INSERVICE INSPECTION SUMMARY REPORT

FOR

MAINE YANKEE ATOMIC POWER COMPANY

AUGUSTA, MAINE

GENERATING PLANT: Maine Yankee Atomic Power Plant
Wiscasset, Maine

UNIT DESIGNATION NUMBER: Reactor Vessel 20865

COMMERCIAL OPERATING DATE: 12/29/72

EXAMINATION DATES FROM 12/29/92 to 10/13/93 IN THE THIRD TEN YEAR INTERVAL.

COMPLETION DATE: 10/13/93

Andrew Mason
Originator

Peter Mehlhorn
ISI Coordinator

William Schubert
Performance Engineering Section Head

Christopher Shaw
Manager, Plant Engineering

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CONTENTS

PREFACE

This report summarizes the preservice and inservice inspections and pressure tests that were performed at Maine Yankee Atomic Power Plant between December 29, 1992 and October 13, 1993, that fall within the scope of Section XI of the ASME Boiler and Pressure Vessel Code and the Maine Yankee Inservice Inspection Program. The end of the first inspection period of the third ten year inspection interval is April 29, 1996. This completes a portion of the examinations scheduled for the first period, the rest of which will be completed prior to the end of this period.

Several of the exams identified in the Abstract of Examinations were limited due to physical interferences and/or geometry. Specific calculations for percent coverage have not been concluded due to the ambiguity of ASME Code Case N-460 and the lack of guidance for coverage calculations. When this issue has been clarified, Relief Requests, if necessary, will be submitted as required by 10CFR50.55a.

The Authorized Nuclear Inservice Inspector on-site to witness the inspections was Mr. Ray Griffin of Factory Mutual Engineering.

FORM NIS-1 OWNER'S DATA REPORT FOR INSERVICE INSPECTIONS

As required by the Provisions of the ASME Code Rules

1. Owner Maine Yankee Atomic Power Company, Augusta, Maine Sheet 1 of 28
(Name and Address of Owner)

2. Plant Maine Yankee Atomic Power Plant, Wiscasset, Maine
(Name and Address of Owner)

3. Plant Unit 1 4. Owner Certificate of Authorization (if required) DPR-36

5. Commercial Service Date 12/29/72 6. National Board for Unit Reactor Vessel 20865

7. Components Inspected

Component or Appurtenance	Manufacturer or Installer	Manufacturer or Installer Serial No.	State or Province No.	National Board No.
Reactor	C.E.	67206	N/A	20865
St. Gen. #1	C.E.	67501	N/A	20919
St. Gen. #2	C.E.	67502	N/A	20920
St. Gen. #3	C.E.	67503	N/A	20921
Pressurizer	C.E.	67601	N/A	20858
RCP #1	Byron-Jackson	681-N-0421	N/A	N/A
RCP #3	Byron-Jackson	681-N-0423	N/A	N/A
Piping	Stone & Webster	N/A	N/A	N/A

Note: Supplemental sheets in form of lists, sketches, or drawings may be used provided (1) size is 8-1/2 in. x 11 in., (2) information in items 1 through 6 on this data report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

8. Examination Dates 12/29/92 to 10/13/93.

9. Inspection Interval from 12/29/92 to 12/29/02.

10. Applicable Editions of Section XI 1986 Addenda N/A.

11. Abstract of Examinations. Include a list of examinations and a statement concerning status of work required for current interval. See Sheets 3 through 8 and Preface.

12. Abstract of Conditions Noted. See sheets 9 through 28.

13. Abstract of Corrective Measures Recommended and Taken. See sheets 9 through 28.

We certify that the statements made in this report are correct and the examinations and corrective measures taken conform to the rules of the ASME Code, Section XI.

Date 21 DECEMBER 1993 Signed Maine Yankee By Peter J. Mehl
Owner

Certificate of Authorization No. (if applicable) N/A Expiration Date N/A

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State or Province of Maine and employed by Factory Mutual Eng. of Norwood, MA have inspected the components described in this Owners' Data Report during the period 12/29/92 to 10/13/93, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in tnis Owners' Data Report in accordance with the requirements of the ASME Code, XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in this Owners' Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any person injury or property damage or a loss of any kind arising from or connected with this inspection.

Date December 21 1993

~~Inspector's Signature~~ Commissions Maine 697
~~National Board~~, State, ~~Province and~~ Endorsements

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine

2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine

3. PLANT UNIT: 1 4. OWNER CERTIFICATE OF AUTHORIZATION: DPR-36

5. COMMERCIAL SERVICE DATE: 12/29/72

6. NATIONAL BOARD NUMBER FOR UNIT: Reactor Vessel 20865

10. ABSTRACT OF EXAMINATIONS:

CATEGORY	ITEM	NUMBER EXAMINED	METHOD
B-D	B3.110	2 - Nozzle-to-Vessel Welds, Pressurizer	UT
	B3.120	2 - Nozzle Inside Radius Sections, Pressurizer	UT
	B3.130	2 - Nozzle-to-Vessel Welds, Steam Generator #1	UT
	B3.140	2 - Nozzle Inside Radius Sections, Steam Generator #1	UT
B-F	B5.40	1 - 4" Safe End-to-Nozzle Weld, Pressurizer	PT
	B5.50	1 - 3" Nozzle-to-Safe End Weld, Pressurizer	PT
	B5.130	2 - 33-1/2" Dissimilar Metal Butt Welds	PT, UT
		1 - 12" Dissimilar Metal Butt Weld	PT, UT
	B5.140	1 - 3" Dissimilar Metal Butt Weld	PT
		1 - 2" Dissimilar Metal Butt Weld	PT
B-G-1	B6.180	16 - Pump Studs, Reactor Coolant Pump #3	VT-1
	B6.190	1 - Pump Flange Surface, Reactor Coolant Pump #3	VT-1
	B6.200	16 - Pump Stud Nuts, Reactor Coolant Pump #3	VT-1
B-G-2	B7.50	2 - Sets - Flange Bolting	VT-1

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10. ABSTRACT OF EXAMINATIONS:

CATEGORY	ITEM	NUMBER EXAMINED	METHOD
B-J	B9.11	2 - 33-1/2" Circumferential Pipe Welds	MT, UT
		3 - 12" Circumferential Pipe Welds	PT
		1 - 4" Circumferential Pipe Weld	PT, UT
	B9.21	6 - 3" Circumferential Pipe Welds	PT
	B9.32	1 - 3" Branch Pipe Connection Weld	PT
		1 - 2" Branch Pipe Connection Weld	PT
	B9.40	9 - 2" Socket Welds	PT
B-L-2	B12.20	1 - Pump Casing Surface, Reactor Coolant Pump	VT-3
B-M-2	B12.50	1 - Valve Body	VT-3
B-P	B15.10	1 - Reactor Coolant System Leakage Test	VT-2
B-Q	B16.20	Sample selected I.A.W. EPRI PWR Steam Generator Examination Guidelines which exceed the requirements of Maine Yankee Technical Specification 4.10, Steam Generator Tube Surveillance. (continued on next page)	

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10. ABSTRACT OF EXAMINATIONS:

CATEGORY	ITEM	NUMBER EXAMINED	METHOD
B-Q (cont'd)	B16.20 (cont'd)	Steam Generator #1	
		20% (1140 tubes) Bobbin coil technique and 50% (2852 tubes) Motorized Rotating Pancake Coil (MRPC) technique of hot leg (T_H) tubesheet Expansion Transition Zone (ETZ), first sample.	ECT
		6% (344 tubes) Bobbin and 6% (344 tubes) MRPC of T_H ETZ, second sample.	ECT
		12% (688 tubes) Bobbin, third sample.	ECT
		689 tubes Bobbin, fourth sample - voluntary sample.	ECT
		87 tubes Bobbin, fifth sample - voluntary sample.	ECT
		122 tubes Bobbin, sixth sample - voluntary sample.	ECT
		366 tubes Bobbin, seventh sample - voluntary sample.	ECT
		25 tubes MRPC in steam blanketed region U-bend area - voluntary sample.	ECT
		25 tubes MRPC in dented support intersection area - voluntary sample.	ECT

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10. ABSTRACT OF EXAMINATIONS:

CATEGORY	ITEM	NUMBER EXAMINED	METHOD
B-Q (cont'd)	B16.20 (cont'd)	Steam Generator #2	
		20% (1140 tubes) Bobbin and 50% (2852 tubes) MRPC of T_H ETZ, first sample.	ECT
		6% (344 tubes) Bobbin, second sample.	ECT
		24% (1376 tubes) Bobbin, third sample - 688 tubes voluntary sample.	ECT
		782 tubes Bobbin, fourth sample - voluntary sample.	ECT
		25 tubes MRPC in steam blanketed region U-bend area - voluntary sample.	ECT
		Steam Generator #3	
		20.7% (1183 tubes) Bobbin and 50% (2852 tubes) MRPC of T_H ETZ, first sample.	ECT
		6% (344 tubes) Bobbin and 6% (344 tubes) MRPC of T_H ETZ, second sample.	ECT
		12% (688 tubes) Bobbin, third sample.	ECT
		688 tubes Bobbin, fourth sample - voluntary sample.	ECT
		169 tubes Bobbin, fifth sample - voluntary sample.	ECT
		1017 tubes Bobbin, sixth sample - voluntary sample.	ECT
		1556 tubes Bobbin, seventh sample - voluntary sample.	ECT

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10. ABSTRACT OF EXAMINATIONS:

CATEGORY	ITEM	NUMBER EXAMINED	METHOD
B-Q (cont'd)	B16.20 (cont'd)	25 tubes MRPC in steam blanket region U-bend area - voluntary sample.	ECT
F-A	F1.10	5 - Mechanically Connected Supports, Safety Class 1	VT-3
	F1.40	1 - Reactor Coolant Pump Support, Safety Class 1	VT-3
C-C	C3.20	6 - Integrally Welded Attachments, Piping	PT or MT
	C3.30	1 - Integrally Welded Attachment, Containment Spray Pump	PT
C-F-1	C5.11	2 - 10" Circumferential Pipe Welds	PT, UT
		1 - 12" Circumferential Pipe Weld	PT, UT
		4 - 14" Circumferential Pipe Welds	PT, UT
		1 - 16" Circumferential Pipe Weld	PT
		2 - 18" Circumferential Pipe Welds	PT, UT
	C5.21	3 - 4" Circumferential Pipe Welds	PT, UT
	C5.30	2 - 2" Socket Welds	PT
	C5.41	1 - 2" Circumferential Pipe Weld	PT

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10. ABSTRACT OF EXAMINATIONS:

CATEGORY	ITEM	NUMBER EXAMINED	METHOD
C-F-2	C5.51	2 - 14" Circumferential Pipe Welds	MT, UT
		1 - 14" Circumferential Pipe Weld	MT
		1 - 30" Circumferential Pipe Weld	MT, UT
	C5.81	2 - 30" Circumferential Branch Connection Welds	MT
C-G	C6.10	1 - Pump Casing Welds, Containment Spray Pump	PT
F-A	F1.20	9 - Welded Connections to Building Structure, Safety Class 2	VT-3
	F1.40	1 - Containment Spray Pump Support, Safety Class 2	VT-3
D-B	D2.20	1 - Integrally Welded Attachment, Safety Class 3	VT-3
F-A	F1.40	1 - RHR Heat Exchanger Support, Safety Class 3	VT-3

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11. ABSTRACT OF CONDITIONS NOTED:

12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

CATEGORY	ITEM	CONDITION	CORRECTIVE MEASURE
B-F	B5.40	One surface indication was detected near a safe end to nozzle dissimilar metal weld on a 4" pressurizer spray line. The weld defect was a manufacturing type discontinuity.	An ultrasonic examination was performed in the area of the indication to determine if the surface indication had any depth. No depth was observed and the indication was removed and re-examined with no recordable defects.
C-C	C3.20	Three linear surface indications were detected on a containment penetration integrally welded attachment for a 10" safety injection line. The weld defects were lack of fusion from original construction. The areas were most likely uncovered during weld preparation prior to performing examination.	The weld defects were removed and a surface exam was performed on the excavated area to verify complete removal of defects. The area was then weld repaired back to original configuration. An additional surface exam was performed after the repair that revealed no rejectable indications. The two sister train penetration integrally welded attachments were examined as part of the scheduled work plan with no recordable defects.

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CATEGORY	ITEM	CONDITION	CORRECTIVE MEASURE
C-F-2	C5.51	Several linear surface indications were detected on a pipe to elbow weld on a 14" main feedwater line. The indications were maufacturing type discontinuities.	An ultrasonic examination was performed in the areas of the indications to determine if the surface indications had any depth. No depth was observed and the indications were removed and re-examined with no recordable defects.

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12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

CATEGORY	ITEM	CONDITION				CORRECTIVE MEASURE
B-Q	B16.20	STEAM GENERATOR #1 TUBING				
DEFECTIVE TUBES		ROW	LINE	% TWD	LOCATION	
		3	32	FLAW	NEAR 3RD VERTICAL SUPPORT	PLUGGED TUBE
		5	112	CRACK	NEAR 3RD VERTICAL SUPPORT	PLUGGED TUBE
		6	21	CRACK	NEAR 3RD VERTICAL SUPPORT	PLUGGED TUBE
		7	8	CRACK	NEAR 3RD VERTICAL SUPPORT	PLUGGED TUBE
		7	42	97	0.9% ABOVE 3RD VERTICAL SUPPORT	PLUGGED TUBE
		7	130	PRECURSOR	NEAR 3RD VERTICAL SUPPORT AND NEAR 6TH HOT LEG SUPPORT	PLUGGED TUBE
		18	97	CRACK	NEAR 1ST HOT LEG SUPPORT	PLUGGED TUBE
		19	112	CRACK	NEAR 2ND AND 5TH HOT LEG SUPPORT	PLUGGED TUBE
		23	122	CRACK	NEAR 2ND HOT LEG SUPPORT	PLUGGED TUBE
		24	39	50	0.9" BELOW 1ST HOT LEG SUPPORT	PLUGGED TUBE
		26	111	DENTED	NEAR 2ND HOT LEG SUPPORT	PLUGGED TUBE
		27	100	51	13.0" ABOVE COLD TUBESHEET	PLUGGED TUBE
		27	106	43	9.6" ABOVE COLD TUBESHEET	PLUGGED TUBE
		35	46	CRACK	NEAR 1ST HOT LEG SUPPORT	PLUGGED TUBE
		44	17	79	AT HOT TUBESHEET	PLUGGED TUBE
		50	55	43	10.6" ABOVE COLD TUBESHEET	PLUGGED TUBE
		54	93	56	0.4" BELOW 2ND HOT LEG SUPPORT	PLUGGED TUBE

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12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

CATEGORY	ITEM	CONDITION				CORRECTIVE MEASURE
B-Q (cont'd)	B16.20 (cont'd)	STEAM GENERATOR #1 TUBING (cont'd)				
DEGRADED TUBES		ROW	LINE	% TWD	LOCATION	
		2	109	25	0.57" ABOVE HOT TUBESHEET	NONE REQUIRED
		7	2	23	7.27" ABOVE HOT TUBESHEET	FOR LESS THAN
		8	127	23	3.70" ABOVE COLD TUBESHEET	40% TWD
		11	124	21	3.58" ABOVE HOT TUBESHEET	
		12	95	21	1.91" ABOVE HOT TUBESHEET	
		12	107	35	4.71" ABOVE HOT TUBESHEET	
		19	30	34	0.34" ABOVE 1ST HOT LEG SUPPORT	
		20	107	37	6.03" ABOVE COLD TUBESHEET	
		22	127	25	0.76" ABOVE HOT TUBESHEET	
		23	124	35	4.39" ABOVE HOT TUBESHEET	
		24	45	21	8.20" ABOVE COLD TUBESHEET	
		26	47	37	3.80" ABOVE COLD TUBESHEET	
		27	44	37	8.87" ABOVE HOT TUBESHEET	
		27	112	30	10.77" ABOVE HOT TUBESHEET	
				39	10.63" ABOVE HOT TUBESHEET	
		27	116	21	5.92" ABOVE HOT TUBESHEET	
				31	6.32" ABOVE HOT TUBESHEET	
				20	6.44" ABOVE HOT TUBESHEET	
				26	6.06" ABOVE HOT TUBESHEET	
		28	45	22	13.81" ABOVE HOT TUBESHEET	
		28	107	24	15.03" ABOVE HOT TUBESHEET	

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12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

CATEGORY	ITEM	CONDITION				CORRECTIVE MEASURE
B-Q (cont'd)	B16.20 (cont'd)	STEAM GENERATOR #1 TUBING (cont'd)				
DEGRADED TUBES (cont'd)		ROW	LINE	% TWD	LOCATION	
		29	108	28	14.09" ABOVE HOT TUBESHEET	NONE REQUIRED
		30	101	28	9.84" ABOVE HOT TUBESHEET	FOR LESS THAN
		31	122	27	4.19" ABOVE HOT TUBESHEET	40% TWD
		34	123	24	4.43" ABOVE HOT TUBESHEET	
		35	28	25	3.17" ABOVE HOT TUBESHEET	
		35	98	38	14.26" ABOVE COLD TUBESHEET	
		35	114	22	6.44" ABOVE HOT TUBESHEET	
		38	37	34	3.20" ABOVE 2ND HOT LEG SUPPORT	
		38	43	28	2.46" ABOVE 1ST HOT LEG SUPPORT	
		39	60	27	25.66" ABOVE HOT TUBESHEET	
		43	42	22	9.98" ABOVE COLD TUBESHEET	
		43	78	38	2.88" ABOVE HOT TUBESHEET	
		45	54	33	9.69" ABOVE COLD TUBESHEET	
		45	86	26	5.14" ABOVE HOT TUBESHEET	
		49	56	21	10.22" ABOVE COLD TUBESHEET	
		50	99	29	1.28" ABOVE 2ND COLD LEG SUPPORT	
		53	112	25	0.26" ABOVE 2ND HOT LEG SUPPORT	
		57	90	36	1.54" ABOVE HOT TUBESHEET	
		58	33	22	1.00" ABOVE HOT TUBESHEET	
		59	58	36	7.91" ABOVE COLD TUBESHEET	
		63	80	38	2.01" ABOVE HOT TUBESHEET	

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12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

CATEGORY	ITEM	CONDITION				CORRECTIVE MEASURE
B-Q (cont'd)	B16.20 (cont'd)	STEAM GENERATOR #1 TUBING (cont'd)				
DEGRADED TUBES (cont'd)		ROW	LINE	% TWD	LOCATION	
		67	36	27	1.08" ABOVE HOT TUBESHEET	NONE REQUIRED
		70	37	27	7.80" ABOVE HOT TUBESHEET	FOR LESS THAN
				32	7.22" ABOVE HOT TUBESHEET	40% TWD
		77	42	25	0.85" ABOVE HOT TUBESHEET	
		79	86	27	1.86" ABOVE HOT TUBESHEET	
		81	50	21	0.82" ABOVE HOT TUBESHEET	
		85	92	24	1.24" ABOVE HOT TUBESHEET	
		87	54	30	6.00" ABOVE HOT TUBESHEET	
		88	57	30	1.15" ABOVE HOT TUBESHEET	
		91	60	25	6.29" ABOVE HOT TUBESHEET	
		104	103	31	10.93" ABOVE 4TH VERTICAL SUPPORT	
				28	12.41" ABOVE 8TH HOT LEG SUPPORT	

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CATEGORY	ITEM	CONDITION				CORRECTIVE MEASURE
B-Q (cont'd)	B16.20 (cont'd)	STEAM GENERATOR #1 TUBING				
IMPERFECT TUBES		ROW	LINE	% TWD	LOCATION	
		7	124	7	2.19" ABOVE HOT TUBESHEET	NONE REQUIRED
		9	4	12	20.74" ABOVE 6TH HOT LEG SUPPORT	FOR LESS THAN
		12	95	4	3.11" ABOVE HOT TUBESHEET	40% TWD
		13	107	18	10.91" ABOVE HOT TUBESHEET	
		18	33	15	4.52" ABOVE HOT TUBESHEET	
		19	18	17	0.37" BELOW 6TH COLD LEG SUPPORT	
		21	106	18	1.93" ABOVE COLD TUBESHEET	
		24	9	13	10.79" ABOVE 3RD COLD LEG SUPPORT	
		24	33	19	5.45" ABOVE HOT TUBESHEET	
		25	114	18	2.94" ABOVE HOT TUBESHEET	
		26	105	17	9.02" ABOVE COLD TUBESHEET	
		26	131	16	6.04" ABOVE COLD TUBESHEET	
		27	108	16	3.31" ABOVE 5TH HOT LEG SUPPORT	
		27	116	5	6.27" ABOVE HOT TUBESHEET	
				18	5.95" ABOVE HOT TUBESHEET	
		29	106	9	5.72" ABOVE HOT TUBESHEET	
		30	37	4	14.09" ABOVE HOT TUBESHEET	
		31	110	8	3.61" ABOVE 6TH HOT LEG SUPPORT	
		31	114	17	7.28" ABOVE HOT TUBESHEET	
		31	116	10	21.74" ABOVE 3RD COLD LEG SUPPORT	
		32	37	6	1.48" ABOVE HOT TUBESHEET	
		33	40	19	1.86" ABOVE HOT TUBESHEET	
		33	106	14	1.59" ABOVE HOT TUBESHEET	
		33	112	18	8.07" ABOVE HOT TUBESHEET	
		34	37	16	11.63" ABOVE COLD TUBESHEET	

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12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

CATEGORY	ITEM	CONDITION				CORRECTIVE MEASURE
B-Q (cont'd)	B16.20 (cont'd)	STEAM GENERATOR #1 TUBING (cont'd)				
IMPERFECT TUBES (cont'd)		ROW	LINE	% TWD	LOCATION	
		35	58	6	0.03" ABOVE HOT TUBESHEET	NONE REQUIRED
		35	86	15	1.27" ABOVE HOT TUBESHEET	FOR LESS THAN
		36	81	17	24.13" ABOVE 5TH COLD LEG SUPPORT	40% TWD
		38	123	19	1.28" ABOVE HOT TUBESHEET	
		42	109	9	4.47" ABOVE HOT TUBESHEET	
		46	63	19	3.36" ABOVE COLD TUBESHEET	
		48	57	17	9.03" ABOVE COLD TUBESHEET	
		48	97	12	1.75" ABOVE HOT TUBESHEET	
		49	56	3	7.60" ABOVE COLD TUBESHEET	
		49	124	16	0.84" ABOVE HOT TUBESHEET	
		52	57	18	9.19" ABOVE COLD TUBESHEET	
		55	60	9	12.11" ABOVE COLD TUBESHEET	
		57	50	7	0.39" BELOW 5TH HOT LEG SUPPORT	
		58	109	19	4.34" ABOVE HOT TUBESHEET	
		62	65	17	1.95" ABOVE HOT TUBESHEET	
		63	78	17	13.77" ABOVE COLD TUBESHEET	
				18	1.52" ABOVE HOT TUBESHEET	
		65	82	18	2.35" ABOVE COLD TUBESHEET	
		67	94	2	3.20" ABOVE HOT TUBESHEET	
		72	111	19	2.13" ABOVE HOT TUBESHEET	
		73	118	18	1.54" ABOVE HOT TUBESHEET	
		74	105	6	2.69" ABOVE HOT TUBESHEET	
		79	88	15	0.61" BELOW 4TH HOT LEG SUPPORT	
		82	109	14	1.21" ABOVE HOT TUBESHEET	

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine

2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine

3. PLANT UNIT: 1 4. OWNER CERTIFICATE OF AUTHORIZATION: DPR-36

5. COMMERCIAL SERVICE DATE: 12/29/72

6. NATIONAL BOARD NUMBER FOR UNIT: Reactor Vessel 20865

11. ABSTRACT OF CONDITIONS NOTED:

12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

CATEGORY	ITEM	CONDITION				CORRECTIVE MEASURE
B-Q (cont'd)	B16.20 (cont'd)	STEAM GENERATOR #1 TUBING (cont'd)				
IMPERFECT TUBES (cont'd)		ROW	LINE	% TWD	LOCATION	
		84	93	4	21.61" ABOVE 3RD COLD LEG SUPPORT	NONE REQUIRED
		85	76	4	4.47" ABOVE HOT TUBESHEET	FOR LESS THAN
		87	60	13	2.56" ABOVE HOT TUBESHEET	40% TWD
		88	61	17	1.16" ABOVE HOT TUBESHEET	
		88	77	9	2.73" ABOVE HOT TUBESHEET	
		89	68	15	3.75" ABOVE HOT TUBESHEET	
		89	84	15	4.15" ABOVE HOT TUBESHEET	

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine

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3. PLANT UNIT: 1 4. OWNER CERTIFICATE OF AUTHORIZATION: DPR-36

5. COMMERCIAL SERVICE DATE: 12/29/72

6. NATIONAL BOARD NUMBER FOR UNIT: Reactor Vessel 20865

11. ABSTRACT OF CONDITIONS NOTED:

12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

CATEGORY	ITEM	CONDITION				CORRECTIVE MEASURE
B-Q (cont'd)	B16.20 (cont'd)	STEAM GENERATOR #2 TUBING				
		ROW	LINE	% TWD	LOCATION	
DEFECTIVE TUBES		6	31	QUESTION-ABLE	NEAR 3RD VERTICAL SUPPORT	PLUGGED TUBE
		7	100	CRACK	NEAR 3RD VERTICAL SUPPORT	PLUGGED TUBE
		8	112	CRACK	NEAR 3RD VERTICAL SUPPORT	PLUGGED TUBE
		36	97	DISTORTED	NEAR HOT TUBESHEET AND NEAR 5TH COLD LEG SUPPORT	PLUGGED TUBE
DEGRADED TUBES		ROW	LINE	% TWD	LOCATION	
		13	96	23	1.28" ABOVF COLD TUBESHEET	NONE REQUIRED
		31	106	22	7.71" ABOVE HOT TUBESHEET	FOR LESS THAN
		32	19	34	0.51" BELOW 4TH COLD LEG SUPPORT	40% TWD
		35	38	27	5.46" ABOVE COLD TUBESHEET	
		36	97	22	0.81" ABOVE HOT TUBESHEET	
				37	1.33" ABOVE HOT TUBESHEET	
		42	91	38	8.32" ABOVE COLD TUBESHEET	
				22	7.24" ABOVE COLD TUBESHEET	
		44	95	25	6.49" ABOVE COLD TUBESHEET	
		46	39	38	5.06" ABOVE COLD TUBESHEET	
		46	55	30	7.95" ABOVE COLD TUBESHEET	
		46	95	24	6.37" ABOVE COLD TUBESHEET	
		47	38	20	4.69" ABOVE COLD TUBESHEET	
				28	5.02" ABOVE COLD TUBESHEET	
		47	54	34	8.77" ABOVE COLD TUBESHEET	
		47	66	27	6.87" ABOVE COLD TUBESHEET	
		48	55	32	9.77" ABOVE COLD TUBESHEET	
				20	9.07" ABOVE COLD TUBESHEET	

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine

2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine

3. PLANT UNIT: 1 4. OWNER CERTIFICATE OF AUTHORIZATION: DPR-36

5. COMMERCIAL SERVICE DATE: 12/29/72

6. NATIONAL BOARD NUMBER FOR UNIT: Reactor Vessel 20865

11. ABSTRACT OF CONDITIONS NOTED:

12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

CATEGORY	ITEM	CONDITION				CORRECTIVE MEASURE
B-Q (cont'd)	B16.20 (cont'd)	STEAM GENERATOR #2 TUBING (cont'd)				
DEGRADED TUBES (cont'd)		ROW	LINE	% TWD	LOCATION	
		48	77	33	7.91" ABOVE COLD TUBESHEET	NONE REQUIRED
		51	94	27	6.71" ABOVE COLD TUBESHEET	FOR LESS THAN
		52	53	25	8.39" ABOVE COLD TUBESHEET	40% TWD
		52	95	29	7.68" ABOVE COLD TUBESHEET	
		53	86	29	8.73" ABOVE COLD TUBESHEET	
		53	94	34	8.33" ABOVE COLD TUBESHEET	
		54	53	39	8.04" ABOVE COLD TUBESHEET	
		55	80	25	6.51" ABOVE COLD TUBESHEET	
		59	56	28	8.15" ABOVE COLD TUBESHEET	
		59	70	24	10.15" ABOVE COLD TUBESHEET	
		59	88	24	1.08" ABOVE HOT TUBESHEET	
		64	69	36	6.94" ABOVE COLD TUBESHEET	
		80	83	24	0.63" ABOVE 3RD COLD LEG SUPPORT	
		80	117	21	5.68" ABOVE HOT TUBESHEET	
		93	52	36	0.33" ABOVE 8TH COLD LEG SUPPORT	

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine

2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine

3. PLANT UNIT: 1 4. OWNER CERTIFICATE OF AUTHORIZATION: DPR-36

5. COMMERCIAL SERVICE DATE: 12/29/72

6. NATIONAL BOARD NUMBER FOR UNIT: Reactor Vessel 20865

11. ABSTRACT OF CONDITIONS NOTED:

12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

CATEGORY	ITEM	CONDITION				CORRECTIVE MEASURE
B-Q (cont'd)	B16.20 (cont'd)	STEAM GENERATOR #2 TUBING (cont'd)				
IMPERFECT TUBES		ROW	LINE	% TWD	LOCATION	
		5	6	12	4.79" ABOVE 2ND HOT LEG SUPPORT	NONE REQUIRED
		9	26	5	0.36" ABOVE 2ND COLD LEG SUPPORT	FOR LESS THAN
		9	116	12	0.45" ABOVE 4TH HOT LEG SUPPORT	40% TWD
		15	40	8	10.86" ABOVE COLD TUBESHEET	
				13	14.07" ABOVE COLD TUBESHEET	
		15	96	19	1.61" ABOVE COLD TUBESHEET	
		17	96	13	1.69" ABOVE COLD TUBESHEET	
		29	34	4	0.55" ABOVE 3RD COLD LEG SUPPORT	
		32	39	19	2.46" ABOVE HOT TUBESHEET	
		36	105	5	0.77" ABOVE 3RD COLD LEG SUPPORT	
				8	1.92" ABOVE HOT TUBESHEET	
		37	38	14	4.51" ABOVE COLD TUBESHEET	
		40	103	17	1.39" ABOVE HOT TUBESHEET	
		44	39	13	4.78" ABOVE COLD TUBESHEET	
		46	39	18	4.57" ABOVE COLD TUBESHEET	
		46	55	19	8.28" ABOVE COLD TUBESHEET	
		47	54	16	9.35" ABOVE COLD TUBESHEET	
		47	78	16	8.31" ABOVE COLD TUBESHEET	
		49	88	10	6.22" ABOVE COLD TUBESHEET	
		50	65	10	0.16" BELOW 3RD VERTICAL SUPPORT	
		51	54	16	5.16" ABOVE COLD TUBESHEET	
		52	95	16	7.26" ABOVE COLD TUBESHEET	
		53	24	9	0.26" ABOVE 1ST HOT LEG SUPPORT	
		54	119	5	19.68" ABOVE 3RD HOT LEG SUPPORT	

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine

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3. PLANT UNIT: 1 4. OWNER CERTIFICATE OF AUTHORIZATION: DPR-36

5. COMMERCIAL SERVICE DATE: 12/29/72

6. NATIONAL BOARD NUMBER FOR UNIT: Reactor Vessel 20865

11. ABSTRACT OF CONDITIONS NOTED:

12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

CATEGORY	ITEM	CONDITION				CORRECTIVE MEASURE
B-Q (cont'd)	B16.20 (cont'd)	STEAM GENERATOR #2 TUBING (cont'd)				
IMPERFECT TUBES (cont'd)		ROW	LINE	% TWD	LOCATION	
		58	67	2	0.57" ABOVE HOT TUBESHEET	NONE REQUIRED
		60	81	9	1.23" ABOVE HOT TUBESHEET	FOR LESS THAN
		61	84	6	1.38" ABOVE HOT TUBESHEET	40% TWD
		64	59	9	3.64" ABOVE COLD TUBESHEET	
		66	93	6	6.18" ABOVE HOT TUBESHEET	
		67	20	5	12.45" ABOVE 8TH HOT LEG SUPPORT	
				15	30.68" ABOVE 8TH HOT LEG SUPPORT	
		69	50	15	0.28" BELOW 1ST HOT LEG SUPPORT	
		70	99	7	2.10" ABOVE HOT TUBESHEET	
		73	94	10	2.36" ABOVE HOT TUBESHEET	
		76	97	13	0.33" ABOVE 4TH COLD LEG SUPPORT	
		82	61	10	3.50" ABOVE HOT TUBESHEET	
		84	79	12	16.04" ABOVE 1ST HOT LEG SUPPORT	

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine

2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine

3. PLANT UNIT: 1 4. OWNER CERTIFICATE OF AUTHORIZATION: DPR-36

5. COMMERCIAL SERVICE DATE: 12/29/72

6. NATIONAL BOARD NUMBER FOR UNIT: Reactor Vessel 20865

11. ABSTRACT OF CONDITIONS NOTED:

12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

CATEGORY	ITEM	CONDITION				CORRECTIVE MEASURE
B-Q (cont'd)	B16.20 (cont'd)	STEAM GENERATOR #3 TUBING				
DEFECTIVE TUBES		ROW	LINE	% TWD	LOCATION	
		6	123	CRACK	NEAR 3RD VERTICAL SUPPORT	PLUGGED TUBE
		7	104	CRACK	NEAR 3RD VERTICAL SUPPORT	PLUGGED TUBE
		14	27	CRACK	NEAR 1ST HOT LEG SUPPORT	PLUGGED TUBE
		14	103	89	0.3" ABOVE 2ND HOT LEG SUPPORT	PLUGGED TUBE
		14	131	CRACK	NEAR 2ND HOT LEG SUPPORT	PLUGGED TUBE
		14	133	CRACK	NEAR 2ND HOT LEG SUPPORT	PLUGGED TUBE
		25	22	CRACK	NEAR 1ST HOT LEG SUPPORT	PLUGGED TUBE
		25	28	CRACK	NEAR 1ST AND 2ND HOT LEG SUPPORT	PLUGGED TUBE
		28	131	CRACK	NEAR 2ND HOT LEG SUPPORT	PLUGGED TUBE
		36	37	42	6.5" ABOVE COLD TUBESHEET	PLUGGED TUBE
		36	91	CRACK	NEAR 1ST HOT LEG SUPPORT	PLUGGED TUBE
		37	94	52	9.4" ABOVE COLD TUBESHEET	PLUGGED TUBE
		38	91	DISTORTED	NEAR 1ST HOT LEG SUPPORT	PLUGGED TUBE
		41	100	CRACK	NEAR 1ST HOT LEG SUPPORT	PLUGGED TUBE
		43	18	45	3.2" ABOVE 1ST HOT LEG SUPPORT	PLUGGED TUBE
		43	28	67	3.8" ABOVE HOT TUBESHEET	PLUGGED TUBE
		45	56	64	0.7" BELOW 1ST HOT LEG SUPPORT	PLUGGED TUBE
		47	94	CRACK	NEAR 1ST HOT LEG SUPPORT	PLUGGED TUBE
		49	58	58	18.2" ABOVE 1ST HOT LEG SUPPORT	PLUGGED TUBE
		49	108	97	0.5" BELOW 1ST HOT LEG SUPPORT	PLUGGED TUBE

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine

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11. ABSTRACT OF CONDITIONS NOTED:

12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

CATEGORY	ITEM	CONDITION				CORRECTIVE MEASURE
B-Q (cont'd)	B16.20 (cont'd)	STEAM GENERATOR #3 TUBING (cont'd)				
DEFECTIVE TUBES (cont'd)		ROW	LINE	% TWD	LOCATION	
		50	45	CRACK	NEAR 2ND HOT LEG SUPPORT	PLUGGED TUBE
		55	48	49	0.4" ABOVE 2ND HOT LEG SUPPORT	PLUGGED TUBE
		60	87	DISTORTED	NEAR COLD TUBESHEET	PLUGGED TUBE
		61	32	OBSTRUCTED		PLUGGED TUBE
		65	66	94	0.4" BELOW 1ST HOT LEG SUPPORT	PLUGGED TUBE
		68	27	79	NEAR HOT TUBESHEET	PLUGGED TUBE
		75	82	CRACK	NEAR 1ST HOT LEG SUPPORT	PLUGGED TUBE
		85	26	43	1.0" ABOVE 5TH COLD LEG SUPPORT	PLUGGED TUBE
		87	84	CRACK	NEAR 2ND HOT LEG SUPPORT	PLUGGED TUBE
		90	79	53	1.3" ABOVE HOT TUBESHEET	PLUGGED TUBE
		91	72	41	1.2" ABOVE HOT TUBESHEET	PLUGGED TUBE
		94	31	OBSTRUCTED		PLUGGED TUBE
		105	102	DISTORTED	NEAR 4TH COLD LEG SUPPORT	PLUGGED TUBE
		113	58	44	12.1" ABOVE HOT TUBESHEET	PLUGGED TUBE

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine

2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine

3. PLANT UNIT: 1 4. OWNER CERTIFICATE OF AUTHORIZATION: DPR-36

5. COMMERCIAL SERVICE DATE: 12/29/72

6. NATIONAL BOARD NUMBER FOR UNIT: Reactor Vessel 20865

11. ABSTRACT OF CONDITIONS NOTED:

12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

CATEGORY	ITEM	CONDITION				CORRECTIVE MEASURE
B-Q (cont'd)	B16.20 (cont'd)	STEAM GENERATOR #3 TUBING (cont'd)				
DEGRADED TUBES		ROW	LINE	% TWD	LOCATION	
		1	118	38	1.02" ABOVE HOT TUBESHEET	NONE REQUIRED
		8	103	21	1.15" ABOVE HOT TUBESHEET	FOR LESS THAN
		14	125	27	1.88" ABOVE HOT TUBESHEET	40% TWD
		15	30	24	20.90" ABOVE 1ST HOT LEG SUPPORT	
		24	37	30	5.94" ABOVE COLD TUBESHEET	
				32	4.75" ABOVE COLD TUBESHEET	
		27	44	26	1.53" ABOVE HOT TUBESHEET	
		31	30	34	7.65" ABOVE HOT TUBESHEET	
				37	7.33" ABOVE HOT TUBESHEET	
		31	48	37	1.12" ABOVE HOT TUBESHEET	
		31	98	21	13.42" ABOVE HOT TUBESHEET	
		32	29	33	7.01" ABOVE HOT TUBESHEET	
		32	103	37	9.71" ABOVE HOT TUBESHEET	
		34	45	20	11.26" ABOVE 1ST HOT LEG SUPPORT	
		35	10	20	0.52" BELOW 3RD HOT LEG SUPPORT	
		38	97	23	29.00" ABOVE 1ST HOT LEG SUPPORT	
		38	99	31	16.23" ABOVE HOT TUBESHEET	
		39	32	21	11.74" DIAGONAL HOT LEG SUPPORT	
		39	36	37	6.43" ABOVE COLD TUBESHEET	
		40	37	24	6.83" ABOVE COLD TUBESHEET	
		41	62	32	1.00" ABOVE HOT TUBESHEET	
		42	97	24	2.63" ABOVE HOT TUBESHEET	
		43	24	31	2.08" ABOVE HOT TUBESHEET	
				25	0.87" ABOVE HOT TUBESHEET	
		43	38	31	2.44" ABOVE COLD TUBESHEET	

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine

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11. ABSTRACT OF CONDITIONS NOTED:

12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

CATEGORY	ITEM	CONDITION				CORRECTIVE MEASURE
B-Q (cont'd)	B16.20 (cont'd)	STEAM GENERATOR #3 TUBING (cont'd)				
DEGRADED TUBES (cont'd)		ROW	LINE	% TWD	LOCATION	
		44	37	29	2.48" ABOVE COLD TUBESHEET	
		45	50	32	12.47" ABOVE COLD TUBESHEET	NONE REQUIRED
		46	51	23	12.64" ABOVE COLD TUBESHEET	IF LESS THAN
				33	13.00" ABOVE COLD TUBESHEET	40% TWD
		46	83	36	14.12" ABOVE 4TH HOT LEG SUPPORT	
		50	49	39	13.03" ABOVE COLD TUBESHEET	
		54	61	36	3.84" ABOVE COLD TUBESHEET	
		55	28	26	1.12" ABOVE HOT TUBESHEET	
		55	30	33	1.16" ABOVE HOT TUBESHEET	
				33	1.17" ABOVE HOT TUBESHEET	
		55	34	29	3.84" ABOVE HOT TUBESHEET	
				20	3.58" ABOVE HOT TUBESHEET	
		60	13	38	4.22" ABOVE 7TH HOT LEG SUPPORT	
		61	74	25	0.93" ABOVE HOT TUBESHEET	
		66	67	29	28.83" ABOVE 1ST HOT LEG SUPPORT	
		69	16	26	2.13" DIAGONAL COLD LEG SUPPORT	
		76	45	36	1.02" ABOVE HOT TUBESHEET	
		79	42	32	2.13" ABOVE HOT TUBESHEET	
		83	43	35	1.21" ABOVE HOT TUBESHEET	
		83	58	29	3.13" ABOVE HOT TUBESHEET	
		87	82	37	2.22" ABOVE HOT TUBESHEET	
		88	51	33	0.95" ABOVE HOT TUBESHEET	
		88	55	37	1.51" ABOVE HOT TUBESHEET	
				30	0.77" ABOVE HOT TUBESHEET	

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine

2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine

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11. ABSTRACT OF CONDITIONS NOTED:

12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

CATEGORY	ITEM	CONDITION				CORRECTIVE MEASURE
B-Q (cont'd)	B16.20 (cont'd)	STEAM GENERATOR #3 TUBING (cont'd)				
DEGRADED TUBES (cont'd)		ROW	LINE	% TWD	LOCATION	
		89	58	30	1.35" ABOVE HOT TUBESHEET	NOT REQUIRED
		89	80	29	1.23" ABOVE HOT TUBESHEET	FOR LESS THAN
		91	60	33	1.35" ABOVE HOT TUBESHEET	40% TWD
		91	62	27	1.20" ABOVE HOT TUBESHEET	
		91	64	22	1.04" ABOVE HOT TUBESHEET	
		91	74	30	1.14" ABOVE HOT TUBESHEET	
		92	57	32	5.26" ABOVE HOT TUBESHEET	
				22	1.03" ABOVE HOT TUBESHEET	
		93	70	34	1.20" ABOVE HOT TUBESHEET	
		99	108	27	1.41" ABOVE 5TH COLD LEG SUPPORT	
		106	101	23	21.26" ABOVE COLD TUBESHEET	

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine

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11. ABSTRACT OF CONDITIONS NOTED:

12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

CATEGORY	ITEM	CONDITION				CORRECTIVE MEASURE
B-Q (cont'd)	B16.20 (cont'd)	STEAM GENERATOR #3 TUBING (cont'd)				
IMPERFECT TUBES		ROW	LINE	% TWD	LOCATION	
		1	118	8	0.70" ABOVE HOT TUBESHEET	NOT REQUIRED
		7	20	4	1.69" ABOVE HOT TUBESHEET	FOR LESS THAN
		15	28	6	4.45" ABOVE HOT TUBESHEET	40% TWD
		18	41	6	0.86" ABOVE HOT TUBESHEET	
		19	46	10	1.10" ABOVE HOT TUBESHEET	
		20	41	18	0.83" ABOVE HOT TUBESHEET	
		20	51	9	1.67" ABOVE HOT TUBESHEET	
		21	44	16	2.36" ABOVE HOT TUBESHEET	
		28	95	18	5.82" ABOVE HOT TUBESHEET	
		31	8	18	0.91" ABOVE COLD TUBESHEET	
		31	14	2	20.82" ABOVE 6TH HOT LEG SUPPORT	
		34	103	13	3.92" ABOVE COLD TUBESHEET	
		35	48	11	11.79" ABOVE COLD TUBESHEET	
		38	37	18	6.40" ABOVE COLD TUBESHEET	
		38	95	8	1.65" ABOVE HOT TUBESHEET	
		38	97	17	17.77" 2ND HOT LEG SUPPORT	
				10	29.75" 1ST HOT LEG SUPPORT	
		42	37	1	6.55" ABOVE COLD TUBESHEET	
				17	6.80" ABOVE COLD TUBESHEET	

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine

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11. ABSTRACT OF CONDITIONS NOTED:

12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

CATEGORY	ITEM	CONDITION				CORRECTIVE MEASURE
B-Q (cont'd)	B16.20 (cont'd)	STEAM GENERATOR #3 TUBING (cont'd)				
IMPERFECT TUBES (cont'd)		ROW	LINE	% TWD	LOCATION	
		44	115	10	5.30" ABOVE HOT TUBESHEET	NOT REQUIRED
		49	26	12	1.13" ABOVE HOT TUBESHEET	FOR LESS THAN
		54	29	5	1.67" ABOVE HOT TUBESHEET	40% TWD
		60	29	15	1.73" ABOVE HOT TUBESHEET	
		60	33	10	1.94" ABOVE HOT TUBESHEET	
		60	89	16	1.44" ABOVE HOT TUBESHEET	
		62	81	17	5.08" ABOVE 2ND COLD LEG SUPPORT	
		63	70	12	10.49" ABOVE COLD TUBESHEET	
		68	33	19	0.79" ABOVE HOT TUBESHEET	
		68	71	19	10.26" ABOVE COLD TUBESHEET	
		71	54	13	3.75" ABOVE HOT TUBESHEET	
		74	113	9	4.73" ABOVE 1ST VERTICAL SUPPORT	
		75	44	5	1.23" ABOVE HOT TUBESHEET	
		76	93	12	1.93" ABOVE HOT TUBESHEET	
		79	20	1	0.77" ABOVE HOT TUBESHEET	
		82	79	19	11.83" ABOVE HOT TUBESHEET	
		82	107	17	1.20" ABOVE HOT TUBESHEET	
		84	87	12	2.91" ABOVE HOT TUBESHEET	
		90	81	11	1.42" ABOVE HOT TUBESHEET	
		115	58	16	11.80" ABOVE HOT TUBESHEET	

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner Maine Yankee Atomic Power Company (Name)
Augusta, Maine (Address)
Date 1993 Refueling
Sheet 1 of 18

2. Plant Maine Yankee Atomic Power Plant (Name)
Wiscasset, Maine (Address)
Unit Reactor Vessel 20865
See sheets 3 through 18 Repair Organization P.O. No., Job No., etc.

3. Work Performed by See sheets 3 through 18 (Name)
(Address)
Type Code Symbol Stamp _____
Authorization No. _____
Expiration Date _____

4. Identification of System See sheets 3 through 18.

5. (a) Applicable Construction Code B31.1 19 77 Edition, ___ Addenda, Code Class _____

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements - 1986

6. Identification of Components Repaired or Replaced, and Replacement Components.

Name of Components	Name of Mfr.	Mfrs. S/N	Nat'l Bd. No.	CRN No.	Other ID	Year Built	Repaired Replaced or Replacement	ASME Code Stamped (yes or no)
See sheets 3 through 18.								

7. Description of Work See sheets 3 through 18.

8. Tests Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ]
Other [ ] Pressure _____ psi Test Temp. _____ °F.

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. x 11 in., (2) information in Items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets os recorded at the top of this form.

9. Remarks _____
(Applicable Manufacturer's Data Reports to be attached)

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and this repair & replacement conforms to Section XI of the ASME Code.

Type Code Symbol Stamp _____

Certification of Authorization No. N/A Expiration Date N/A

Signed [signature] (Owner or Owner's Designee) ISI Senior Engineer (Title) 12/17, 19 93 (Date)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Maine, employed by Factory Mutual Engineering of Norwood, MA, have inspected the components described in this Owner's Report during the period December 29, 19 92 to October 13, 1993, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[signature] Commissions Maine 697
Inspector's Signature National Board, State, ~~Province, and Endorsements~~

Date December 21 19 93

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine — DATE: 1993 Refueling; SHEET: 3 of 18

2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine — UNIT: Reactor Vessel 20865

Job No. and Title: W.O. 92-3651, Shock Suppressor RC-HSS-102 Replacement

System: Reactor Coolant (RCS)

Safety Class: 1

Work Performed by: Cianbro Corporation, Pittsfield, ME

Description: A safety injection/residual heat line shock suppressor was removed and replaced with an improved shock suppressor as part of a shock suppressor upgrade program.

Section XI Preservice NDE: The support was VT-3 inspected I.A.W. Category F-A, Item F1.10.

Pressure Test: None required.

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine — DATE: 1993 Refueling; SHEET: 4 of 18

2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine — UNIT: Reactor Vessel 20865

Job No. and Title: W.O. 93-901, Shock Suppressor RH-HSS-102 Replacement.

System: Reactor Coolant (RCS)

Safety Class: 1

Work Performed by: Cianbro Corporation, Pittsfield, ME

Description: A residual heat removal line shock suppressor was removed and replaced with an improved shock suppressor as part of a shock suppressor upgrade program.

Section XI Preservice NDE: The support was VT-3 inspected I.A.W. Category F-A, Item F1.10.

Pressure Test: None required.

1. OWNER: Maine Yankee Atomic Power Company
Augusta, Maine
DATE: 1993 Refueling
SHEET: 5 of 18

2. PLANT: Maine Yankee Atomic Power Plant
Wiscasset, Maine
UNIT: Reactor Vessel 20865

Job No. and Title: W.O. 91-5413 (EDCR 91-27), Valve LD-M-2 Relocation.

System: Charging (CH)

Safety Class: 1

Work Performed by: Cianbro Corporation, Pittsfield, ME

Description: The 2½" valve LD-M-2 was replaced due to valve degradation and relocated to allow reduced radiological exposure during routine maintenance as part of an engineering design change. In addition, a support was added near the new valve location for additional piping support.

Section XI Preservice NDE: The new welds were PT inspected I.A.W. Category B-J, Item B9.21. The new support was VT-3 inspected I.A.W. Category F-A, Item F1.10. Additional inspections per ANSI B31.1 and Yankee General Specifications were also completed.

Pressure Test: The new welds were hydrostatically tested to 1.10 times the nominal operating pressure of 2235 psig at a temperature greater than 40°F I.A.W. IWA-4400, Pressure Test.

1. OWNER: Maine Yankee Atomic Power Company
Augusta, Maine
DATE: 1993 Refueling
SHEET: 6 of 18

2. PLANT: Maine Yankee Atomic Power Plant
Wiscasset, Maine
UNIT: Reactor Vessel 20865

Job No. and Title: W.O. 92-3661, Shock Suppressor RH-HSS-101 Replacement

System: Residual Heat Removal (RHR)

Safety Class: 2

Work Performed by: Cianbro Corporation, Pittsfield, ME

Description: A residual heat removal/low pressure safety injection line shock suppressor was removed and replaced with an improved shock suppressor as part of a shock suppressor upgrade program.

Section XI Preservice NDE: The support was VT-3 inspected I.A.W. Category F-A, Item F1.20.

Pressure Test: None required.

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine — DATE: 1993 Refueling; SHEET: 7 of 18

2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine — UNIT: Reactor Vessel 20865

Job No. and Title: W.O. 92-3756-04, Valve MS-M-161 Replacement

System: Main Steam (MS)

Safety Class: 2

Work Performed by: Maine Yankee

Description: The 6" valve MS-M-161 was removed and replaced with a new valve due to excessive seat leakage.

Section XI Preservice NDE: The new welds were PT and UT inspected I.A.W. Category C-F-2, Item C5.51. Additional inspections per ANSI B31.1 and Yankee General Specifications were also completed.

Pressure Test: The new welds were hydrostatically tested to 1.25 times the design pressure of 985 psig at a temperature greater than 40°F I.A.W. IWA-4400, Pressure Test.

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine — DATE: 1993 Refueling; SHEET: 8 of 18

2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine — UNIT: Reactor Vessel 20865

Job No. and Title: W.O. 92-3563, Shock Suppressor WFPD-HSS-203 Replacement
W.O. 93-900, Shock Suppressor WFPD-HSS-204 Replacement
W.O. 93-898, Shock Suppressor WFPD-HSS-215 Replacement
W.O. 93-899, Shock Suppressor WFPD-HSS-216 Replacement

System: Feedwater (FW)

Safety Class: 2

Work Performed by: Cianbro Corporation, Pittsfield, ME

Description: The main feedwater line shock suppressors listed above were removed and replaced with improved shock suppressors as part of a shock suppressor upgrade program.

Section XI Preservice NDE: The supports were VT-3 inspected I.A.W. Category F-A, Item F1.20.

Pressure Test: None required.

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine — DATE: 1993 Refueling; SHEET: 9 of 18

2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine — UNIT: Reactor Vessel 20865

Job No. and Title: W.O. 93-588 (EDCR 93-2004), Shock Suppressors WFPD-HSS-209, 210, 213, and 214 Replacement

System: Feedwater (FW)

Safety Class: 2

Work Performed by: Cianbro Corporation, Pittsfield, ME

Description: The main feedwater line shock suppressors listed above were analyzed and replaced with rigid struts to correct repetitive shock suppressor misalignment as part of an engineering design change.

Section XI Preservice NDE: The supports were VT-3 inspected I.A.W. Category F-A, Item F1.20.

Pressure Test: None required.

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine — DATE: 1993 Refueling; SHEET: 10 of 18

2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine — UNIT: Reactor Vessel 20865

Job No. and Title: W.O. 92-4399, Valve CH-10 replacement.

System: Charging (CH)

Safety Class: 2

Work Performed by: Maine Yankee

Description: The 4" valve CH-10 was removed and replaced with a new valve due to excessive seat leakage.

Section XI Preservice NDE: The new welds were PT and UT or RT inspected I.A.W. Category C-F-1, Item C5.21. Additional inspections per ANSI B31.1 and Yankee General Specifications were also completed.

Pressure Test: The new welds received a leak test at operating pressure in lieu of a hydrostatic pressure test since the welds were not isolable from the charging pump suction piping. A hydrostatic test could have been performed, but the maximum allowable test pressure would have been 1.25 times design pressure of the suction piping (150 psig). See IWA-2240, Alternate Examinations.

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine — DATE: 1993 Refueling; SHEET: 11 of 18

2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine — UNIT: Reactor Vessel 20865

Job No. and Title: W.O. 92-2905, Valve MS-246 Replacement

System: Main Steam (MS)

Safety Class: 2 (upstream side) and 0 (downstream side)

Work Performed by: Maine Yankee

Description: The 1-1/2" valve MS-246 was removed and replaced with a new valve due to excessive seat leakage.

Section XI Preservice NDE: None required. The new welds were inspected per ANSI B31.1 and Yankee General Specifications.

Pressure Test: The upstream weld received a leak test at operating pressure and a surface exam in lieu of a hydrostatic pressure test because the weld could not be isolated from the steam generator. See IWA-5214(d), IWA-2240, and Code Case N416.

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine
DATE: 1993 Refueling
SHEET: 12 of 18

2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine
UNIT: Reactor Vessel 20865

Job No. and Title: W.O. 92-3757-04, Valve MS-M-255

System: Main Steam (MS)

Safety Class: 2 (upstream side) and 0 (downstream side)

Work Performed by: Maine Yankee

Description: The 4" valve MS-M-255 was removed and replaced with a new valve due to excessive seat leakage.

Section XI Preservice NDE: None required. The new welds were inspected per ANSI B31.1 and Yankee General Specifications.

Pressure Test: The upstream side weld was hydrostatically tested to 1.25 times the design pressure of 985 psig at a temperature greater than 40°F I.A.W. IWA-4400, Pressure Test.

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine — DATE: 1993 Refueling; SHEET: 13 of 18

2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine — UNIT: Reactor Vessel 20865

Job No. and Title: W.O. 92-5184 and 5185, Valves BD-52 and 53 Replacement

System: Steam Generator Blowdown (BD)

Safety Class: 2

Work Performed by: Maine Yankee

Description: The 2" valves BD-52 and 53 were removed and replaced with a new valve due to excessive seat leakage.

Section XI Preservice NDE: None required. The new welds were inspected per ANSI B31.1 and Yankee General Specifications.

Pressure Test: The downstream sides of the valves were hydrostatically tested to 1.25 times the design pressure of 950 psig at a temperature greater than 40°F I.A.W. IWS-4400, Pressure Test.

The upstream sides of the valves received a leak test and a surface exam in lieu of a hydrostatic pressure test because these welds could not be isolated from the steam generator. See IWA-5214(d), IWA-2240, and Code Case N416.

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine — DATE: 1993 Refueling; SHEET: 14 of 18

2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine — UNIT: Reactor Vessel 20865

Job No. and Title: W.O. 92-5855, Valve LD-A-51 Flange Machining

System: Letdown (LD)

Safety Class: 2

Work Performed by: Maine Yankee

Description: A leaking piping flange connected to 3" valve LD-A-51 was removed and skim cuts were taken on the flange surface in order to obtain optimum gasket crush and reduce the potential for future flange leaks. The piping flange was then rewelded back into the system.

Section XI Preservice NDE: None required. The new welds were inspected per ANSI B31.1 and Yankee General Specifications.

Pressure Test: The new welds were pneumatically tested to 1.25 times the design pressure of 150 psig at a temperature greater than 70°F I.A.W. IWA-4400, Pressure Test.

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine — DATE: 1993 Refueling; SHEET: 15 of 18

2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine — UNIT: Reactor Vessel 20865

Job No. and Title: W.O. 93-721, 722,and 569, Steam Trap TR-1, 2, and 3 Replacement

System: Auxiliary Steam (AS)

Safety Class: 2

Work Performed by: Maine Yankee

Description: The 2" steam traps mentioned above and miscellaneous piping were removed and replaced with new steam traps and piping due to trap and piping degradation.

Section XI Preservice NDE: None required. The new welds were inspected per ANSI B31.1 and Yankee General Specifications.

Pressure Test: The new welds were hydrostatically tested to 1.25 times the design pressure of 950 psig at a temperature greater than 40°F I.A.W. IWA-4400, Pressure Test.

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine — DATE: 1993 Refueling; SHEET: 16 of 18

2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine — UNIT: Reactor Vessel 20865

Job No. and Title: W.O. 93-1380, Primary Drain Pump Suction Piping Replacement

System: Primary Drain (PD)

Safety Class: 3

Work Performed by: Maine Yankee

Description: The 4" suction piping for the primary drain pumps was removed and replaced with new piping due to the presence of numerous radioactive "hot spots" in the piping.

Section XI Preservice NDE: None required. The new welds were inspected per ANSI B31.1 and Yankee General Specifications.

Pressure Test: The new welds were hydrostatically tested to 1.25 times the design pressure of 150 psig at a temperature greater than 40°F I.A.W. IWA-4400, Pressure Test.

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine — DATE: 1993 Refueling; SHEET: 17 of 18

2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine — UNIT: Reactor Vessel 20865

Job No. and Title: W.O. 93-3389, Valve BD-T-22 Replacement

System: Steam Generator Blowdown (BD)

Safety Class: 2 (upstream side) and 0 (downstream side)

Work Performed by: Maine Yankee

Description: The 2" valve BD-T-22 was removed and replaced with a new valve due to external valve body leakage and degradation.

Section XI Preservice NDE: None required. The new welds were inspected per ANSI B31.1 and Yankee General Specifications.

Pressure Test: The upstream weld was hydrostatically tested to 1.25 times the design pressure of 950 psig at a temperature greater than 40°F I.A.W. IWA-4400, Pressure Test.

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine — DATE: 1993 Refueling; SHEET: 18 of 18

2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine — UNIT: Reactor Vessel 20865

Job No. and Title: W.O. 91-3240, Valve DR-S-11 Repair

System: Reactor Coolant Drain (DR)

Safety Class: 2 (upstream side) and 0 (downstream side)

Work Performed by: Maine Yankee

Description: The 2" relief valve DR-S-11 was removed for servicing and testing, and reinstalled back into the system.

Section XI Preservice NDE: None required. The new welds were inspected per ANSI B31.1 and Yankee General Specifications.

Pressure Test: The upstream weld was hydrostatically tested to 1.25 times the relief valve setting of 200 psig at a temperature greater than 40°F I.A.W. IWA-4400, Pressure Test.