactor Engineer
L. Bednar, Electrical Engineer

*Indicates presence at exit meetings.

2. Operational Safety Verification

The NRC inspector observed control room operations, instrumentation, controls, reviewed applicable logs, and conducted discussions with control room operators. The inspector verified operability of:

Reactor Protection System
Power Distribution System
Automatic Depressurization System
'B' Core Spray System
No. 1 Diesel Generator
Standby Liquid Control System
Fire Protection System

The inspector reviewed safety clearance records, including verification that affected components were removed from and returned to service in a correct and approved manner, that redundant equipment was verified operable, and that limiting conditions for operation were adequately identified and maintained. The inspector also verified that maintenance requests had been initiated for equipment discovered to require repair or routine preventative upkeep, appropriate priority was assigned, and maintenance commenced in a timely manner commensurate with assigned priorities.

Tours of accessible areas of the facility were conducted to observe normal security practices, plant and equipment conditions including cleanliness, radiological controls, fire suppression systems, emergency equipment, potential fire hazards, fluid leaks, excessive vibration and instrumentation adequacy.

These reviews and observations were conducted to verify that facility operations were in conformance with the requirements established in the Technical Specification, 10 CFR, and Administrative Procedures.

No violations or deviations were identified in these areas.

3. Surveillance Observations

The inspector observed portions of Technical Specification required surveillance tests to verify that testing was performed in accordance with adequate procedures, test instrumentation was in calibration, limiting conditions for operations were met, removal and subsequent restoration of affected components was accomplished, test results conformed with Technical Specification and procedure requirements, tests were reviewed by personnel other than the person directing the test, and deficiencies identified during testing were properly reviewed and resolved by appropriate management personnel.

The following Surveillance Tests were selected and observed:

- 6.1.24A Rod Worth Minimizer Functional Test for Shutdown
- 6.1.26A Rod Sequence Control System Functional Test for Shutdown
- 6.2.2.5.11 RHR Crosstie Valve Position Monitor Functional Test
- 6.3.18.1 Service Water Pump Operability Test
- 6.3.18.2 Service Water Motor Operated Valve Operability Test
- STP 82.9 Reactor Recirculation Pump Trip Test

*STP - Special Test Procedure

These reviews and observations were conducted to verify that facility operations were in conformance with the requirements established in the Technical Specification, 10 CFR, and Administrative Procedures.

No violations or deviations were identified in these areas.

4. Maintenance Observations

The inspector observed portions of the following maintenance activities:

- MWR 81-1631, Scram Discharge Volume Modification
- MWR 81-2106, Reactor Protection System, Motor Generator Set Modification
- MWR 82-0659, Lower Control Rod Drive Removal and Replacement
- MWR 82-0947, Disassemble and Reassemble Reactor Vessel and Internals

*MWR - Maintenance Work Request

The following Clearance Orders were independently verified for proper placement/restoration of affected components:

- 82-280, 'B' Condenser Circulating Water Pump
- 82-281, Residual Heat Removal Valve RHR-MO-16A
- 82-282, Main Steam Isolation Valves
- 82-283, High Pressure Coolant Injection Pump
- 82-284, Reactor Core Isolation Cooling Pump

5. Licensee Event Followup

The following LER's are closed on the basis of the inspector's inoffice review,

review of licensee documentation, and discussions with licensee personnel:

- LER 82-01 Misalignment of Reactor Vessel Level Switch NBI-LIS-72C
- LER 82-02 Excessive Setpoint Drift of Reactor Vessel Level Switch NBI-LIS-101A
- LER 82-03 Failure of the Brake Coil for Valve RHR-MO-26B
- LER 82-04 Operation With the Indicated Minimum Critical Power Ratio (MCPR) Less Than the MCPR Operating Limit
- LER 82-05 Main Steam Isolation Valve 86A Closing Time Faster Than Technical Specification Limit

6. Preparation for Refueling

The inspector reviewed the following documents:

- a. Approved procedures applicable to the receipt, inspection, transfer, and storage of new fuel assemblies
- b. Approved procedures applicable to refueling, core shuffle operations, and core verifications following the completion of refueling activities
- c. Approved procedures applicable to the removal and inspection of reactor pressure vessel internals
- d. New fuel shipping and receiving documents
- e. Fuel accountability records
- f. Health physics radiation and survey records related to receipt and inspection of new fuel
- g. The licensee's overall refueling outage schedule, plans, and controls

The inspector verified that the licensee had submitted proposed changes to Appendix A and B of the Technical Specification. The proposed changes would permit future reload analysis to be accomplished under the provisions of 10 CFR 50.59 (reference a letter from Pilant to Vassello, dated April 30, 1982).

The inspector observed the receipt, handling, inspection, and storage of three new fuel assemblies. He also performed a verification of the new fuel assemblies storage location in the spent fuel pool.

These reviews and observations were conducted to verify that new fuel handling procedures, fuel accountability records, fuel shipping documents, and radiation survey records were complete and accurate; that licensee personnel performed new fuel handling activities in accordance with approved procedures, and that the licensee had submitted a Technical Specification change relevant to core reload.

No violations or deviations were identified in these areas.

7. Exit Meetings

Exit meetings were conducted at the conclusion of each portion of the inspection. The Plant Superintendent was informed of the above findings.