NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: [| | | | | |] (1)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

Line	Field	Value
01	LICENSEE CODE (2)	A L B R F 1
	LICENSE NUMBER (3)	0 0 - 0 0 0 0 0 - 0 0
	LICENSE TYPE (4)	4 1 1 1 1
	CAT (5)	
CON'T 01	REPORT SOURCE (6)	L
	DOCKET NUMBER (7)	0 5 0 0 0 2 5 9
	EVENT DATE (8)	0 8 0 4 8 2
	REPORT DATE (9)	0 8 1 9 8 2

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 During computer analysis of branch lines for suppression chamber attached piping,
03 the unit 1 drywell and torus purge system was found to have a branch line which was
04 anchored to the floor. This configuration would not allow process line movement in
05 a seismic event without possible failure and loss of primary containment capability
06 (T.S. 3.7.A.2). There was no effect on public health and safety. Primary contain-
07 ment remained intact.
08

Line	SYSTEM CODE (11)	CAUSE CODE (12)	CAUSE SUBCODE (13)	COMPONENT CODE (14)	COMP. SUBCODE (15)	VALVE SUBCODE (16)
09	S A	B	A	Z Z Z Z Z Z	Z	Z

LER/RO REPORT NUMBER (17)	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 2	—	0 5 4	/	0 1	T	—	0

ACTION TAKEN (18)	FUTURE ACTION (19)	EFFECT ON PLANT (20)	SHUTDOWN METHOD (21)	HOURS (22)	ATTACHMENT SUBMITTED (23)	NPRD-4 FORM SUB. (24)	PRIME COMP. SUPPLIER (25)	COMPONENT MANUFACTURER (26)
F	Z	Z	Z	0 0 0 0	Y	N	Z	Z 9 9 9

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The problem was caused by a design deficiency. The corrective action included
11 changing the anchor configuration to allow movement of the process line in case of a
12 seismic event. The correction was completed on 7/29/82.
13
14

Line	Field	Value
15	FACILITY STATUS (28)	G
	% POWER (29)	0 0 0
	OTHER STATUS (30)	NA
	METHOD OF DISCOVERY (31)	B
	DISCOVERY DESCRIPTION (32)	Branch line analysis
16	ACTIVITY RELEASED (33)	Z
	CONTENT OF RELEASE (34)	Z
	AMOUNT OF ACTIVITY (35)	NA
	LOCATION OF RELEASE (36)	NA
17	PERSONNEL EXPOSURES NUMBER (37)	0 0 0
	TYPE (38)	Z
	DESCRIPTION (39)	NA
18	PERSONNEL INJURIES NUMBER (40)	0 0 0
	DESCRIPTION (41)	NA
19	LOSS OF OR DAMAGE TO FACILITY TYPE (42)	Z
	DESCRIPTION (43)	NA
20	PUBLICITY ISSUED (44)	N
	DESCRIPTION (45)	NA

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PDR ADOCK 05000259
S PDR

NRC USE ONLY

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Tennessee Valley Authority
Browns Ferry Nuclear Plant

Form BF 17
BF 15.2
2/12/82

LER SUPPLEMENTAL INFORMATION

BFRO-50- 259 / 82054 Technical Specification Involved 3.7.A.2

Reported Under Technical Specification 6.7.2.a.(9) * Date Due NRC 8/20/82

Event Narrative:

Units 1 and 2 were in cold shutdown (0-percent power); unit 2 was in refueling outage (0-percent power). The unit 1 drywell and torus purge system had a branch line that connected to the process line and was anchored to the floor. This arrangement would not accommodate process line movement in a seismic event without failure. The problem was discovered during a branch line analysis of torus attached piping for IE Bulletin 79-14. The problem does not exist on units 2 or 3. A failure could have resulted in loss of primary containment with a seismic event. Corrective action was the removal of stanchion supports H16 and R36 and the connection with a U-bolt to an existing penetration support in accordance with design specifications. There was no effect on public health and safety. Primary containment remained intact.

* Previous Similar Events:

BFRO-50-259/79018, 79032, 80005, 80017, 81003

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: JRP