

Dr. Thomas E. Murley, Director Office of Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission Washington, D.C. 20555

Attention:

Document Control Desk

Subject:

Response to the Request for Additional Information Regarding the Byron/Braidwood Steam Generator Tube Sleeving Proposed Technical Specification Amendment

Byron Station Units 1 and 2 (NPF-37/66; NRC Docket Nos. 50-454/455)

Braidwood Station Units 1 and 2 (NPF-72/77; NRC Docket Nos. 50-456/457)

References:

See Attachment A

Dear Dr. Murley:

R. R. Assa's letter dated December 2, 1993, (Reference 3) transmitted a Request for Additional Information (RAI) related to the Byron/Braidwood proposed Technical Specification amendment regarding steam generator tube sleeving (Reference 4). Reference 2 transmitted Commonwealth Edison Company's (CECo) response to that RAI. Subsequent to that response, a teleconference was held (Reference 1) between CECo, the Nuclear Regulatory Commission (NRC) and Westinghouse to provide clarification to Reference 2. During that teleconference CECo was asked to provide a written response to the following: 1) How were the results of the 7/8 inch tube sleeve evaluation modified to be applicable to steam generators with 3/4 inch tubes? 2) Provide additional information concerning: a) the suitability of the tubing used for the Westinghouse corrosion tests of laser welded sleeves, and b) the data supporting the minimum 1250°F stress relief for laser welded sleeves. Attachment B provides CECo's response.

Please note that the information presented in the Attachment contains information proprietary to Westinghouse Electric Corporation and is supported by an affidavit signed by Westinghouse, the owner of the information. The affidavit sets forth the basis on which the information may be withheld from public disclosure by the Commission and addresses with specificity the considerations listed in paragraph (b)(4) of Section 2.790 of the Commission's regulations. Accordingly, it is respectfully requested that the information which is proprietary to Westinghouse be withheld from public disclosure in accordance with 10 CFR Section 2.790 of the Commission's regulations.

Corre pondence with respect to the proprietary aspects of the items in Attachment B or the supporting West nghouse Affidavit should reference CAW-94-570 and should be addressed to N. J. Liparulo, Manager of Nuclear Safety & Regulatory Activities, Westinghouse Electric Corporation, P.O. Box 355, Pittsburgh, Pennsylvania 15230-0355.

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To the best of my knowledge and belief, the statements contained in this document are true and correct. In some respects these statements are not based on my personal knowledge, but on information furnished by other CECo employees, contractor employees, and/or consultants. Such information has been reviewed in accordance with company practice, and I believe it to be reliable.

Please address any comments or questions regarding this matter to this office.

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MOTARY PUBLIC, STATE OF ILLINOIS MY COMMISSION EXPIRES: 11/29/87 Respectfully,

Denise M. Saccomando

Nuclear Licensing Administrator

Descrip 19, 1994

Attachments'

: R. R. Assa, Braidwood Project Manager - NRR

H. Peterson, SRI - Byron

S. G. Dupont, SRI - Braidwood

J. Martin, Regional Administrator - Region III Office of Nuclear Facility Safety - IDNS