

Patrick M Donnelly Plans Manager

Big Rock Point Nuclear Plant, 10269 US-31 North, Charlevoix, MI 49720

February 1, 1994

No. Regulatory Commission Document Control Desk Washington, DC 20555

Dear Sir:

Enclosed please find the statistical data for the Big Rock Point Nuclear Plant cover: be period of January 1, 1994 through January 31, 1994.

Sincerely,

P M Donnelly Plant Manager

Enclosures

cc: Admin.strator Region III, Nuclear Regulatory Commission DRHahn, Department of Public Heath JRPadgett, Michigan Public Service Commission RAben, Michigan Department of Labor MPCass, American Nuclear Insurers FYost, Research Services Utility Data Institute INPO Records Center NRC Resident Inspector Document Control, Big Rock Point, 740/22*35*10 DPHoffman, P24-117A GCWithrow, Big Rock Point File

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JE24



NUCLEAR OPENATIONS DEPARTMENT Unit Shutdowns and Power Reductions

Report Month	Docket Number	Unit	Date	Completed by	Telephone
January, 1994	55-150	Big Bock Point Plant	February 1, 1994	JR JOHNSTON	(616) 547-8223

umber	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor	Licensee Event Report Number	System Code ⁴	Component Code ⁵	Cause and Corrective Action To Prevent Recurrence
						NONE -			

1F = Forced S = Scheduled 2_{Reason:}

A = Equipment Failure (Explain)

B = Maintenance of Test

C = Refueling

D = Regulatory Restriction

E = Operator Training and License Examination

F = Administrative

G = Operational Error (Explain)

H = Other (Explain)

3_{Method}:

1 = Manual

2 = Manual Scram

3 = Automatic Scram

4 = Other (Explain)

⁴Exhibit G = Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵Exhibit I = Same Source

GREYBOOK OPERATING DATA REPORT

DOCKET NO. 50-155 DATE: 2 / 1 / 94

BY: JR JOHNSTON

OPERATING STATUS

NOTES

PHONE: 616-547-6537, EXT 223

1. UNIT NAME: BIG ROCK POINT NUCLEAR PLANT

2. REPORTING PERIOD: 1 / 94

3. LICENSED THERMAL POWER (MWT): 240

4. NAMEPLATE RATING (GROSS MWE): 75

5. DESIGN ELECTRICAL RATING (NET MWE): 72

6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 71.0

7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 67.0

B. IF CHANGES OCCUR IN CAPACITY RATINGS(ITEMS 3 THRU 7) SINCE LAST REPORT, GIVE REASONS:

- 9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE):
- 10. REASONS FOR RESTRICTIONS, IF ANY.

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD 12. NUMBER OF HOURS REACTOR WAS CRITICAL 13. REACTOR RESERVE SHUTDOWN HOURS 14. HOURS GENERATOR ON-LINE 15. UNIT RESERVE SHUTDOWN HOURS 16. GROSS THERMAL ENERGY GENERATED (MWH) 17. GROSS ELECTRICAL ENERGY GENERATED (MWH) 18. NET ELECTRICAL ENERGY GENERATED (MWH) 19. UNIT SERV/CE FACTOR 20. UNIT AVAILABILITY FACTOR 21. UNIT CAPACITY FACTOR (USING MDC NCT) 22. UNIT CAPACITY FACTOR (USING DER NET)	744.0 744.0 0.0 744.0 0.0 170664.0 54397.0 51662.7 100.0% 100.0% 103.6% 96.4%	744.0 744.0 0.0 744.0 0.0 170664.0 54397.0 51662.7 100.0% 100.0% 103.6% 96.4%	270403.0 195862.4 0.0 192823.2 0.0 37073655.0 11776512.0 11140922.7 71.3% 71.3% 61.2% 57.2%
23. UNIT FORCED DUTAGE RATE	0.0%	0.0%	11.2%

- 24. SHUTODWAS SCHEDULED OVER NEXT 6 MONTHS(TYPE, DATE, & DURATION OF EACH):
- 25. IF SHIT DOWN AT END OF REPORT Pt. 100, ESTIMATEDDATE OF STARTUP:

RUNTIME=02/01/94 14:05:34	- CYCLE 27																															
3-001 14/03/93	(MMEN) (1/94)	69.66		Stat. 303											69.56										00 69			69.43		68.90		69.28
STATS VERSION 93.001; EA-8-STATS-93-001	AVERAGE DAILY POWER(MWT)	228.75	0.0	133	53	35	83	83	7.1	230.25	229.08	228.38	229.96	230.54	230.08	228.54	230.88	230.13	229.75	228.83	229.25	230.25	229.33	17.622	228.88	220.17	528.29	230.54	228.58	228.46	.30,13	229.54
1 STATS	DAY		2	445	***	un.	8	7	00	đi	10	111	12	13	10.00	125	16	17	18	13	27.	653	523	23	24	52	92	12	28	53	30	31

Refueling Information Request

- 1. Facility Name: Big Rock Point Plant
- 2. Scheduled date for next refueling shutdown: September, 1994.
- 3. Scheduled date for restart following shutdown: November, 1994.
- 4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? No

If yes, explain:

If no, has the reload fuel design and core enfiguration been reviewed by Plant Safety Review Committee to determine whether any unreviewed safety questions as associated with the core reload (Reference 10 CFR, Section 50.59)? Yes

If no review has taken place, when is it scheduled?

- 5. Scheduled date(s) for submittal of proposed licensing action and supporting information:
- 6. Important licensing considerations associated with refueling, eg. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design new operating procedures:
- 7. Number of fuel assemblies in: core 84; spent fuel pool storage 316; new fuel storage 0.
- 8. Present licensed spent fuel pool storage capacity: 441
 Size of any increase in licensed storage capacity that has been requested or planned (in number of fuel assemblies): 0
- Projected date of last refueling that can be discharged to spent fuel pool assuming the present license capacity: Last total core off load 1996; Last refueling 1999.

REFUELING.DO6 January 3, 1994