NRC FORM 366
(7-77)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | A L B R F 2 (2) | 0 0 - 0 0 0 0 0 - 0 0 (3) | 4 1 1 1 1 (4) | (5)
LICENSEE CODE | LICENSE NUMBER | LICENSE TYPE | CAT

CON'T
0 1 | REPORT SOURCE L (6) | 0 5 0 0 0 2 6 0 (7) | 0 7 2 0 8 2 (8) | 0 8 1 2 8 2 (9)
DOCKET NUMBER | EVENT DATE | REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During normal operation, while performing SI 4.7.D.1.B-1(A). (H_2O_2 sys. isol. vlv.
0 3 operability) "B" hydrogen analyzer became inoperable (T.S. 3.7.H.2) due to the failure
0 4 of the torus hydrogen sample valve 2-FSV-76-65. There was no effect on public
0 5 health and safety. "A" hydrogen analyzer was available and operable. T.S. 3.7.H.2
0 6 permits operation for 30 days with one hydrogen analyzer inoperable. "B" hydrogen
0 7 analyzer was inoperable for about 10 days, until the refueling outage began.
0 8

SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
S E (11)	E (12)	A (13)	V A L V O P (14)	E (15)	Z (16)

(17) LER/RO REPORT NUMBER

EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
8 2	—	0 2 1	/	0 3	L	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
A (18)	D (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	Y (24)	L (25)	V 0 3 0 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Valcor Engineering Corporation Solenoid Operator P/N V52630-529-1 failed. The
1 1 solenoid assembly was replaced, but the solenoid valve failed again while
1 2 performing SI 4.7.D.1.B.1-(A). The valve will be repaired during the refueling
1 3 outage. These failures will be investigated further, with results expected by
1 4 1/1/83.

	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
1 5	E (28)	0 7 9 (29)	NA	B (31)	Surveillance testing

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
1 6	Z (33)	Z (34)	NA	NA

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION (39)
1 7	0 0 0 (37)	Z (38)	NA

PERSONNEL INJURIES

	NUMBER	DESCRIPTION (41)
1 8	0 0 0 (40)	NA

LOSS OF OR DAMAGE TO FACILITY (43)

	TYPE	DESCRIPTION
1 9	Z (42)	NA

8208270291 820812
PDR ADOCK 05000260
S PDR

PUBLICITY

	ISSUED	DESCRIPTION (45)	NRC USE ONLY
2 0	N (44)		

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Tennessee Valley Authority
Browns Ferry Nuclear Plant

Form BF 17
BF 15.2
2/12/82

LER SUPPLEMENTAL INFORMATION

BFRO-50- 260 / 82021 Technical Specification Involved 3.7.H.2

Reported Under Technical Specification 6.7.2.b.(2)* Date Due NRC 8/19/82

Event Narrative:

Unit 1 was operating at 89-percent power and unit 3 was operating at 96-percent power. Both units were unaffected by this event. With unit 2 operating at 79-percent power, during the performance of Surveillance Instruction (SI) 4.7.D.1.B-1(A) (H_2O_2 System Isolation Valve Operability), "B" hydrogen analyzer became inoperable due to the failure of torus hydrogen sample valve 2-FSV-76-65. There was no effect on public health and safety. A hydrogen analyzer was available and operable. Technical Specification 3.7.H.2 allows operation for 30 days with one hydrogen analyzer operable. The solenoid assembly was replaced. The solenoid assembly for 2-FSV-76-65 failed again during the performance of SI 4.7.D.1.B-1(A). Solenoid valve 2-FSV-76-65 will be repaired during the refueling outage which began July 30, 1982. These failures will be evaluated further with results expected by January 1, 1983.

* Previous Similar Events:

LER BFRO-50-296/82019, 81037
BFRO-50-259/82022

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: JRP