HEALTH PHYSICS REPORT

for

W. R. GRACE COMPANY Wayne, New Jersey

Radiological Survey Following Decontamination of Facilities

June 18, 1974

Maryanne Mc Clocky Health Physicist

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INTRODUCTION

W. R. Grace Company engaged Applied Health Physics, Inc. of Bethel Park,

Pennsylvania to carry out decontamination of their Davison Chemical Division

facilities located at Pompton Plains, New Jersey. An earlier survey conducted by Applied Health Physics, Inc. personnel revealed contamination of buildings and the property by various thorium - containing materials. See Part II.

Decontamination at the site began on March 11, 1974 and continued through
July 18, 1974.

The goal of the decontamination work was to attain certain limits and conditions prior to the release of these premises for unrestricted use. Appendix A presents these guidelines for unrestricted use.

A qualified consultant, Mr. Paul B. Klevin, was engaged by W. R. Grace Company to provide an expert's opinion on the progress and course of the decontamination, as well as to assure compliance with the state of New Jersey and United States

Atomic Energy Commission regulations.

The overall ground area of the W. R. Grace Plant at Wayne, New Jersey is 6.4 acres. The frontage is on Black Oak Ridge Road with chain link fence boundaries north and south, as shown on Figure 1. A small brook runs east and north of the area, providing the eastern property line.

HEALTH PHYSICS REPORT

for

W. R. GRACE COMPANY Wayne, New Jersey

RESULTS of RADIOLOGICAL EVALUATION

of

THORIUM ORE WASTE DISPOSAL PROBLEM

Prepared by

Maryanne McClosky Health Physicist PART II

(Survey prior to Decontamination)

RESULTS OF RADIOLOGICAL EVALUATION of THORIUM ORE WASTE DISPOSAL PROBLEM

INTRODUCTION

W. R. Grace Company has operated a plant in Wayne, New Jersey for a number of years. The plant is located on Black Oak Ridge Road and has been non-operative for the past few years. Currently, the plant lies empty, except for a few pieces of equipment and materials left onsite. Optical polishing compounds were manufactured at this plant. Some of this raw material contained natural thorium and various rare earths, especially cerium. Possession and use of these naturally occurring radioactive materials (e.g., natural thorium) necessitated that

W. R. Grace obtain a source material license from the U. S. Atomic Energy Commission and comply with US-AEC regulations.

This survey was conducted as a prelude to decontamination, which must be carried out since W. R. Grace Company plans to sell this property. Federal and state regulations specify the conditions and limits which must be achieved prior to release of facilities and equipment for unrestricted use by non-licensed parties. See Appendix A. W. R. Grace has engaged the services of a qualified consultant, Mr. Paul B. Klevin, to advise them as to the procedures they must follow in order to assure compliance with the applicable regulations and safety standards.

RADIATION SURVEY

On December 11, 1973, Applied Health Physics, Inc. was informed by Mr. Klevin and Mr. B. L. Mobley of W. R. Grace Company that the plant contained thorium at various locations within the buildings and had been buried on the site.

Applied Health Physics we requested to survey the buildings and grounds, and to report their findings.

The radiation monitoring started December 12, 1973, with the preliminary survey for alpha and gamma radiation. Instrumentation used by our health physics personnel consisted or a calibrated gas proportional alpha survey meter (Eberline Model PAC-3G) and a portable gamma scintillation spectrometer (Eberline Model PRM 5-3).

The contamination consisted primarily of fine dusts scattered through the buildings, and various isolated pockets of activity; e.g., as in trenches. On-site burial accounted for outdoor activity in varying degrees. A complete outdoor survey could not be accomplished due to inclement weather.

Besides using the forementioned radiation survey meters, our surveys also included the collection of smears to evaluate removable contamination; liquid and sludge samples of various materials were collected for estimates of removable and fixed activities. Maps and drawings made at the time of the sampling indicate precisely where various measurements and samplings of materials were obtained. Results of alpha-gamma radiation survey are summarized for certain locations on Table 1.

Gamma and alpha measurements were taken at surface level. These data are presented in Figures 6 and 7. As indicated in Table 1, surface alpha measurements range from 0 to 88,000 disintegrations per minute. The area of the alpha survey meter probe is 68 cm². This data can only estimate the "fixed" activity, since alpha radiation cannot penetrate even a thin covering of dust or other non-radioactive materials. The actual amount of radionuclides in terms of dpm per gram of contaminated surface may be significantly greater than that measured and reported. Figures 1-5 show conditions at the time of our initial survey, December 12, 1973. Additional photographs were taken January 21, 1974, during the course of our site survey. These photos will be retained on file.

Results of the alpha content of sample are listed in Tables 2 and 3. The highest reading for removable contamination was found to be 1,948 d/m/100 cm², while other values averaged 500 d/m/100 cm². The liquid and sludge samples gave estimates of concentrations of contamination ranging from 2,000 to 10,000 d/m/gram of material. The isotopic content of these samples was determined and found primarily to be natural thorium with traces of radium (0.6 \pm 1 nano-Ci per gram). This was done by gamma spectroscopy; the thorium content of settled dusts inside the plant is 20.0 \pm 1.0 mg Th/g sample.

On January 21, 1974, an employee was seen vacuuming these dust laden areas without respiratory protection and using a bag-type industrial vacuum cleaner which created significant airborne dusts. Mr. Klevin requested the termination of this operation, and the results are shown on Figure 7.

CONCLUSIONS

Waste slags and ores containing thorium are buried at various areas of the plant site. See Figure 6. These locations could not be determined precisely due to lack of sufficient information concerning burial. Furthermore, during January, 1974, earth moving equipment was used to level certain portions of the plant property where source material had been buried. These areas were surveyed January 21 and 22, 1974, and the results are shown in Figure 7. Certain deposits were found to be only partially buried. These locations were detected by gamma scintillation and a few samples were collected. The results of the various radiological surveys are the basis for the following conclusions:

1. The maximum amount of removable alpha radioactivity exceeds within certain portions of the plant as well as on the plant property the recommended limit of 1,000 dpm per 100 cm². Radioactive source materials should be removed, so that acceptable limits are met at these locations. Appendix A contains radioactivity limits for un-

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restricted release of facilities and equipment.

- 2. Clothing, equipment, and fixtures are now in place which harbors radioactive contamination. For example, fans, coveralls, benches etc. which no longer serve a useful purpose should be disposed of in a manner that results in minimal contamination to clean areas and complies with applicable regulations.
- 3. The ground floor of the main building, as well as the floors of smaller places have buildup of thorium concentration due to seepage and deposition over the years. These surfaces should be thoroughly cleansed, or removed, depending on the economics of decontamination, versus feasibility of disposal. The floor conditions in certain areas are so bad as to necessitate disposal.
- 4. Drain lines and trenches were found to have high (16⁴dpm/gram) concentrations of licensed material. These and other water routes are to be areas of attention when clean-up takes place.
- 5. All removal, repackaging or transfer of licensed radioactive materials must be done under the direct supervision of health physics personnel who are experienced in this type of decontamination and waste disposal work.

TABLE 1
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RESULTS OF ALPHA-GAMMA RADIATION SURVEYS

Location	Alpha 2 DPM/100 cm ²	Gamma DPM/100 cm ²	mr/hr
Storage area - 2nd. Level	660	1.55×10^6	0.15
Press room wall - 2nd Level	2.64×10^4	5.18 x 10 ⁶	0.6
7-K Blending & Storage - 2nd Level	1.93 x 10 ³	3.11 x 10 ⁶	0.3
Drying Room - 2nd Level	3.08×10^3	2.07×10^6	0.2
Sulfonation, 1&2 Tank Rooms	1.76×10^4	5.18 x 10 ⁶	0.6
Open Storage area, wall	8.80 x 10 ⁴	1.04 x 10 ⁷	1.1
Storage area, drain	2.20×10^4	6.21 x 10 ⁶	0.8
Waste Treatment Room	1.93×10^4	1.04 x 10 ⁶	0.15
Background (outdoors)	None	6.21 x 10 ⁵	<0.1

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TABLE 2

RESULTS OF ALPHA RADIATION ANALYSES

OF THORIUM BOTTLE SAMPLES

Sample	Area*	Weight (gram)	Total Alpha Activity (d/m/gram)
Number	Alea	(Rr am)	(d/m/gram)
529	1-0-1	0.3843	1.2×10 ⁴
514	1-J	0.4626	7.0x10 ³
522	1-M-1	0.2938	1.6×10 ⁴
504	1-F	0.3304	1.22×10 ⁴
	1-В	1.0586	280
502	1-J	0.4935	3.66×10 ³
512	1-5 1-F	0.0877	8.22x103
505	1-J	0.3001	6.50×103
511	1-J	0.4882	1.43×103
513		0.7708	7.36×103
527	1-0-1		2.23×10 ⁴
528	1-0-1	0.0638	3.47x10 ³
532	1-Q-1	0.1215	3.4/X10
526	1-0-1	0.0262	1.04×10 ⁴
506	1-F	0.0224	5.14×10 ³
531	1-Q-1	0.0679	8.20×10 ³
521	1-M-1	0.0916	2.42×103
523	1-M-1	0.1792	1.95×103
501	1-B	0.0771	6.38x10 ³
510	1-G	0.1095	1.105×10 ⁴
503	1-B	0.0078	7.24×10 ³
530	1-Q-1	0.0538	2.19x103
525	1-0-1	0.4199	119
508	1-G	0.1856	4.69×10^{3}
507	1-G	0.0942	5.43×10^3
509	1-G	0.1995	1.05x104
524	1-M-1	0.8390	1.18x10 ³
537	2-E	1.3249	348
520	2-B	0.0846	7.44x10 ⁴
546	2-F	0.0423	1.81x10 ⁴
539	2-E	0.0925	1.15×104
547	2-H	0.0596	1.48x10 ³
515	2-C	0.1076	1.89×103
545	2-F	0.0670	2.76x103
541	2-E	1.1860	84
543	2-F	0.0929	1.15x103
549	2-I	0.0380	3.75×10 ⁴
518	2-C	0.0306	2.83×10 ³
548	2-I	0.0581	2.13×10^{3}
533	2-D	0.1035	610
536	2-E	1.1921	519
544	2-F	0.1916	30.5
535	2-D	0.0276	2.44×10 ³
516	2-C	0.0352	1.14×103
542	2-F	2.0742	463

TABLE 2, Continued

Sample Number Area*		Weight (gram)	Total Alpha Activity (d/m/gram)	
534	2-D	0.0145	4.85×10 ³	
538	2-E	0.8977	5.05x103	
462	SOIL-2K	0.0465	1.35x103	
554	SOIL-2K	0.2396	198	
552	SOIL-2K	0.0127	1.80x10 ³	
551	SOIL-2K	0.0711	1.03x10 ⁵	
461	SOIL-2K	0.1337	0	
519	2-C	1.3668	115	
517	2-C	1.3013	2.14x103	
555	SOIL-2K	0.5752	925	

NOTES: 1) The areas can be found on the individual drawings.

²⁾ The standard deviations of the activities are within 2 sigma.

TABLE, 3

RESULTS OF ALPHA RADIATION ANALYSES OF THORIUM SMEAR SAMPLES

Sample Number	Area*	Total Alpha Activity (d/m/100cm ²)
285	BLANK	1.9
287	1-A	17.1
289	1-A	3.8
291	1-B	0
292	1-B	22.8
293	1-B	22.8
295	1-B	11.4
297	1-B	15.2
299	1-B	26.5
301	1-B	9.5
303	1-B	13.3
305	1-B	17.1
307	1-B	22.8
309	1-C	19.0
311	1-C	17.1
313	1-D	9.5
	1-D	9.5
315		24.7
317	1-D	
319	1-F	9.5
321	1-F	9.5
323	1-F	9.5
325	1-G	81.5
326	1-G	157
327	1-G	165
328	1-G	150
329	1-G	74
330	1-G	72.1
331	1-G	302
332	1-Н	32.4
334	1-Н	24.7
336	1-Н	28.5
338	1-I	17.1
340	1-I	9.5
342	1-J	74
343	1-J	17.1
344	1-J	74
345	1-J	28.5
346	1-J	26.5
347	.1-J	74
348	1-J	26.5
349	1-J	30.5
350	1-J	20.9
351	1-J	19
352	1-J	30.5
353	1-J	28.5
359	1-K	64.7
360	1-K	60.8

Sample Number	Area*	Total Alpha Activity (d/m/100cm ²)
361	1-K	53.2
362	1-K	49.5
364	1-L	156
365		160
	1-L	
366	1-L	47.5
367	1-L	13.3
368	1-L	9.5
369	1-L	11.4
370	1-1	62.7
371	1-M-1	57.2
372	1-M-1	57.2
373	1-M-1	57.2
374	1-M-1	38.0
375	1-M-1	34.2
376	1-M-1	36.1
377	1-M-1	49.5
378	1-M-1	45.5
379	1-M-1	45.5
380	1-M-1	45.5
381	1-M-1	37.6
382	1-M-1	30.5
383	1-M-1	38.0
384	1-N-1	24.7
385	1-N-1	15.2
386	1-N-1	3.8
388 -	1-N-1	11.4
389	1-N-1	15.2
391	1-0-1	30.5
394	1-0-1	28.5
395	1-0-1	30.5
396	1-0-1	30.5
397	1-0-1	34.2
398	1-0-1	15.2
399	1-0-1	11.4
400	1-0-1	15.2
402	1-0-1	30.5
403	1-P-1	20.9
404	1-P-1	26.5
405	1-P-1	47.5
409	1-Q-1	77.9
410	1-Q-1	34.2
411	1-Q-1	41.8
412	1-Q-1	34.2
413	1-Q-1	224
414	1-Q-1	116
415	1-Q-1	169
416	1-Q-1	98.6
417	1-R-1	24.7
418	1-R-1	9.5
419	T 11 T	15.2

TABLE 3, Continued

Sample Number	Area*	Total Alpha Activity (d/m/100cm ²
420	1-R-1	24.7
421	1-R-1	30.5
422	1-R-1	93
423	1-R-1	176
424	1-R-1	76
425	1-R-1	123.5
426	1-R-1	43.8
427	1-R-1	39.9
428	1-R-1	15.2
429	1-R-1	24.7
430	1-R-1	13.3
431	1-R-1	17.1
432	1-R-1	17.1
439	2-A	358
440	2-A	103
441	2-A	224
449	2-B	795
450	2-B	1,948
451	2-B	402
452	2-В	64.7
445	2-C	65.7
446	2-C	74
447	2C	38
448	2-C	19
454	2-D	30.5
456 .	2-J	3.8
459	2-J	3.8
460	2-J	9.5
463	2-K	13.3
464	2-K	15.2
465	2-K	11.4
466	2-K	13.3

TABLE 4

ANALYSIS OF SAMPLES TAKEN FROM W. R. GRACE COMPANY (The collection points are noted on Figure 6)

Sample	Dry Weight(grams)	Net CPM	DPM/GRAM	uCuries/GRAM
#1 Dump Sump	2.3243	1,851.1	1.50x10 ³	6.8x10 ⁻⁴
#2 Ball Mill Sump	3.2258	1,609.6	949	4.2x10 ⁻⁴
#3 Driveway Sump	1.4697	1,479.6	1.89x10 ³	9.3x10 ⁻⁴
#4 Well Sump	1.1283	658.1	1.14x10 ³	5.2x10 ⁻⁴
#4 Well Sump	1.1283	658.1	1.14x10 ³	5.2x10 ⁻⁴

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APPENDIX A

RADIOACTIVITY LIMITS FOR UNRESTRICTED RELEASE

of

FACILITIES and EQUIPMENT

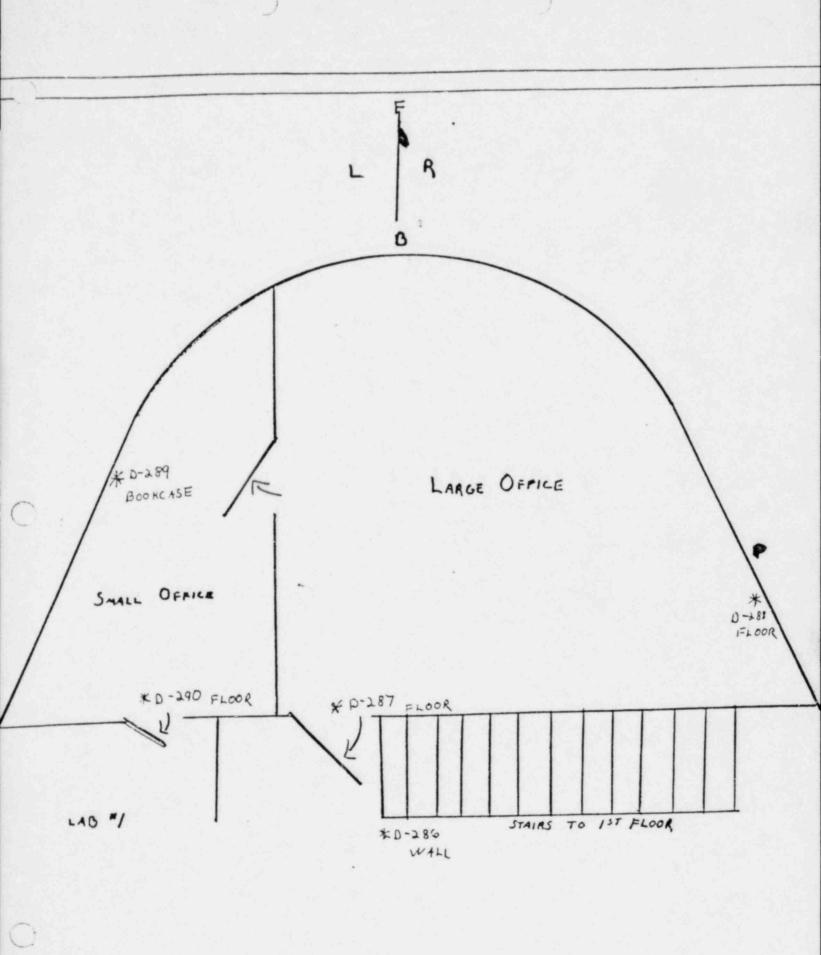
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- The maximum amount of fixed alpha radioactivity in disintegrations per minute per 100 square centimeters on buildings or equipment should not exceed 25,000 dpm.
- 2. The average amount of <u>fixed</u> alpha radioactivity in disintegrations per minute per 100 square centimeters on buildings or equipment should not exceed 5,000 dpm.
- 3. The maximum amount of <u>removable</u> (capable of being removed by wiping the surface with a filter paper or soft absorbent paper) alpha radioactivity in disintegrations per minute per 100 square centimeters on buildings or equipment should not exceed 1,000 dpm.
- 4. (a) The <u>maximum</u> level at one centimeter from the most highly contaminated surface of a building or piece of equipment measured with an open-window beta-gamma survey meter through a tissue equivalent absorber of not more than seven milligrams per square centimeter should not exceed 1.0 millirad per hour.
 - (b) The average radiation level at one centimeter from the contaminated surface of the building or equipment measured in the same manner should not exceed 0.2 millirad per hour.
- 5. The contamination limits for abandonment of facilities involving U-233 or plutonium should not exceed 1/10 of the limits in items 1,2 and 3 above.
- NOTES: A. A reasonable effort should be made to minimize the contamination present.
 - B. Surfaces of premises, equipment or scrap likely to be contaminated, and of such size, construction, or location as to make the surface inaccessible for purposes of measurement, shall be presumed to be contaminated in excess of the levels specified above.
 - C. Premises, equipment or scrap having contaminated surfaces which have been covered by painting, metal plating or other covering material should be presumed to be contaminated in excess of the levels specified above, unless it can be established that the contamination was below the above levels prior to applying the covering.

APPENDIX B

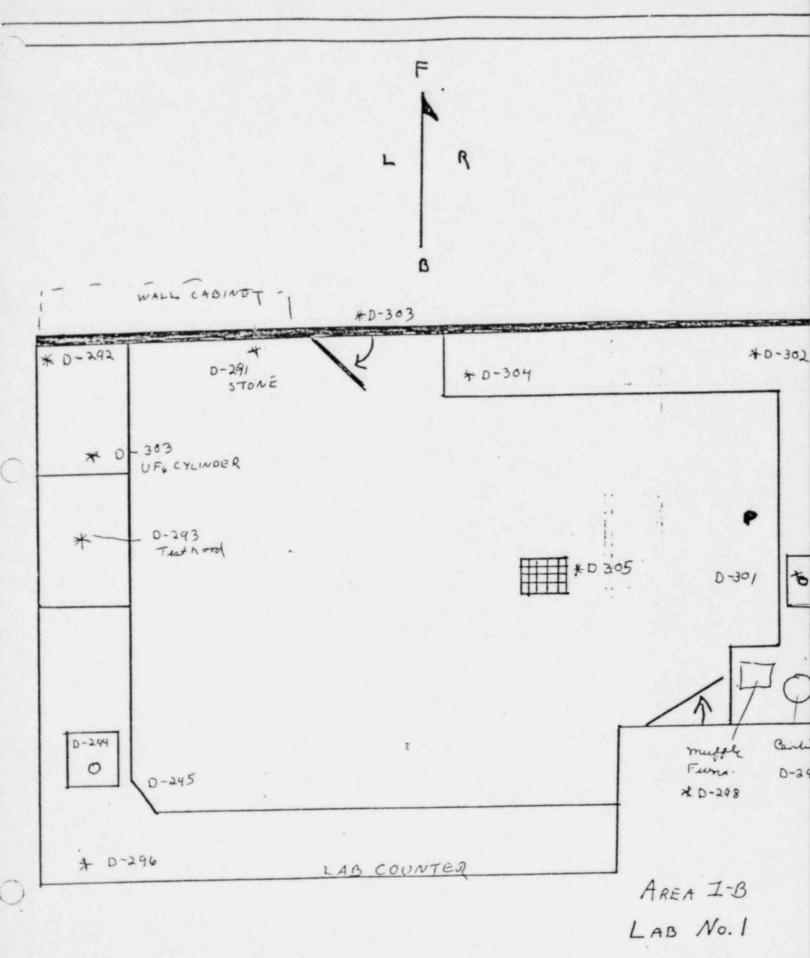
Locations of Various Samples taken at W. R. Grace Company

NOTE: These locations can be found on Figure 7

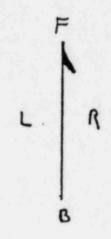


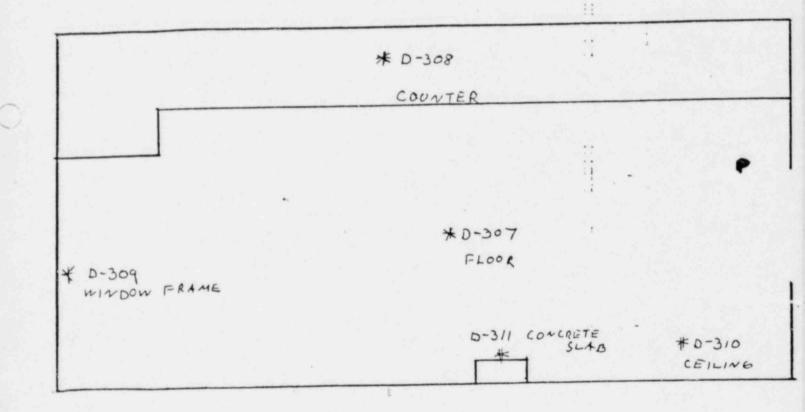
OFFICES

2ND FLOOR, MAIN BL

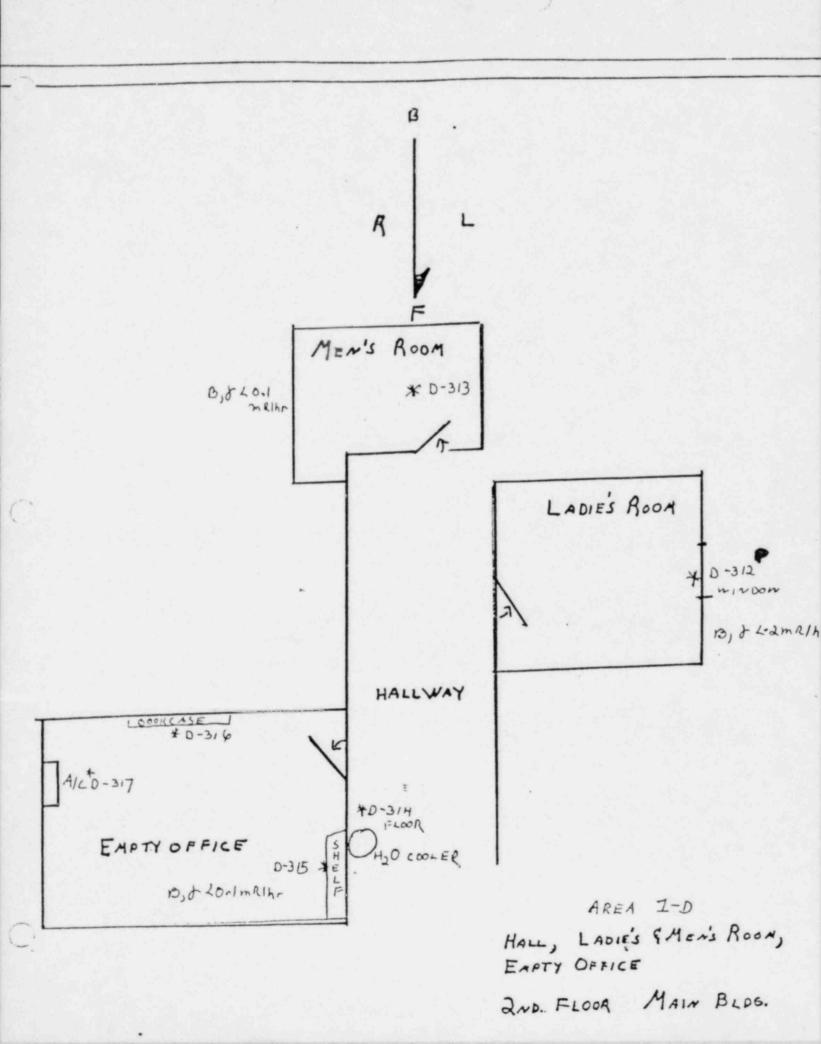


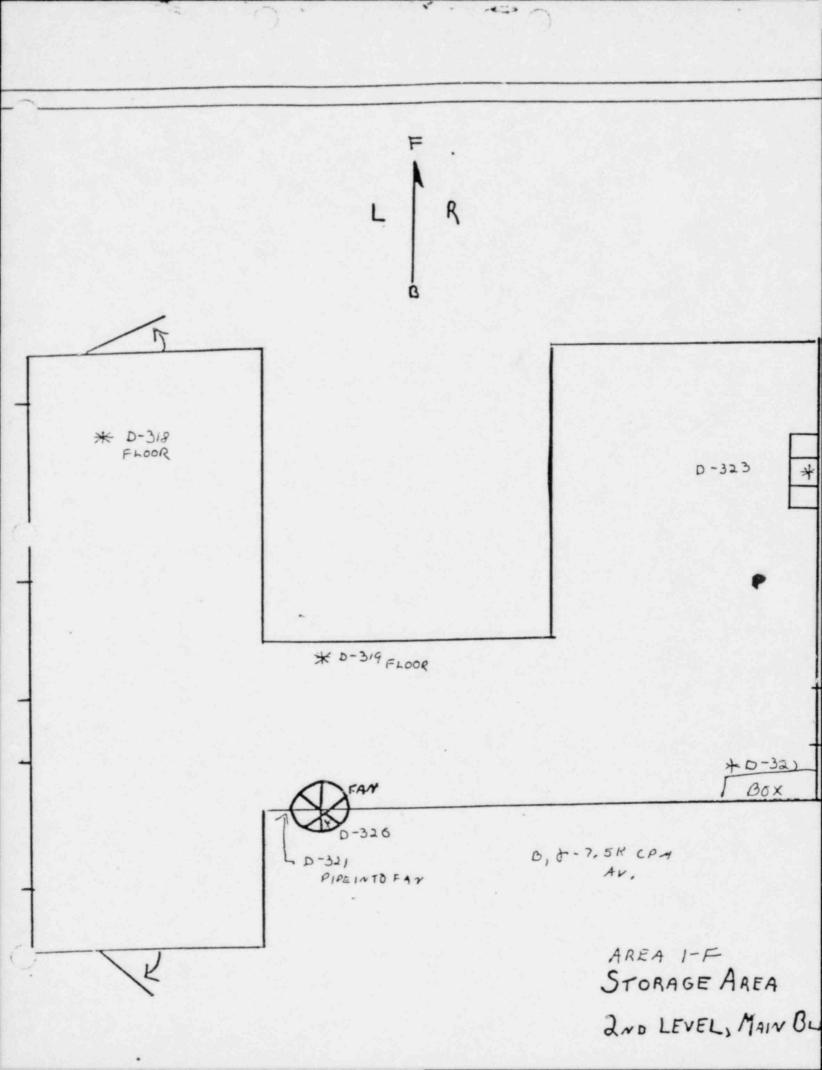
2ND. FLOOR, MAIN BI

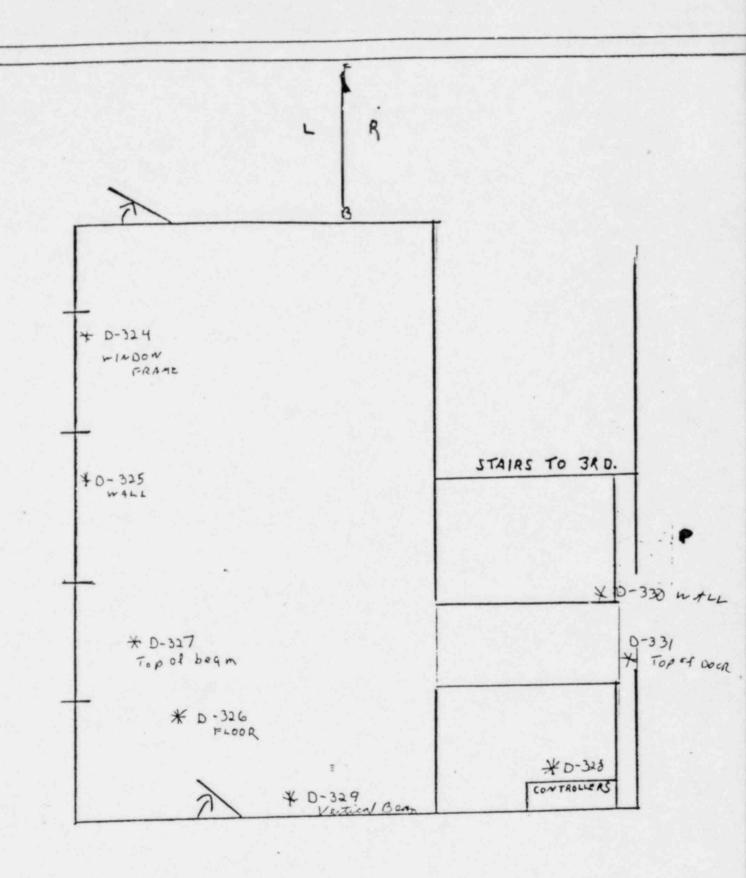




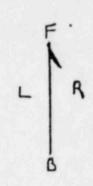
AREA I-C SAMPLE PREPARATION ROOM 2ND FLOOR, MAIN BLOG.

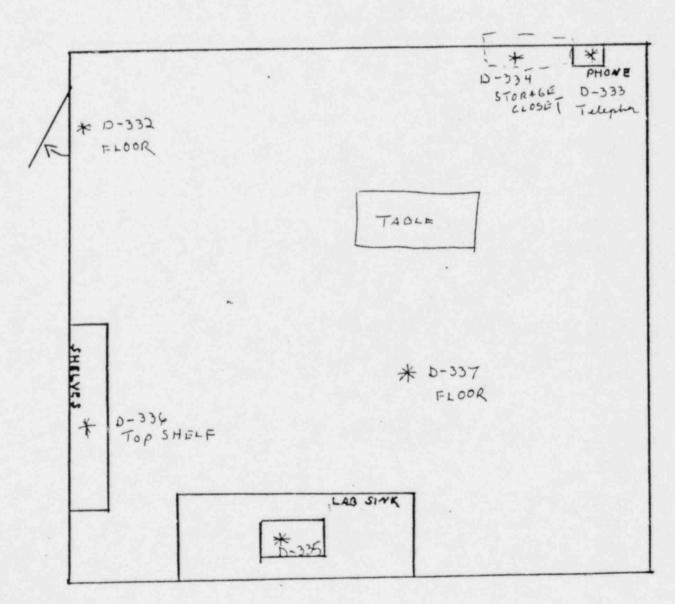






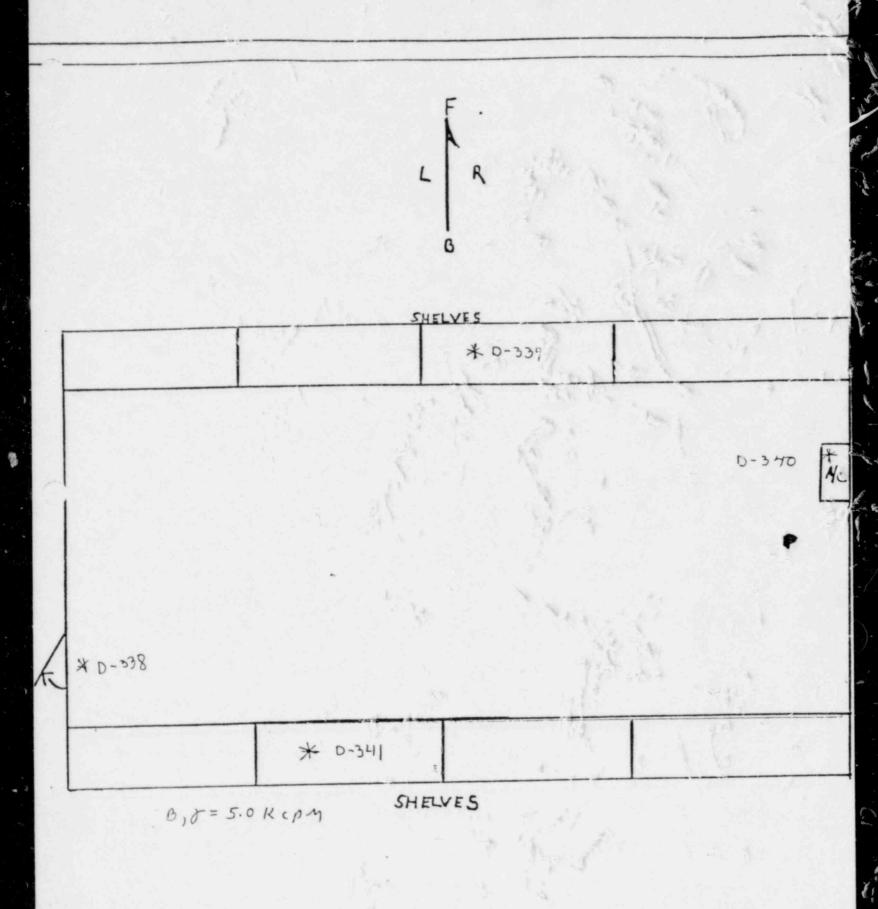
AREA I-G PRESS ROOM & SULFONATION ROO ZNA FLOOR, MAIN BLOG.



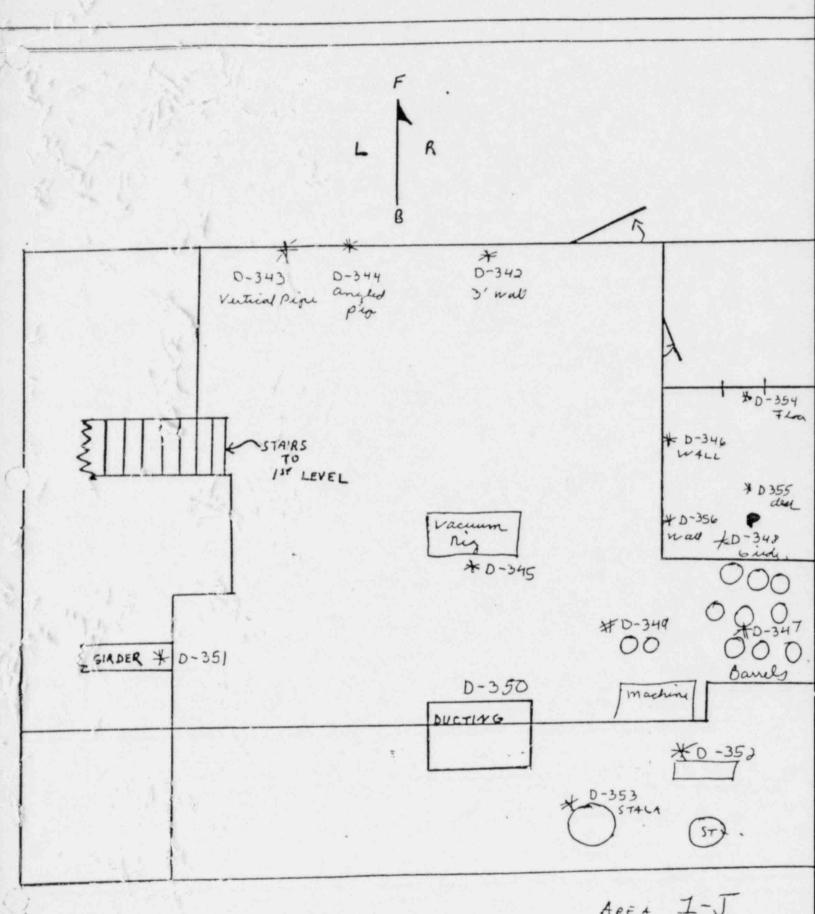


TEST LAB *2

2ND LEVEL, MAIN BL



AREA I-I CONFERENCE ROOM */ 2ND. FLOOR, MAIN BLOG.



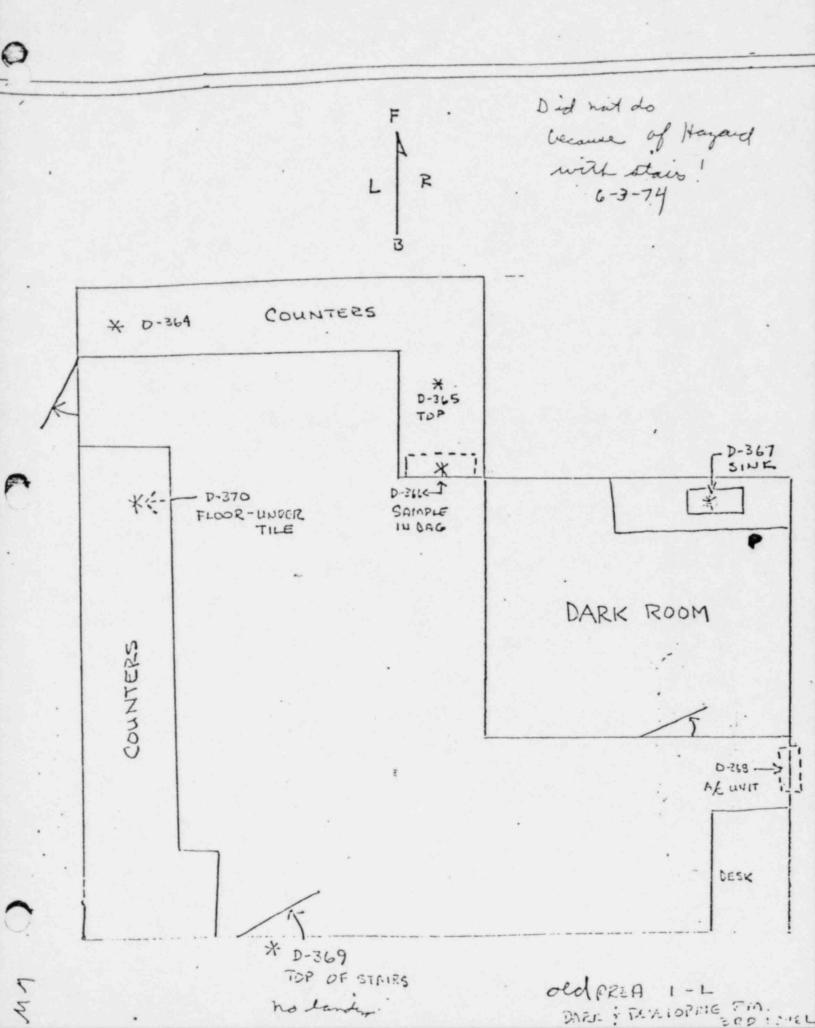
AREA I-J 7-K BLENDING ESTORAGE 2ND LEVEL, MAIN BLD

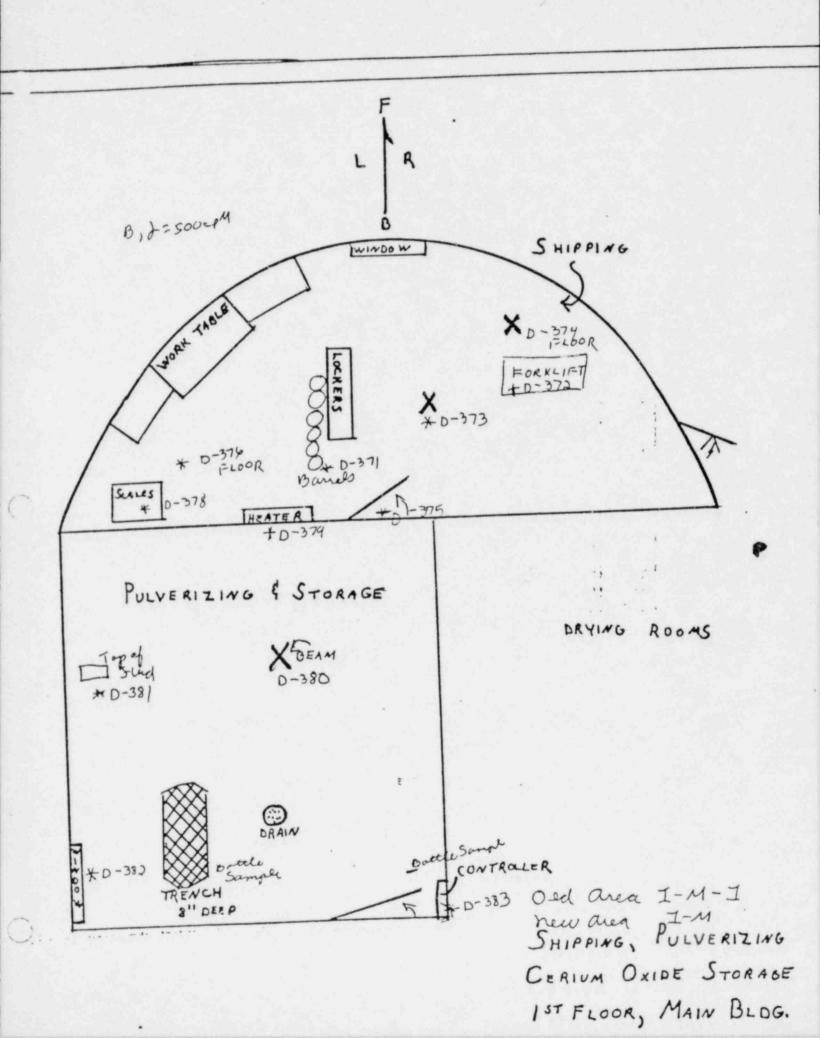
0 Did not do with stais! OPEN-BEAMS D-362 PANEL COMMECTORS Box SHELVES SHELVES D-359 -SHELVES SUPPORT ECAM D-761 3 SHELVES TOP D-360 - + SHELVES DEVELOPING ROOM ← D-363 FROOR

-4.3

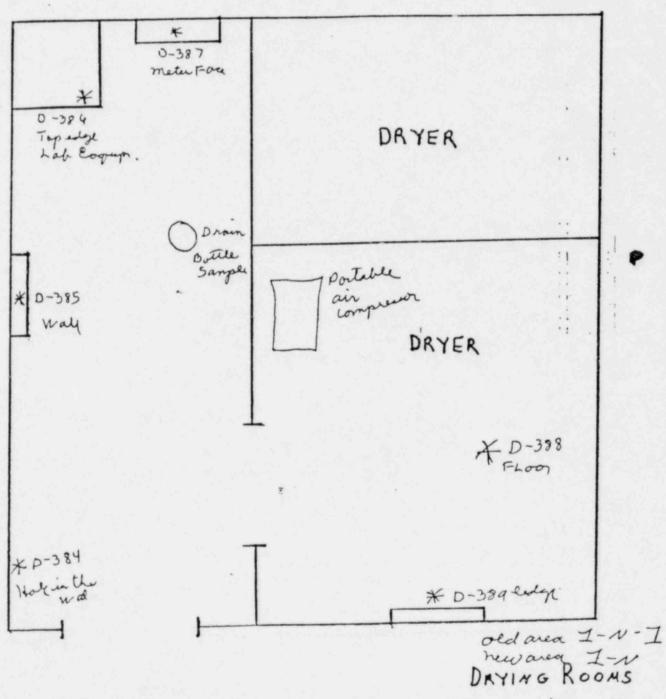
GENERAL AREA

old Aren 1-K

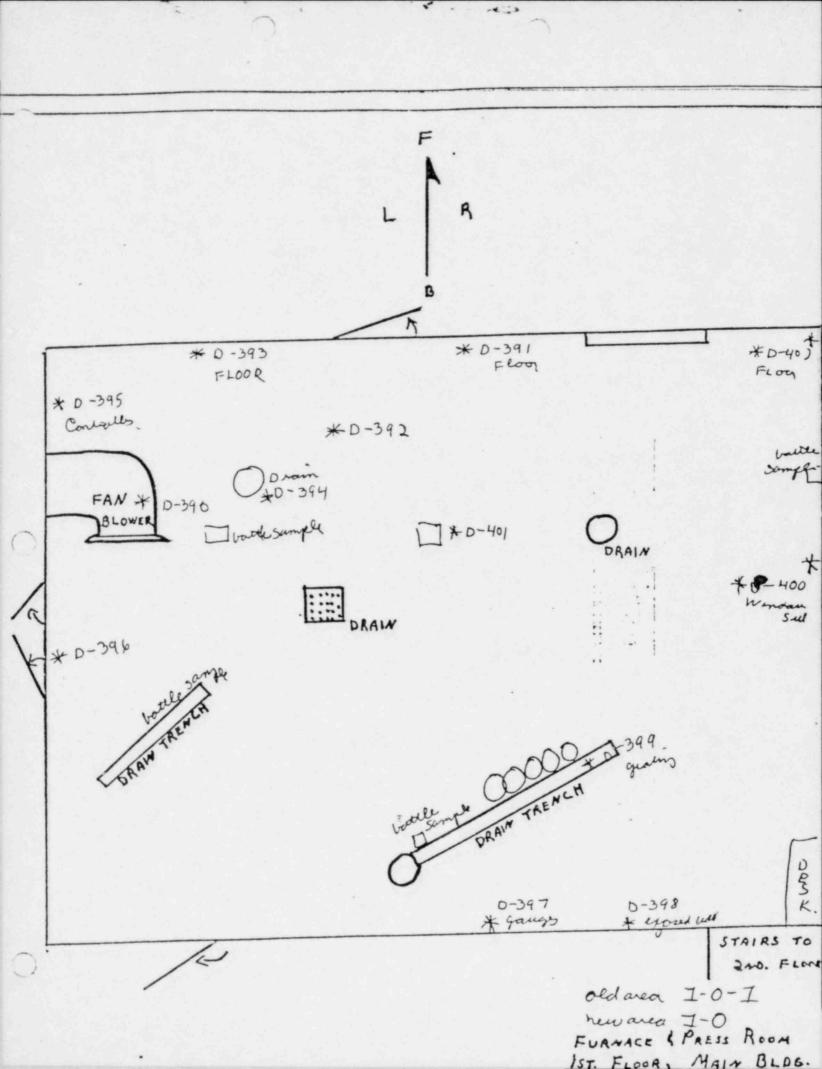




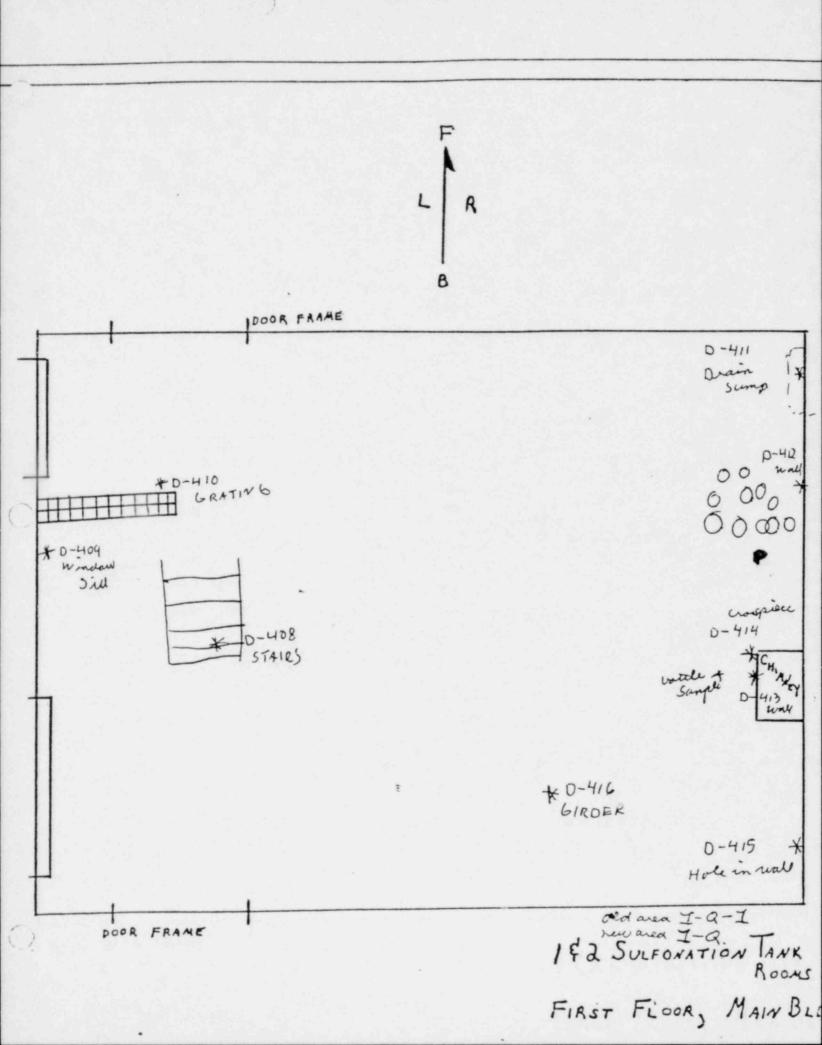


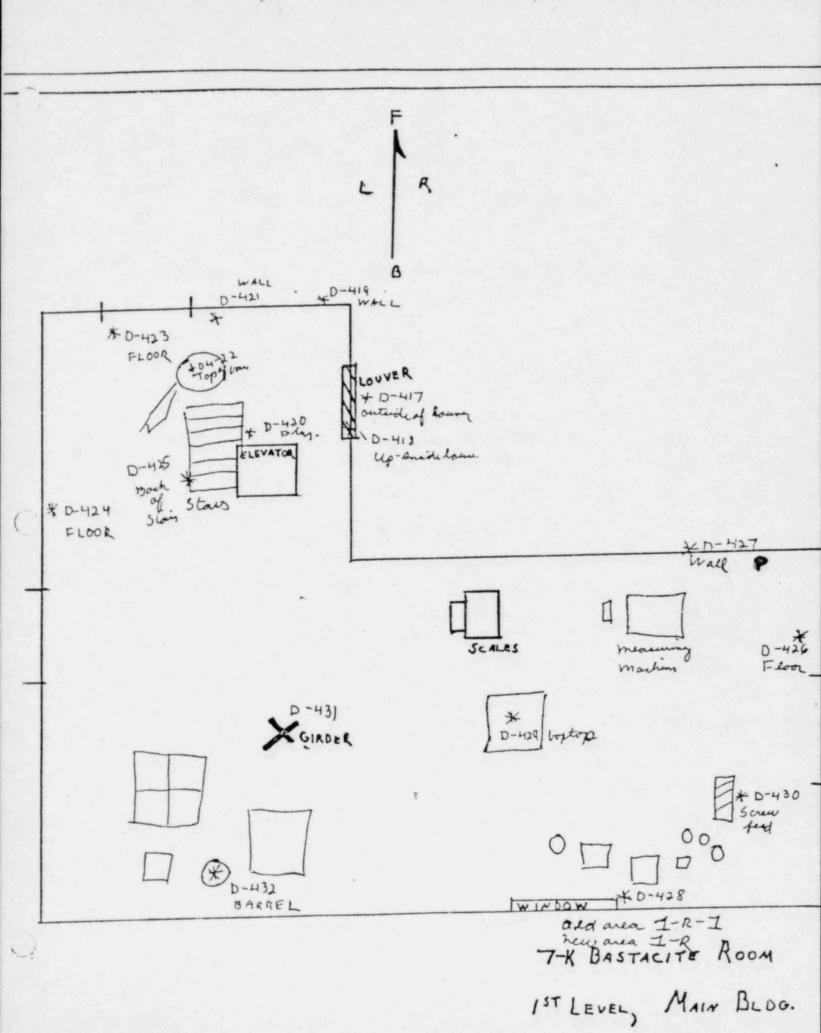


IST. FLOOR MAIN BLDG.



* D-405 in piping D-404 Floor D-403 K wale * D-406 on collecta. * Don Jaml SHARPLES COLLECTOR ROC 1 ST FLOOR, MAIN BLOO



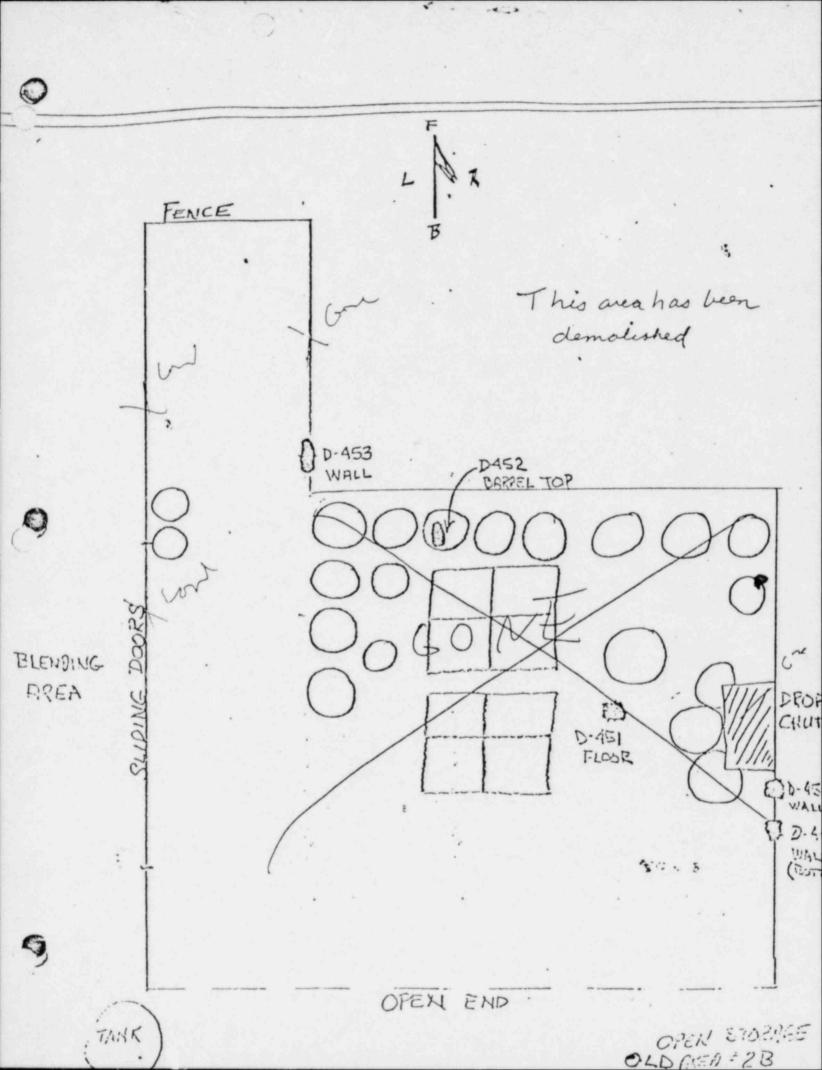


R MACHINE SHOP BOILER ROOM SHOWER TOILET SINK SHOWER Meter Survey Only LOCKERS FLOOR LOCKERS new area I-S-I LOCKER & BOILER ROOMS MACHINE SHOP

IST. FLOOR, MAIN BLOG

Towel Roch + D-443 SHELVES WORK TABLE * 0-440 FLOOR WIRE CAGE WORK TABLE + D-439 5'5" wall 节

> AREA 2-A WORKSHOP BUILDING

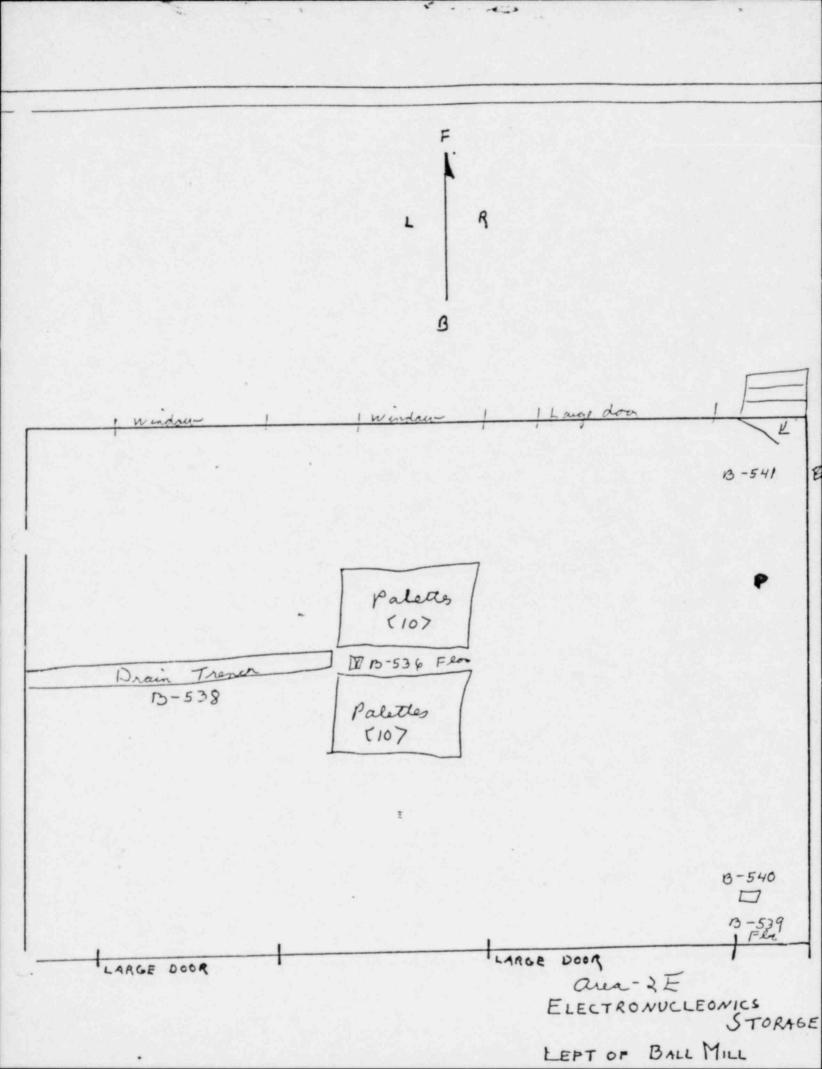


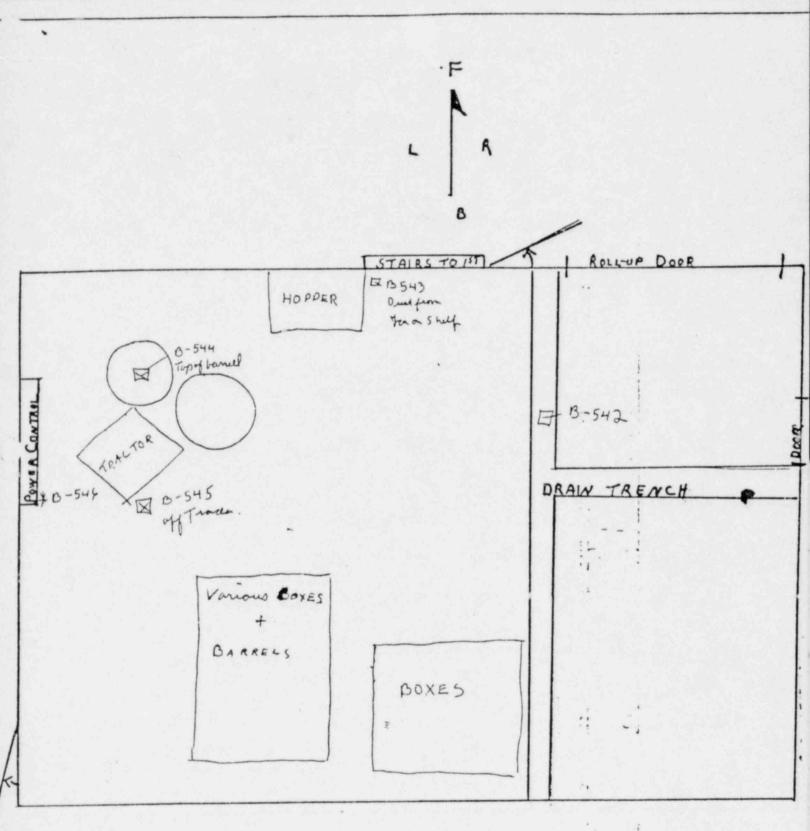
· 4:3 (Bulding demolisks, STEEL GIRCER HEATER 6.5' ELEV 3.0' WALL B-518 GIRDER B-519 SCALE TRENCH B-515 FLOOR D-448 VERTICAL STUD 图 8-517 DRAIN TRENCH D-447 BLENDER TOP of BEAM D 3-516 FLOOR 449 5 D-445 il.L FLOOR This building. has been demoleshed Oldarea 2-C 444 TRAIS FURT PARISHELL WILL

Buildry Denoleste D-455 WALL D-454 WALL MACHINE 8-533 FLOOR 图 B-534 FLUDR 3-535 TOP OF TRAPPEL This area was demoleshed Old area #2-D

nn

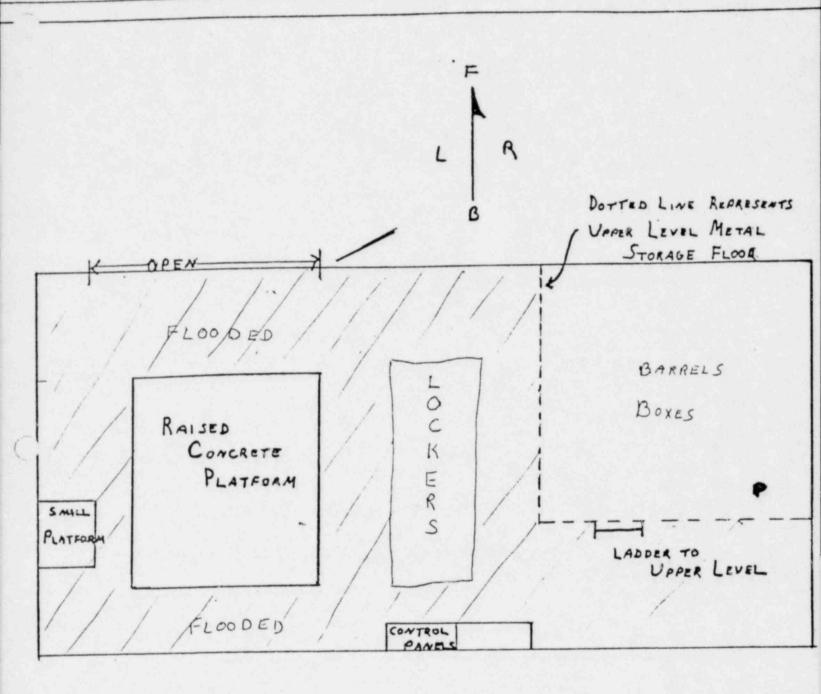
STORIGE PLEA FORMS





· C.3

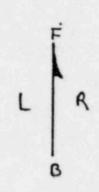
DLD AREA 2F NEW AREA 2-C BALL MILL 2ND LEVEL STORA

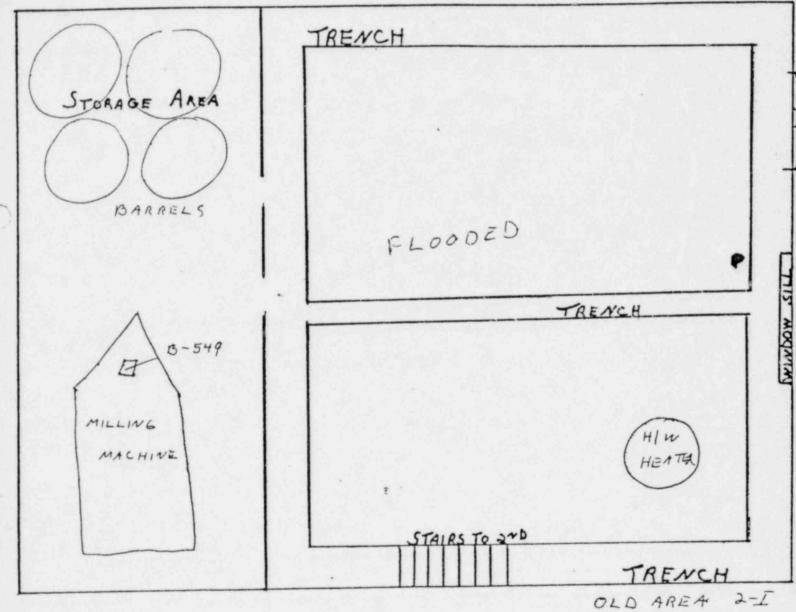


OLD AREA 2-G NEW (LEVEL) 2-D AREA

COMPRESSOR ROOM

0 Good = no more THIS AREA LOCKED AND NAILED SHUT BARREL HETER SURVEY! TO IST LEVEL AREA 2H = 21-10-15 K OFM BY East will 400 CPM 9 tuese sus bus





OLD AREA 2-I NEW AREA 2-C

BALL MILL LOWER OPERATION

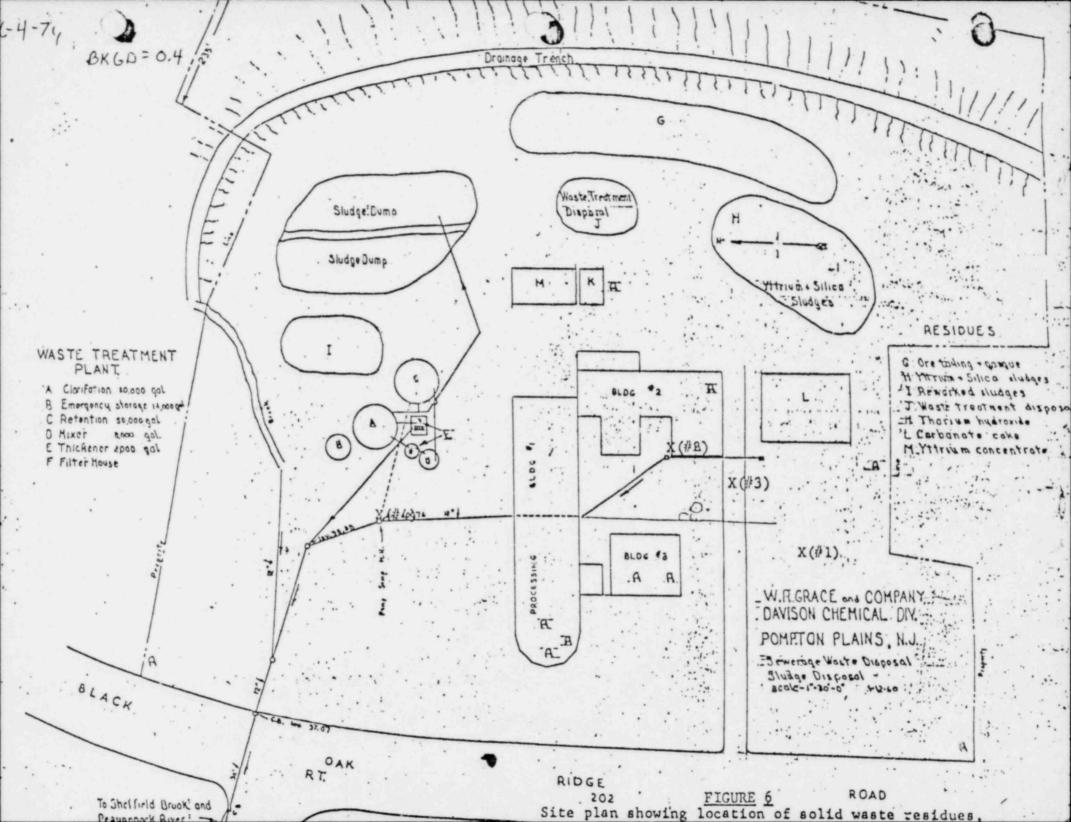
This area has been demolished 000 D-457 ON BEAM IN PIT D-45L WALL O D-4CO TANK TANK D-459

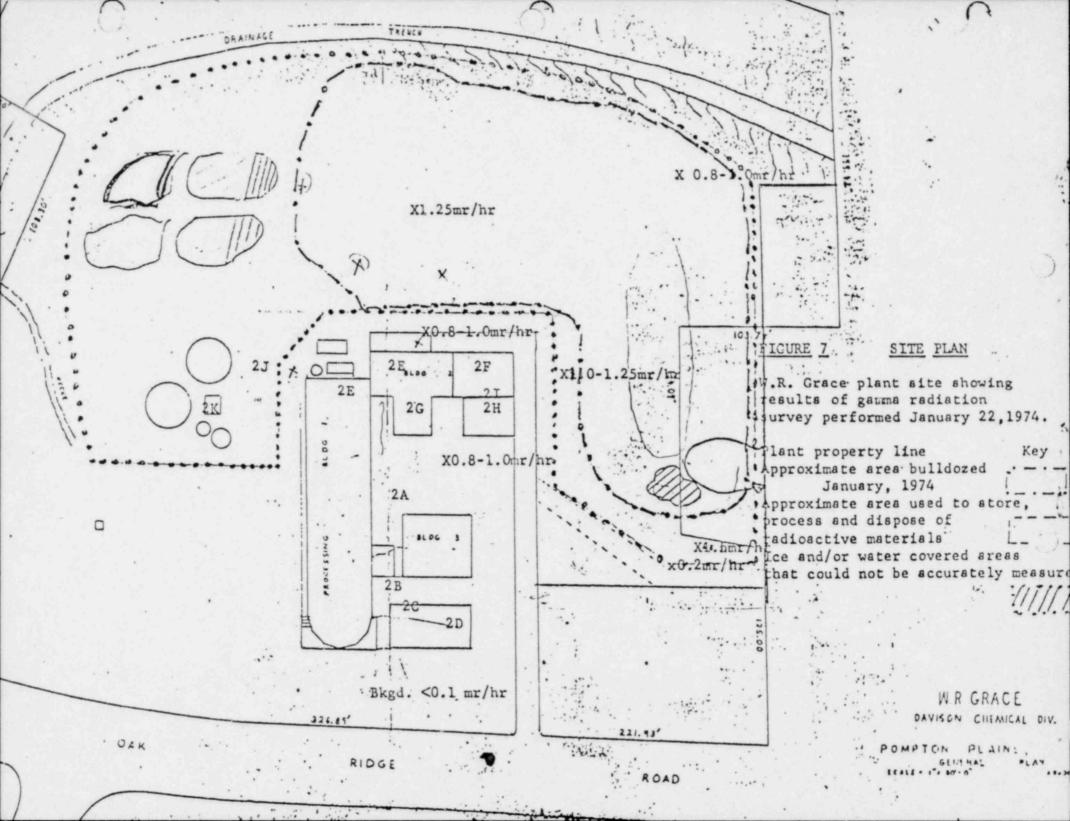
AREA-2J

Where Treathent Phat

UN.

0 This area was demolished PARKING not then LOT 2.5 K CPM WASTE TREATMENT 18 =7 BLOG. CARPORT TRAILER 5.0 K CPM 4,8 2.5 KCFM ZISK CEM B& . 18,81 SHADED AREA 10-15 KC ALDNG ERU 中午 # 2 AREA 2K





DECONTAMINATION

The various areas at the W. R. Grace Plant are described below in relation to what type of decontamination work was performed on them. The reference to the area, such as 1-A, is in keeping with the nomenclature found later in Part IV of this report. Mr. Stephen M. Sorensen of Applied Health Physics, Inc., supervised work at the site. Respiratory equipment was worn where necessary to insure proper health and safety protection of personnel. Air samples were periodically taken to determine air concentrations of natural thorium.

W. R. Grace's Offices (1-A) - These rooms were vacuumed cleaned of natural thorium with an X-100 absolute filter.

Lab #1 (1-B) - The following were removed from this room: muffle furnace, centrifuge, UF₆ cylinder and a stone. The lab was vacuumed, damp wiped with water, spray detergent and paper towels. The floor was damp mopped with detergent and water.

Sample Preparation Room (1-C) - This room was vacuumed, damp wiped and damp mopped.

Hallway, Restrooms and Office (1-D) - The water cooler in the hallway was removed. The area was then vacuumed, damp wiped and mopped.

Grinding and Polishing Test Lab (1-E) - This lab is still in use and was only vacuumed cleaned.

Storage Level (1-F) - Equipment was removed as part of the decontamination. The first layer of fiberboard flooring on the left side catwalk was removed. The walls, ceiling, and remaining floor were cleaned with a high pressure steam cleaner and detergent.

Press and Sulfonation Rooms (1-G) - The left front floor was removed as indicated on the diagram. Brickwork under the windows was removed. The area was completely steam cleaned and finished with a final high pressure washing.

Test Lab #2 (1-H) - A sink was removed after attempts to clean it failed.

The room was damp washed, vacuumed, and wiped.

Conference Room #1 (1-I) - This room was vacuumed, damp wiped and mopped.

7-K Blending and Storage (1-J) - All equipment and furniture were removed from rooms on the right front side. Barrels and equipment were removed from the rest of the area. This area was cleaned with steam and high pressure water before a final wipedown.

Conference Room #2 (1-K) - This room was vacuumed, damp wiped and mopped.

Hallway (1-L) - This area was vacuumed and damp mopped. The stairs to

the first level were steam cleaned.

Shipping, Pulverizing, Cerium Oxide Storage (1-M) - These rooms were vacuumed and the shipping room is still in use.

Drying Rooms (1-N) - A catch box from the upstairs labs was removed and buried, along with another box. These rooms were vacuumed cleaned and the hallway was washed.

Furnace and Press Room (1-0) - This room now contains electronucleonics equipment. Other equipment was removed as part of the decontamination. The drain trenches were dug out by hand and flushed with water. A fan blower and piping were removed and sent to South Pittsburg, Tennessee (a W. R. Grace facility). The shaded area on the diagram in Part IV shows where approximately 1/8 to 1/4 inch of concrete was removed with electric chisels. The walls and ceiling were steam cleaned. The floors were cleaned with high pressure water.

Sharples Collection Room (1-P) - Equipment was removed for cleaning, then replaced. The walls and floors were washed with high pressure water.

182 Sulfonation Tank Rooms (1-Q) - All equipment, the stairs, and two steel beam crosspieces were removed and buried. Concrete flooring in 7/8 of this area was removed to a level of 34 inches below the building footer. All drains going to the left outside of the area were enlarged with electric chisels. Brickwork on the left wall was removed to a height of 5.5 feet. Brickwork on the backwall was removed to a height of 10 feet. Brickwork on the chimney and backwall was removed up to the second level of the building. The front wall remains intact. The complete area was hydroblitzed after removal of debris. The floor was backfilled with clean soil for safety purposes. Demolition and cleaning in this room lasted approximately 3 weeks due to mashing of the sources of radioactivity.

7-K Bastacite Room (1-R) - All equipment was removed with the exception of the scale and screw feeds. Brickwork on the forward right wall was removed to a height of 3 feet with an electric chisel. Brickwork on the back left wall between the doors was removed to a height of 6 feet. The shaded areas on the diagram in Part IV show where 1/8 inch of concrete was removed with an electric chisel. The screw feed was completely dug out and hydroblitzed.

Locker and Boiler Rooms (1-S) - These rooms were washed with a hydroblitz and the boiler room was vacuum cleaned.

Workshop Building (2-A) - Shelves, a towel rack and cabinet were removed.

This building was washed with a hydroblitz. Approximately 1/8 inch of concrete was removed as indicated in Part IV.

Ball-Mill (2-B) - All equipment was removed and anything was buried that was to be discarded by W. R. Grace. The drain trenches were dug and jack-

hammered to a depth of 2 feet. Shaded areas on the figure in Part IV indicate where concrete was chipped. The whole area was vacuumed and washed. Concrete on the wall below the rollup door on the front portion was removed with electric chisels.

Ball Mill (2-C) - All equipment and boxes were removed. The second level was completely taken out. Drain trenches were jackhammered to a depth of 2 feet and widened. All debris in this building was removed. The complete area was hydroblitzed.

Compressor Building (2-D) - Lockers, barrels, boxes, and equipment found in this area were removed. As indicated in Part IV, concrete in the shaded areas was removed a depth of 1/8 to 1/4 inch with electric chisels. The complete area was hydroblitzed.

Attic, Third level of Main Building - All debris, equipment, boxes, barrels, with the exception of metal shelving were removed. All areas were vacuumed clean and damp mopped.

Electronucleonic Storage - This area was jackhammered and hydroblitzed.

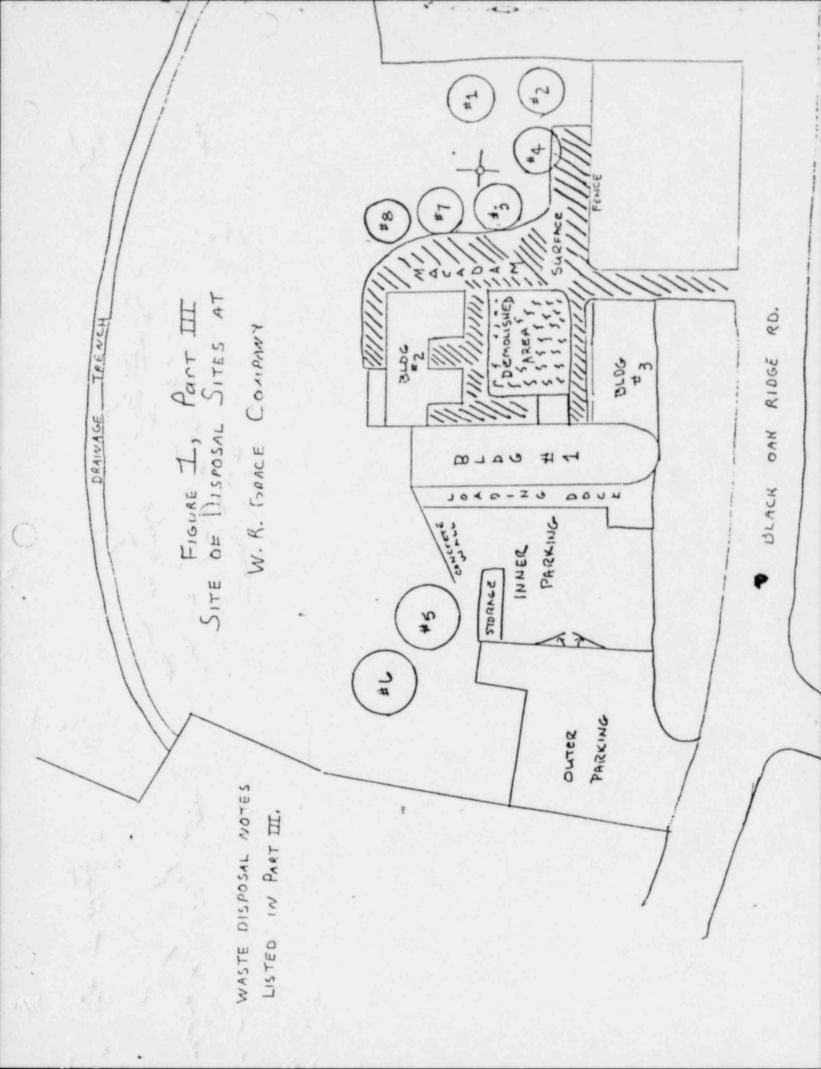
Material found below the concrete floor was removed.

On-site Waste Disposal

W. R. Grace Company received permission to bury material disposed of in the decontamination onsite. The clearance granted by the State of New Jersey was for the burial to be in eight holes, each containing a maximum of no more than 997 pounds per hole. Figure I of Part III shows where the burial sites are located on the premises of W. R. Grace. All holes are 10 feet in diameter, 20 feet deep and spaced 6 feet apart. As part of burial, the holes are to be covered with 4 feet of topsoil. The tanks and waste treatment building on site were demolished and buried along with all debris and sludge resulting from the decontamination work.

Digging and filling of each hole proceeded as follows:

- Hole #1 dug and filled between April 22-26, 1974 by J. Baum Inc. with 100 pounds of materia?.
- Hole #2 dug and filled between April 22-26, 1974 by V. Ottilio & Sons, Inc. with 75 pounds of material
- Hole #3 dug and filled by V. Ottilio & Sons, Inc. between April 22-26, 1974 with 75 pounds of material.
- Hole #4 dug and filled on May 1, 1974 with 50 pounds of material by J. Baum, Inc.
- Hole #5 dug and filled by J. Baum, Inc. on May 21, 1974 with 500 pounds of material.
- Hole #6 dug and filled with 700 pounds of disposal material on May 22, 1974, by J. Baum, Inc.
- Hole #7 dug and filled June 1 through 10, 1974 by J. Baum, Inc. with 100 pounds of material.
- Hole #8 dug and filled June 10 through 28, 1974 by J. Baum, Inc. with P



Radiation Survey after Decontamination

On June 3 through 5 and on June 27, 1974 a final radiation survey was performed following decontamination of the W. R. Grace Company plant site at Wayne, New Jersey. The main building, three of the smaller areas, and part of the grounds were ready for inspection by Applied Health Physics, Inc. personnel. Decontamination was still in progress, due to unforeseen developments. This survey was completed on June 27, 1974 when all work was finished. Certain areas found to contain radioactive materials were resurveyed after further cleaning. These "hot" areas, noted both on Table 1 and certain parts of Figure 1 were cleaned and are now found to be within acceptable limits.

Some of the buildings surveyed prior to decontamination were torn down entirely; this is the result of heavy non-removable thorium— paring material depositions and ground depositions. The demolished structures are: blending and superfast furnace, open storage, storage off blending area, waste treatment plant, and ball-mill 2nd floor operation. In addition, holding tanks and a trailer were removed. One area which could not be surveyed was the third level of the main building. Part of the stairs and landing were removed which posed a safe access problem. A survey in this area was not critical since the survey prior to decontamination (Part II of this report) showed that radiation and contamination levels were within acceptable limits. The buildings and grounds were monitored with a beta-gamma GM survey meter (Victoreen Model 491; Probe Model 491). All readings were taken at a distance of 1 centimeter from the cleansed surfaces. Background levels were in the range of 0.05 to 0.1 mr/hr inside the buildings and 0.1 to 0.4 mr/hr

outdoors on site.

Smear samples were taken to evaluate removable alpha contamination. These were counted on-site and at the office of Applied Health Physics, Inc., Bethel Park, Pa. with a windowless gas-flow proportional counter (NMC, Model PC-3A).

Analysis of these smears appears in Appendix B of Part IV.

The location of the smears and meter readings are located on each individual room diagram, which are collectively known as Figure 1. These results are then summarized in Table I.

Survey Results

On June 3-5, 1974, the beta-gamma radiation levels of the decontaminated surfaces were around the average level of 0.2 mr/hr set for building surfaces and equipment. The places reading 0.8 mr/hr or greater were noted in this report, and the information was relayed to the field supervisor for further decontamination work. The survey conducted on June 27, 1974 showed that the radiation levels in these areas, after cleaning, gave readings of 0.2 mr/hr or less. These corrected readings are in parentheses in Figure 1.

Of the smear samples, the highest found was 107 \pm 7.4 dpm/100 cm² in the workshop area. This June 4, 1974 result is approximately a factor of ten below the accepted limits for removable alpha radioactivity as found in Appendix A. All of the smears are far below this limit of 1,000 dpm/100 cm² set for removable alpha activity.

The results of the property survey taken on June 24, 1974 are contained in Figure II. It should be realized that a covering of soil is still needed on some parts of the site in order for this work to be considered finished. A survey is therefore needed when this work is done.

Conclusion

From the results of the decontamination survey, the property is ready for unlimited occupancy, with the exception of some work that is incomplete outdoors. After this work is done, a survey undertaken by Applied Health Physics, Inc., personnel should verify that this property meets all the requirements set forth by the State of New Jersey and the United States Atomic Energy Commission for release of decontaminated facilities.

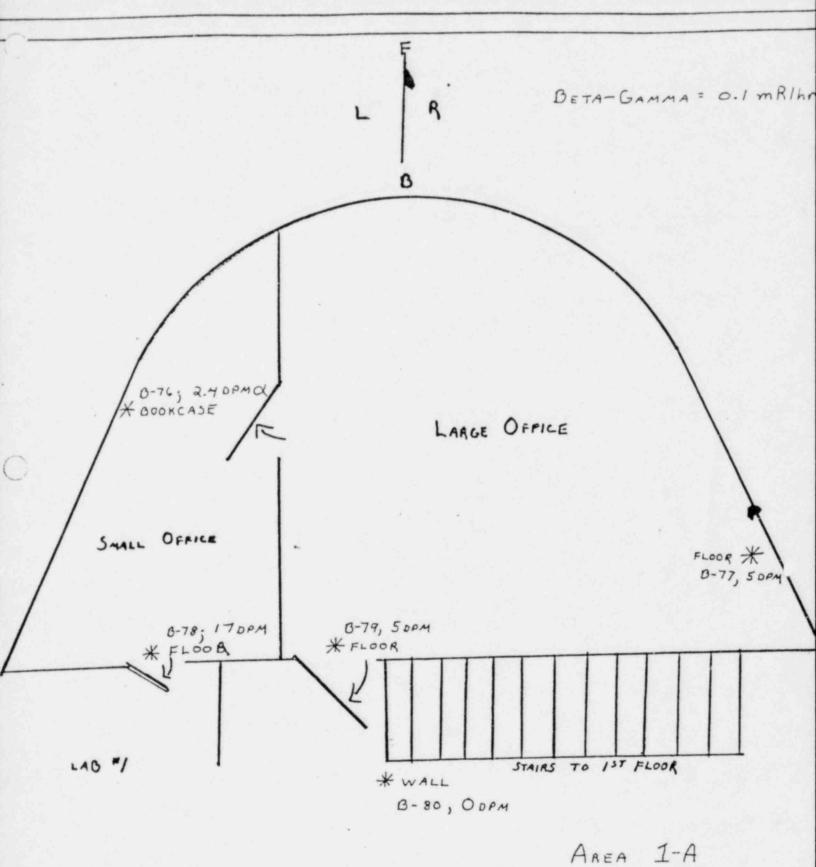
ADDENDUM

On July 18, 1974, the grounds were surveyed after all soil was put in place over the burial sites and smoothed. Figure III shows the results of this survey. The W. R. Grace Co. property now is within the radioactivity contamination limits set forth by the United States Atomic Energy Commission and the State of New Jersey.

Key to Following Diagrams:

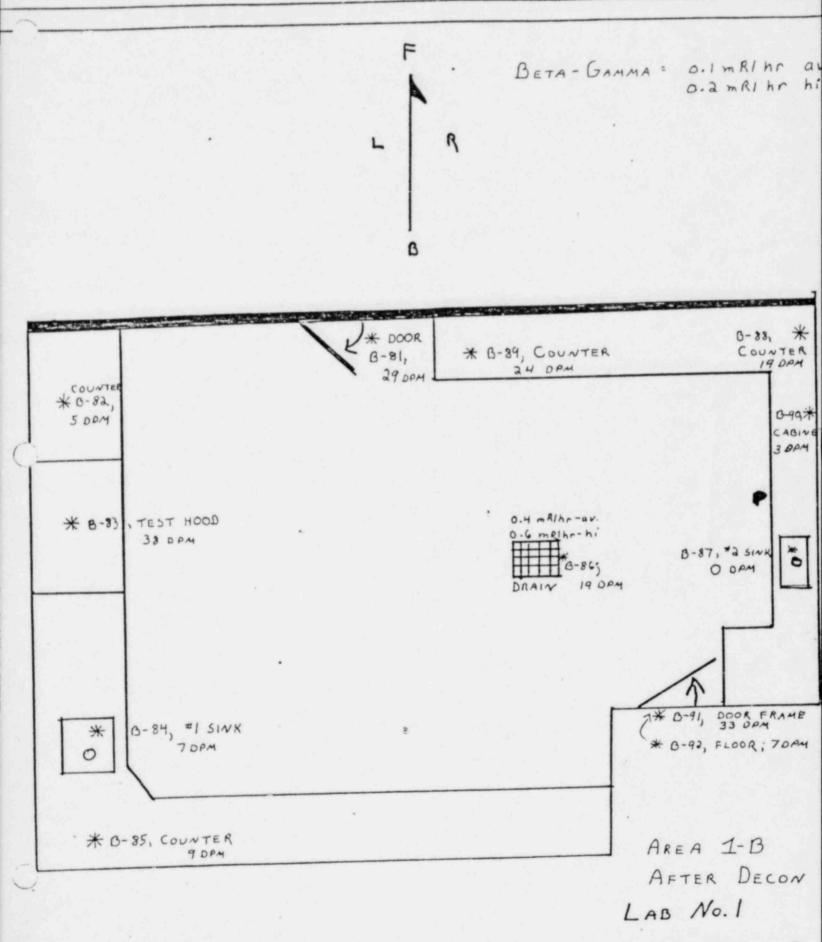
B-78 is the sample number
17 DPM is the removable alpha
DPM/100 CM².

Meter reading of the room is in the upper right corner, except for "hot" spots as indicated.

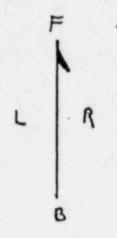


AREA 1-A
AFTER DECON.

OFFICES 2ND. FLOOR, MAIN B

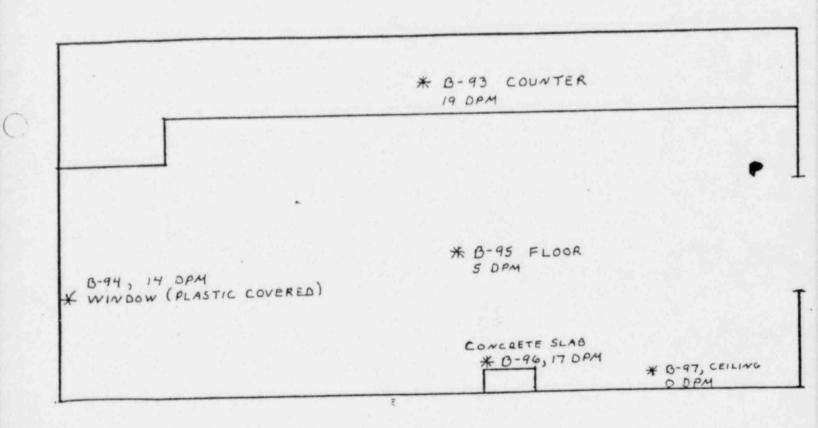


2ND. FLOOR, MAIN BLD



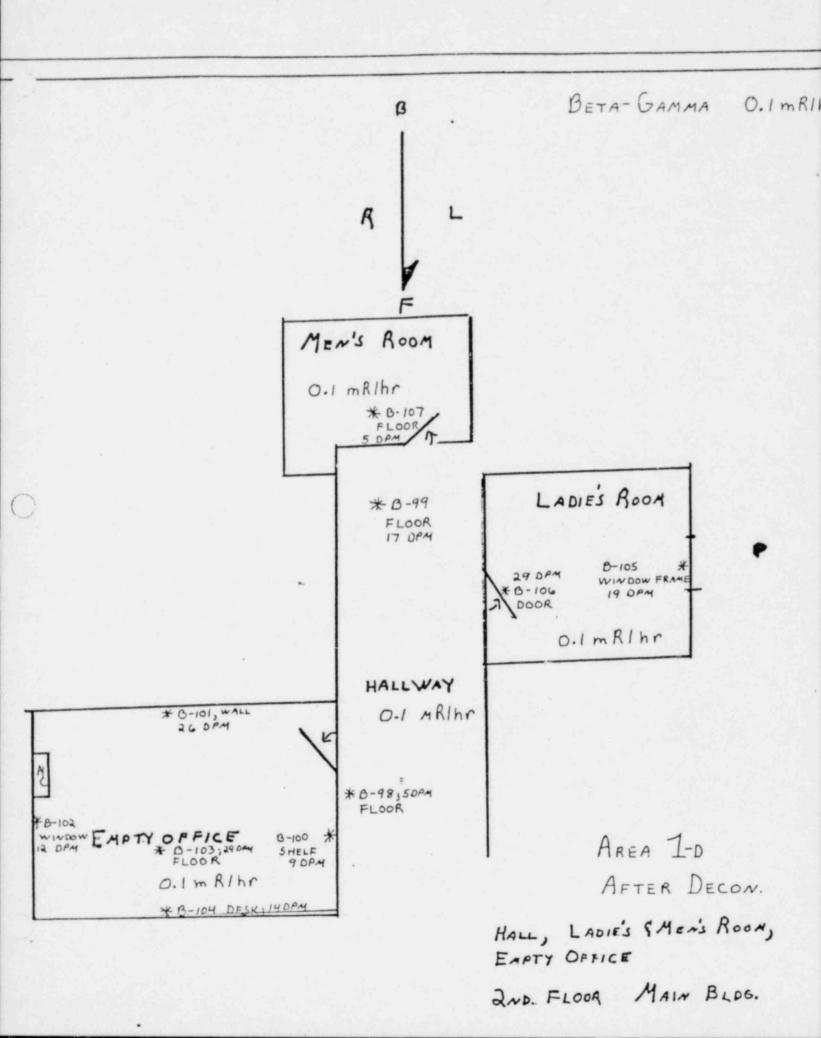
·4:3

BETA-GAMMA = O.I mRIhr

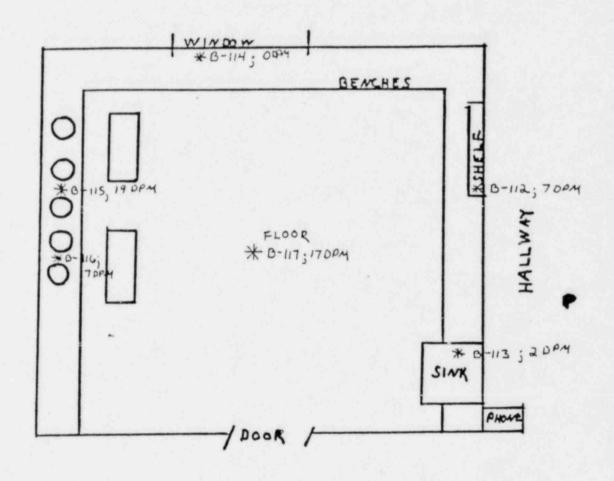


AREA 1-C AFTER DECON.

SAMPLE PREPARATION ROOM 2ND. FLOOR, MAIN BLOG.

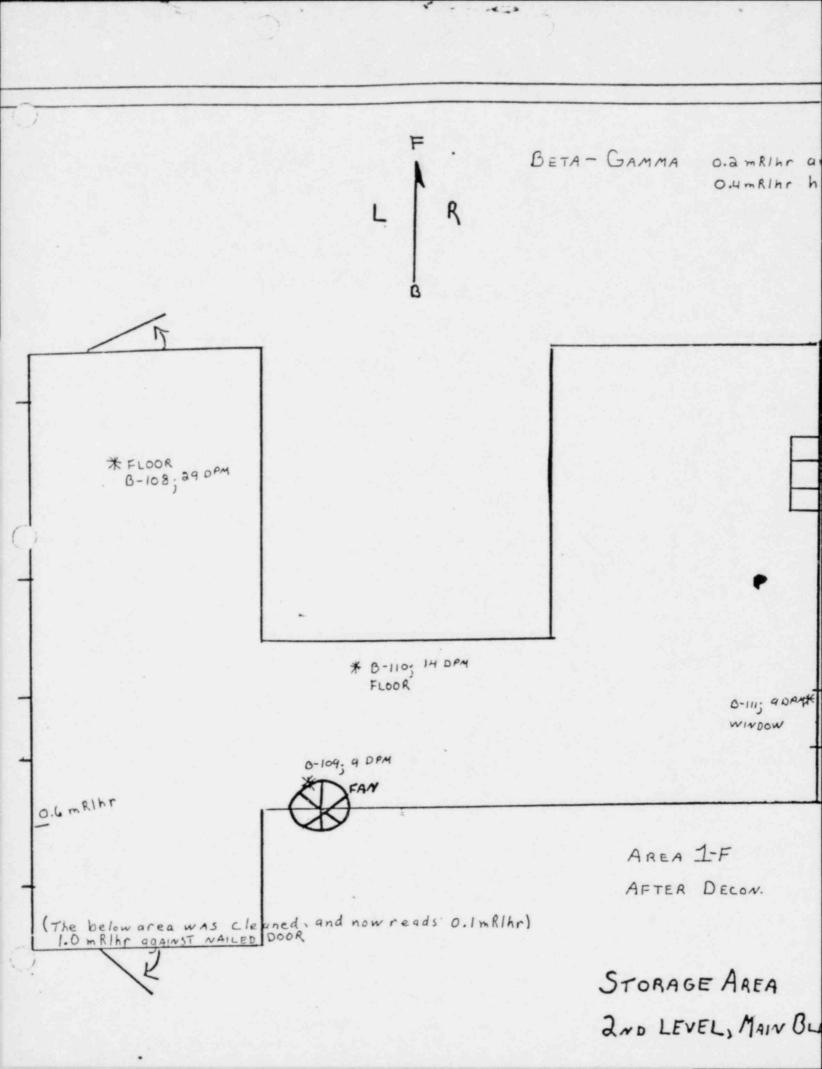


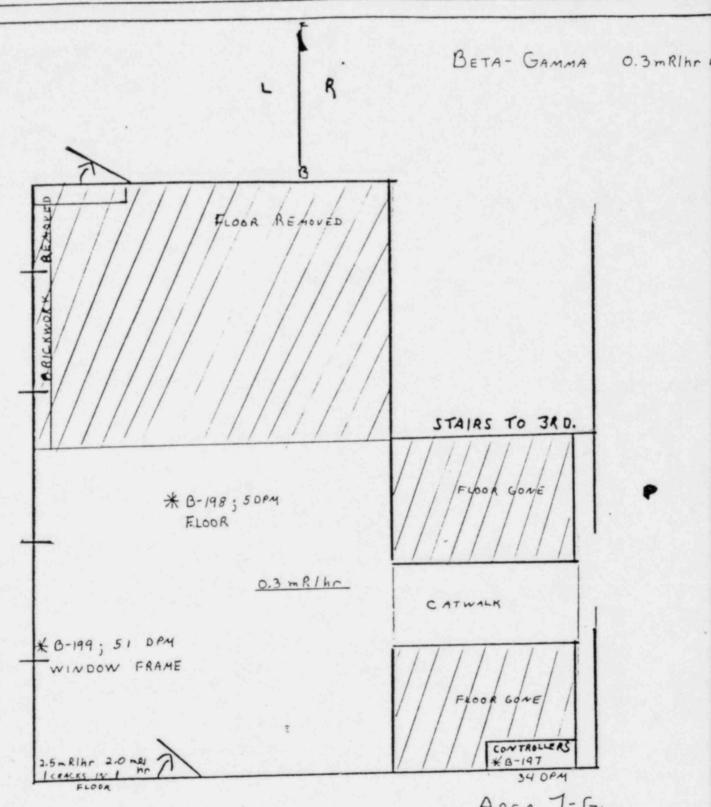
. ..



AREA 1-E AFTER DECON.

GRINDING & POLISHING TEST LAB 2ND FLOOR MAIN BLOG.



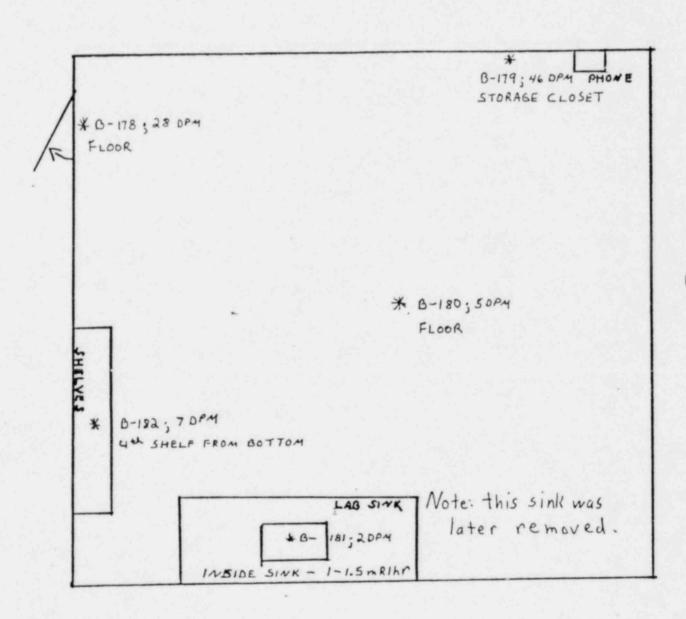


AREA I-G. AFTER DECON.

PRESS ROOM & SULFONATION ROO ZNA FLOOR, MAIN BLOG. L R

BETA - GAMMA

O.15 mR/hr O.2 mR/hr h



AREA 1-H AFTER DECON. TEST LAB 2

200 LEVEL, MAIN BL

BETA- GAMMA O. 15 mRihr a

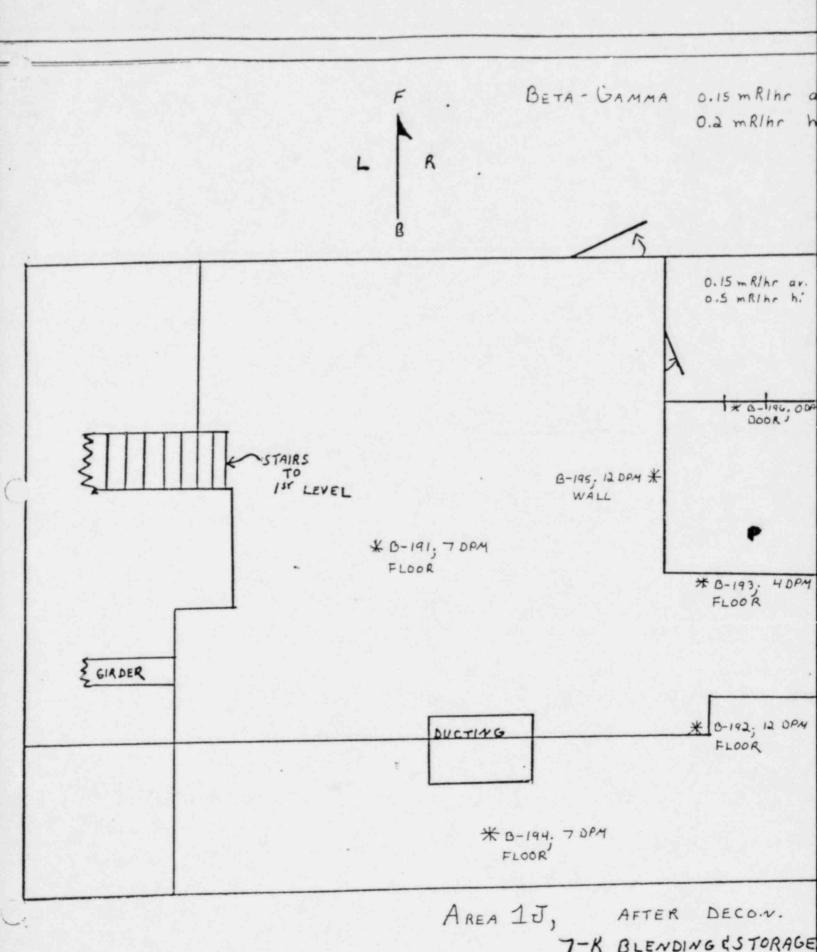
o.amRIhr h

SHELVES * B-183; 70PM 0-185; 14 DAY 4th SHELF FROM BOTTOM B-184, 2 DPM * * 0-186; 70PM FLOOR SHELVES

> CONFERENCE ROOM */ 2ND. FLOOR, MAIN BLOG.

AREA 1-I

AFTER DECON.



7-K BLENDING STORAGE

2ND LEVEL, MAN BLD

L R

BETA- GAMMA

0.15 mRinr 0.25 mRinr

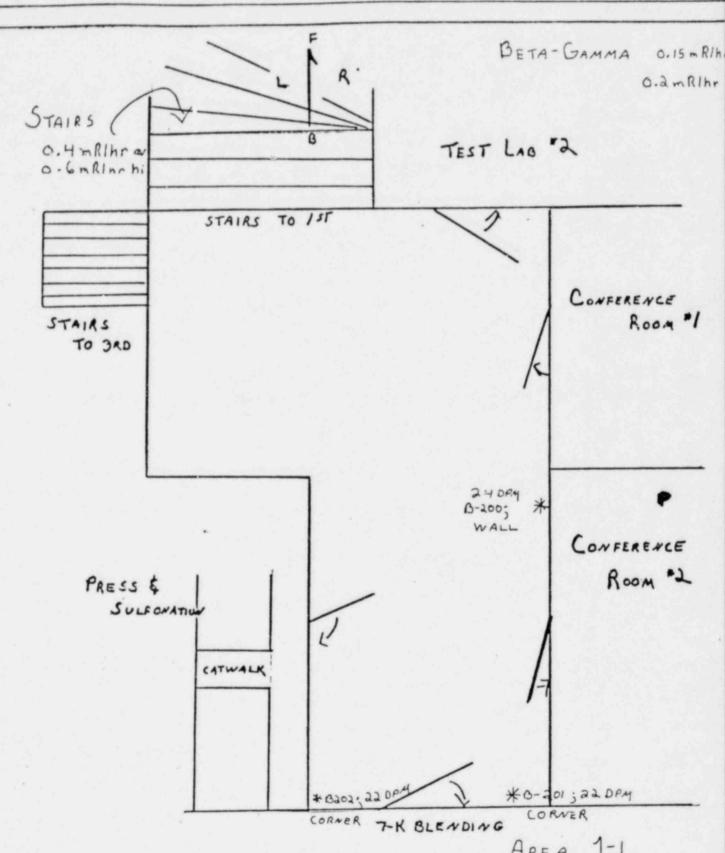
* B-189; 21 DPM
WALL

*B-190; 21 DPM
DOOR JAMO.

*B-188; 0 DPM
PHONE
PHONE

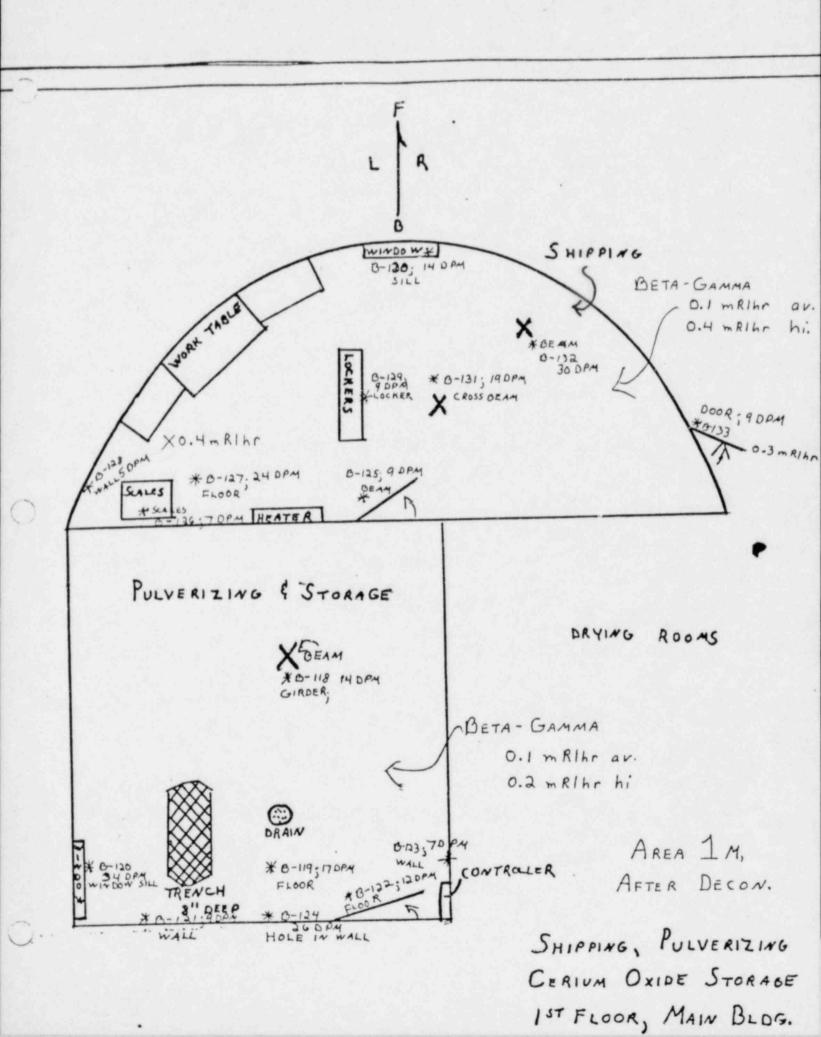
AREA 1K AFTER DECON.

200 FLOOR, MAIN BLOG.



AREA 1-L AFTER DECON.

SECOND FLOOR HALLWAY MAIN BLDG.

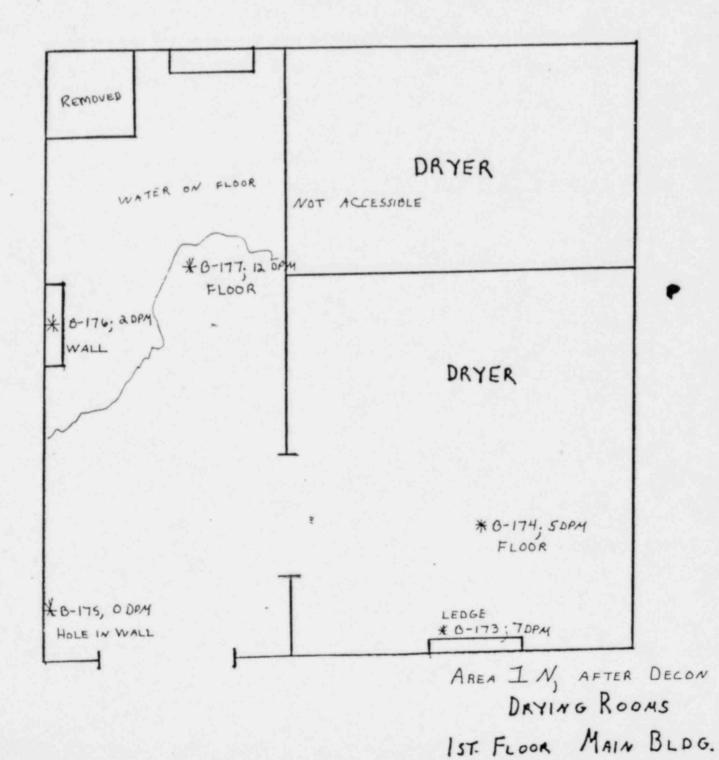


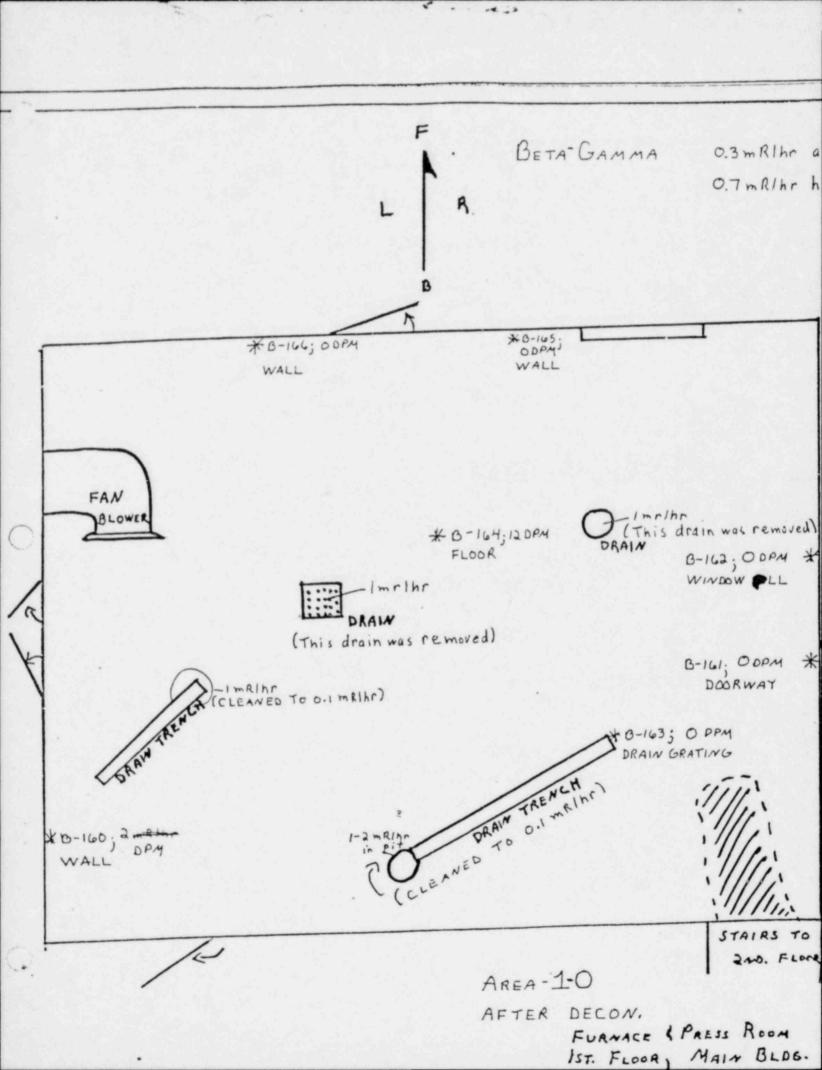
~ _ _

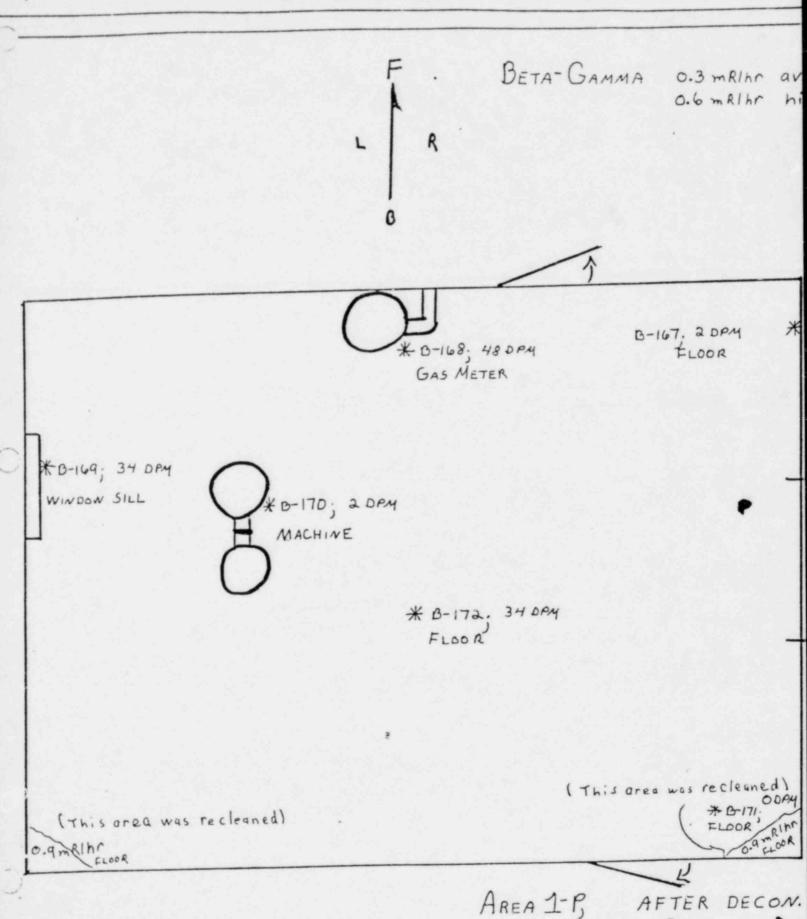
LR

BETA-GAMMA

0.15 mR/hr av



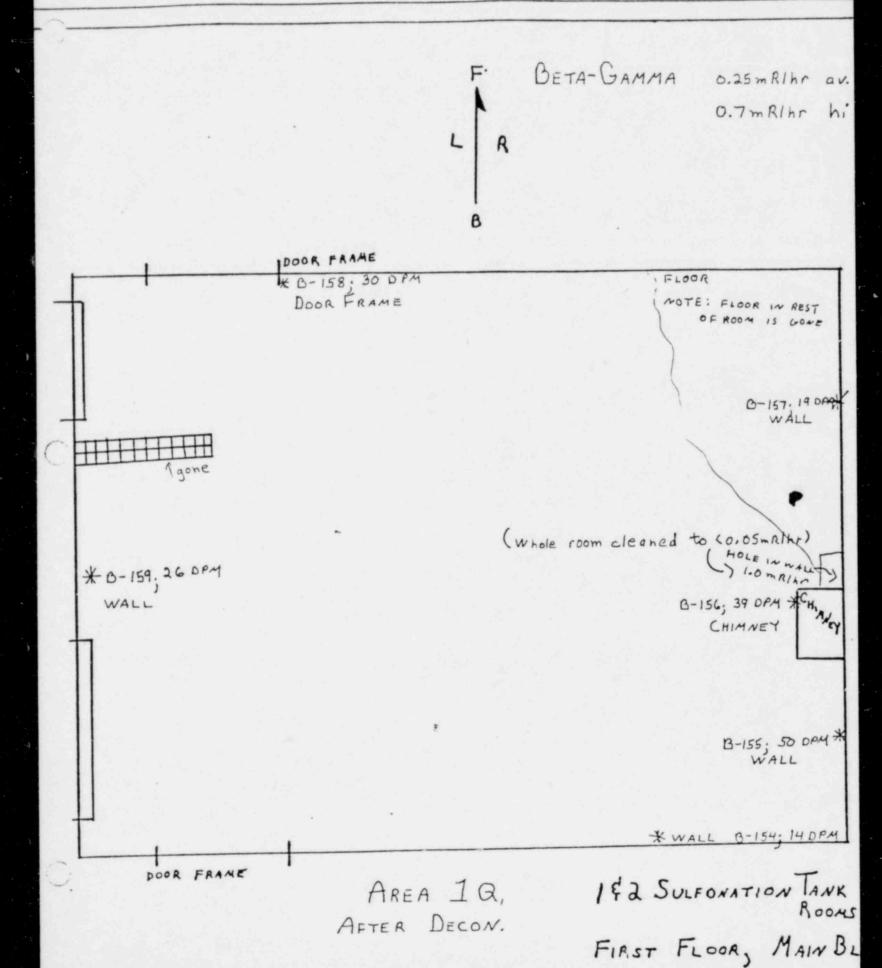


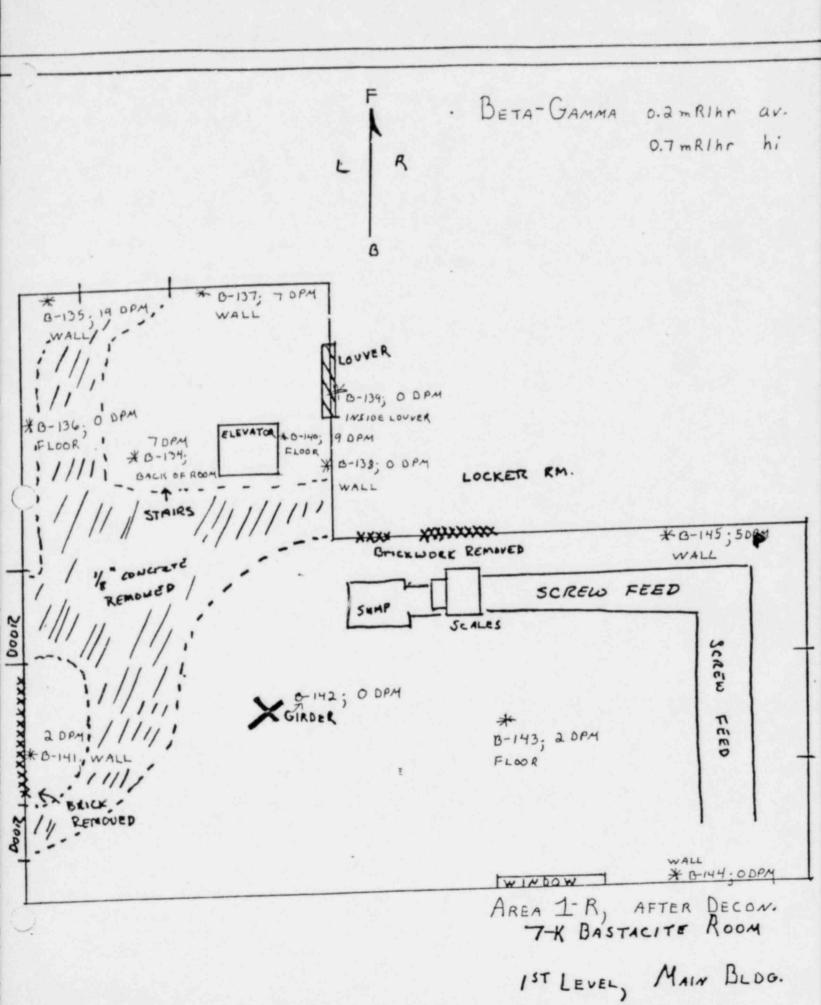


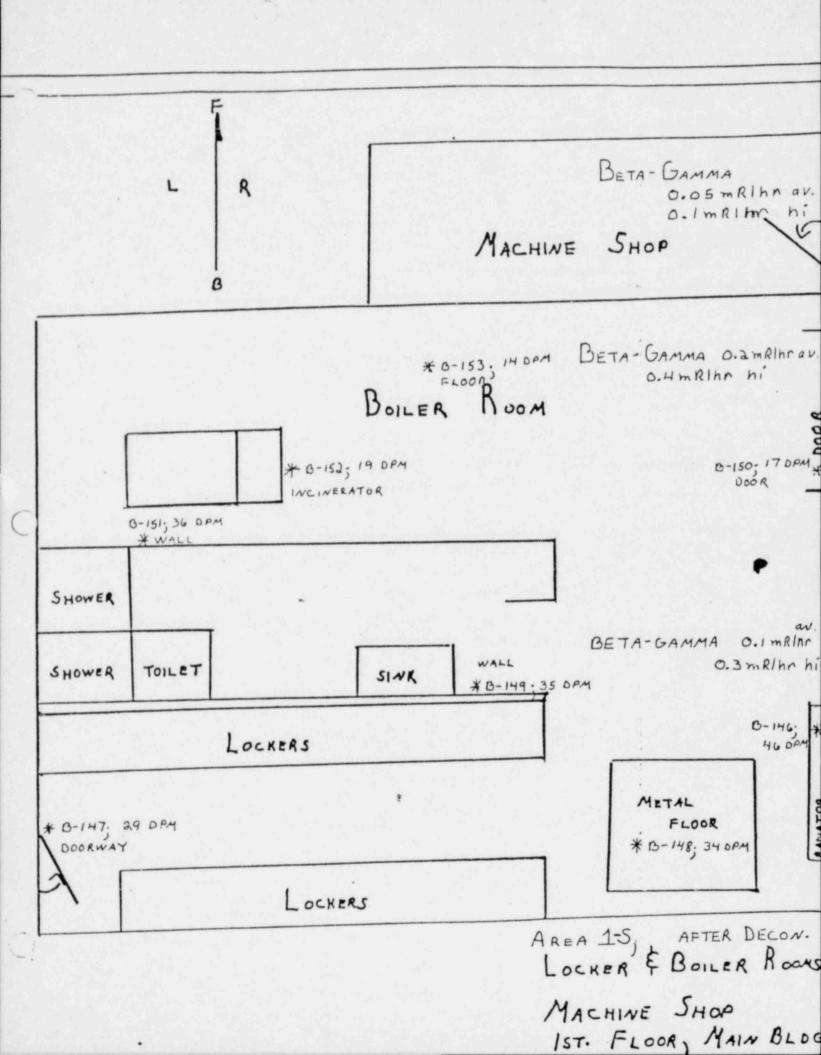
AREA 1-P, AFTER DECON.

SHARPLES COLLECTOR ROO

1ST PLOOR, MAIN BLOG







L R' BETA - GAMMA O.3 m Rihr av.

				# C-576,107
			,	BENC'H
			1////	1////
*C-579; 106 DPM			1////	/////
WALL HEATER	Similar	- 40 DPM	1/1/11	
	1.5.5.5.	*C-580;	WIRE CAGE	44
		11	1.11/11.	
	WORK	1:1//	nemound,	
	TABLE	1://	FLOOR /	
		1://	111.5	
		11/	///:	
		1://	,,'	
	17,7,7	//	/:	
	://///	///	· *C-577;	FLOOR
5'5" WALL				
* C-578 : 19 DPM				

AREA 2-A
AFTER DECON.

WORKSHOP BUILDING

0.3 mRIhr a BETA-GAMMA * 1-2 mRIhr in drai Note: whole area cleaned to less than O.Imalhr **TEMOUED** I STAIRS TO IST 1.5m RIhr CONCRETE REMOVED C-591; 15 DPM* C-589; 88 DPM DRAW TRENCH CONTERETE REMOVER C-590 ; 37 DPM ImRIhr Na.5 mRIAT WALL

AREA 2-B;

AFTER DECON.

BLOG = 2
BALL MILL

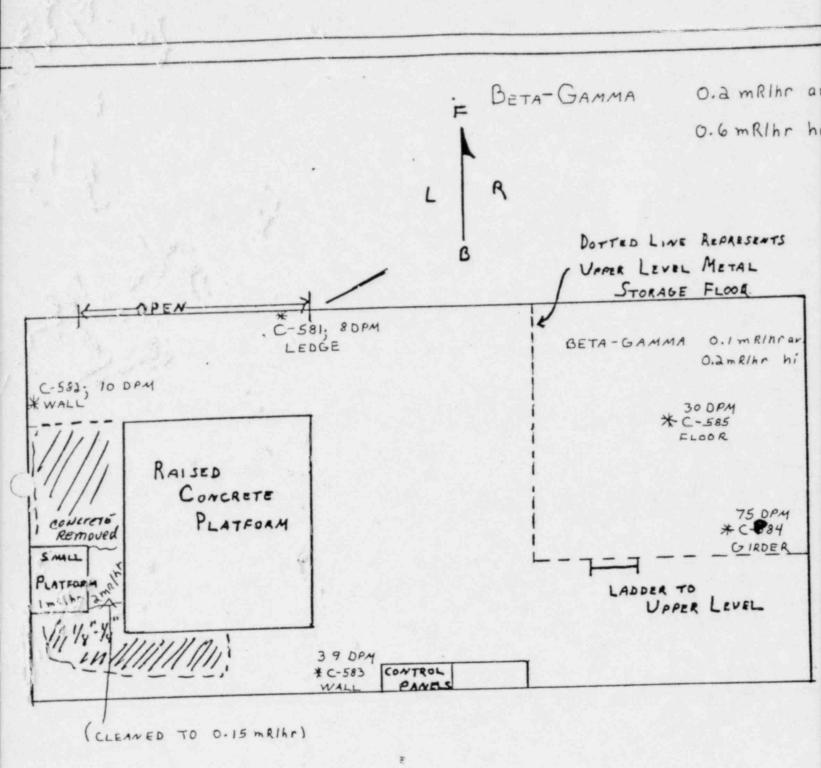
2ND LEVEL STORA

BETA- GAMMA O. SmRIhr av. Im RIhr intren (cleaned to 0.15 mRihr) #C-588; 48 DP4 | mR/hr TRENCH WALL STORAGE AREA * C-586 - 8: OPM FLOOR TRENCH Imalha 1 mR/hr TRENCH AREA 2-C

AFTER DECON.

LOWER OPERATIO

BALL MILL



AREA 2-D AFTER DECON.

COMPRESSOR ROOM

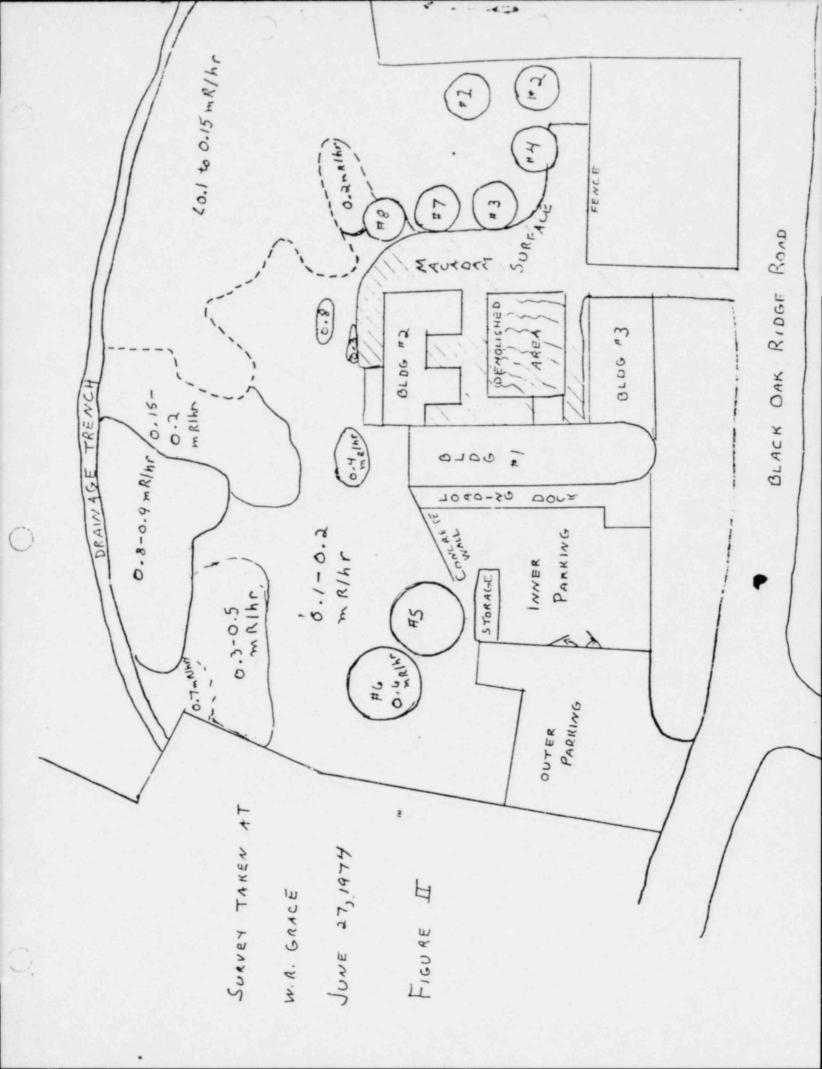
BETA- GAMMA BELOW R Note: whole area was cleaned to less than . 15 m Rlhr, av. ELECTRONUCLEONICS STORAGE, and level WALLS 0.5 mRIhr 1.5mRlhs GROUND 0.7 mRIhr . 5mRInr SCALES 1.0 m R/ hr .8 RIhr DRAIN 2.5 malhr GRATING

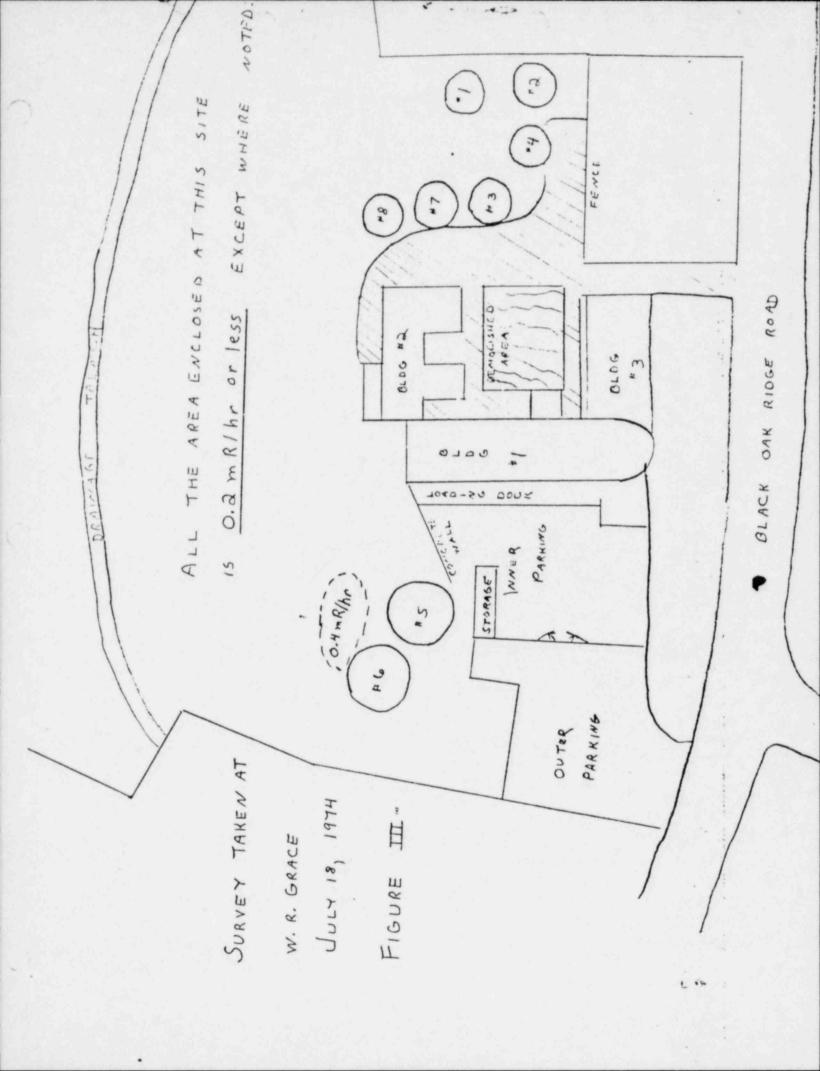
AREA 2-E

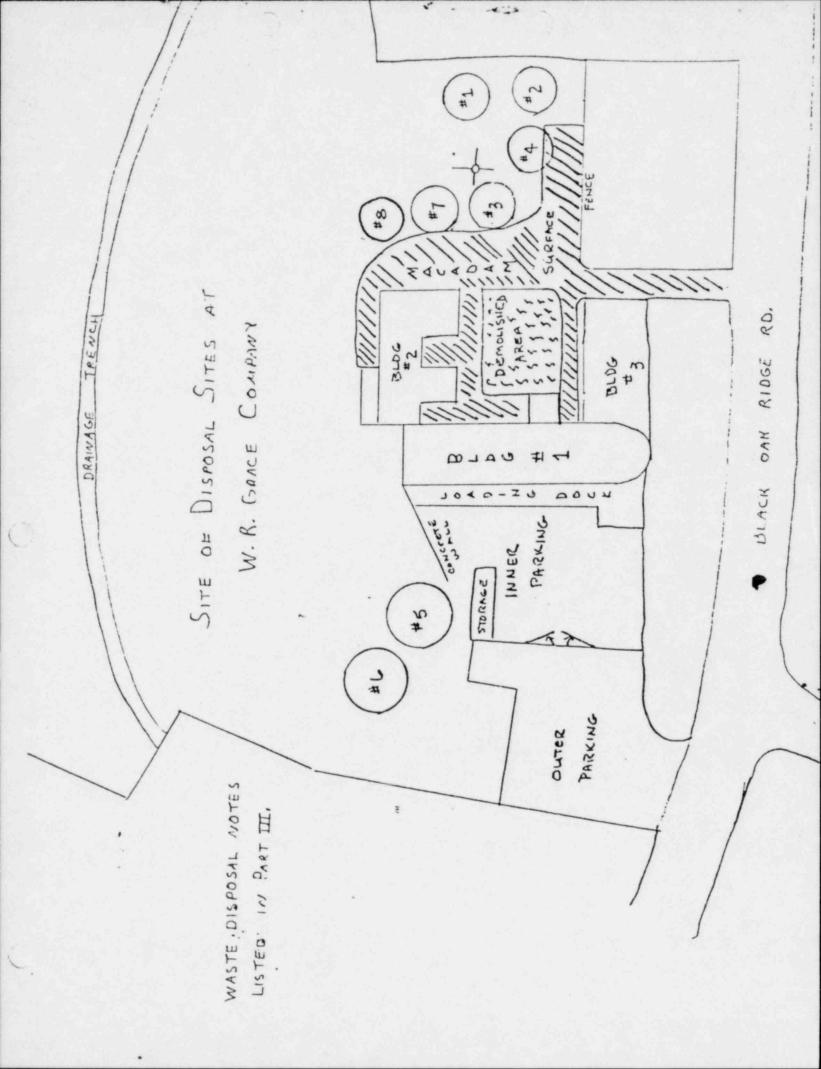
AFTER PRELIMINARY
DECON.

OUTSIDE,

3 BUILDINGS







Radioactivity Limits

for

Unrestricted Release

of

Equipment

RADIOACTIVITY LIMITS FOR UNRESTRICTED RELEASE

of

FACILITIES and EQUIPMENT

- The maximum amount of fixed alpha radioactivity in disintegrations per minute per 100 square centimeters on buildings or equipment should not exceed 25,000 dpm.
- The average amount of <u>fixed</u> alpha radioactivity in disintegrations per minute per 100 square centimeters on buildings or equipment should not exceed <u>5,000</u> dpm.
- 3. The maximum amount of <u>removable</u> (capable of being removed by wiping the surface with a filter paper or soft absorbent paper) alpha radioactivity in disintegrations per minute per 100 square centimeters on buildings or equipment should not exceed 1,000 dpm.
- 4. (a) The maximum level at one centimeter from the most highly contaminated surface of a building or piece of equipment measured with an open-window beta-gamma survey meter through a tissue equivalent absorber of not more than seven milligrams per square centimeter should not exceed 1.0 millirad per hour.
 - (b) The <u>average</u> radiation level at one centimeter from the contaminated surface of the building or equipment measured in the same manner should not exceed <u>0.2</u> millirad per hour.
- 5. The contamination limits for abandonment of facilities involving U-233 or plutonium should not exceed 1/10 of the limits in items 1, 2 and 3 above.
 - NOTES: A. A reasonable effort should be made to minimize the contamination present.
 - B. Surfaces of premises, equipment or scrap likely to be contaminated, and of such size, construction, or location as to make the surface inaccessible for purposes of measurement, shall be presumed to be contaminated in excess of the levels specified above.
 - C. Premises, equipment or scrap having contaminated surfaces which have been covered by painting, metal plating or other covering material should be presumed to be contaminated in excess of the levels specified above, unless it can be established that the contamination was below the above levels prior to applying the covering.

Analyses

of

Removable Contamination

Health Physics Laboratory Report: Analyses of Removable Contamination

0	Clie	ent: W. R. GRACE COMPAN	W,O	#			_ Date:			_ Pag	ge #_	1 of 8
	Des	cription of survey:								Na	tural	
	Sur	veyed by: M.McClosky/R.	. Slayto	n Dat	e: 6/3	3/74	Sus	pected	l Activi			
	Cou	nted with: GM detec	tor D	Scintil	lation	dete	ctor I] Gas 1	Proport	ional	dete	ctor
	Cou	nter Mfg: NMC		1	Model	PC-3/	4	_ s/n_	620(W.	R. Gra	ce)	
		kground of detector be					β		_			
	Effi	ciency of detector: a_	42%	ρ			Col	inted b	y: RJS			
SER.	SAMPLE	EIDENTIFICATION	Ctg.	Al c/m	pha c/m	_	1 d/m/	Cta	Beta-	Gamm c/m	a	d/mg/
	Time	Location		Gross		d/m	100 cm ²	Time	c/m Gross	Net	d/n	cm2
B-76	W.R.	.Grace - Main Office	2 Min.	18	1_	2.4	+ 1.6					
77	F	loor	"	21	2	5	- 1.8					-
78		11	"	29	7		± 3					-
79		II ,	"	19	2	5	± 1.8					
90	Wa	a11	"	15	0	0	± 0.9					
(11	Lar	ge Lab - Door	"	40	12	29	± 3.9					
82		" - Counter	"	20	2	5	# 1.8				P	-
83		" - Test Hood	"	48	16	38	± 4.5					
84		" - #1 Sink	"	23	3	7	± 2.1					
85		" Corner Counter	" "	24	4	9	± 2.3				4	
86		" Drain	· ·	33	8	19	± 3.2					
87		" #2 Sink	"	14	0	0	± 0.9					
88		" Counter	11	33	8	19	± 3.2					
89		11 11	"	36	10	24	± 3.6					
90		" Cabinet	"	17	1	3	± 1.5					
91		" Ledge	"	43	14	33	± 4.2					
92		" Floor	"	23	3	7	± 2.1					
	Samp	le Preparation Room -	"	32	8	19	+ 3.2					
93	Samp	le Preparation Room - ndow - Plastic covered	"	27	6	14	± 2.8					
94	Samp	le Preparation Room -	"	21	2	5	± 1.8	1				
95	Fl	oor		21			1,0	-				
			1									

AHP: HP102

Health Physics Laboratory Report. Analyses of Removable Contamination

1	Client: W.R. GRACE COMPANY	W, C), #			Date			_ Pag	ge #_	2 of
	Description of survey:										
	Surveyed by: M.McClosky/R.	Slavi	ton Dat	e: 6/3	1/74	Sus	pected	Activi	ty: T	atura horiu	1 m
	Counted with: GM detected										
	Counter Mfg: NMC	2		nodei	PC-3	A	_ 5/11_	620(W.1	K. GIA	ce)	
	Background of detector bef Background of detector af					m β β		_			
	Efficiency of detector: a						unted b	y: RJ	5 6-4-	74	111
		1		pha			1	Beta-	Gamm	na	
SER.		Ctg.	c/2m	c/m	1,,	d/m/	Ctg.	c/m	c/m	d/m	d/m
	Time Location Sample Preparation Room - Concrete Slab		Gross		a/m	100cm	Time	Gross	IVEC	4/111	CIII
96	Concrete Slab Sample Preparation Room -	2 Min		7	1	+0.9					
97	Ceiling		10	2	5	÷1.8		<u> </u>			
98	Small Hallway - Floor		20					-			
99	Small Hallway - Floor	"	29	7	17	± 3		-			
100	Empty Office - Shelf		24	4	9	12.3					
1	" - W211	"	38	11	26	±3.7					
102	" - Window	"	26	5	12	±2.6		-		P	
103	" - Floor	"	40	12	29	±3.9					
104	" - Desk	"	28	6	14	±2.3					
105	Ladies' Room-Window Frame	11	32	8	19	±3.2					
106	" - Door	"	39	12	29	±3.9					
107	Men's Room - Floor	11	20	2	5	+ 1.8					
108	Storage Area, 2nd Level -	11	40	12	29	t 3.9					
109	Storage Area, 2nd Level -	11	23	4	9	±2.3					
110	Storage Area, 2nd Level - Floor	11	28	6	14	+2.3					
111	Storage Area, 2nd Level - Window	п	24	4	9	±2.3					
112	Grinding-Polishing Test Lab - Shelf	"	21	3	7	±2.1					
113	Grinding-Polishing Test Lab - Sink	"	18	1	2	±1.3					
114	Grinding-Polishing Test Lab - Bench	ii	15	0	0	±0.9					
115	Grinding -Polishing Test Lab - Bet. 2nd & 3rd Vat	- 11	32	8	19	±3.2					
ن	Grinding-Polishing Test Lab - 4th & 5th Vat	"	21	3	7	+2.1					
117	Crinding-Polishing Test Lab - Floor	Ų	29	7	17	+3					
118	Shipping, Pulverizing Cerium Oxide Storage-Girde	"	27	6	14	±2.3					

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Health Physics Laboratory Report. Analyses of Removable Contamination

	Client: W.R. GRACE	w,o.	#			Date:			_ Pag	e #_:	3 of
	Description of survey:								Nat	tural	
	Surveyed by: M. McClosky/R.	Slayt	on Dat	e: 6-	3-74	Sus	pected	Activi	ty: The	orium	
	Counted with: GM detector	- 0	Scintill	ation	detec	tor D	Gas F	roport	ional	detec	tor
		or U.	,	(adal			s/n	620 (W.	P Cr	are)	
	Counter Mfg: NMC		v	nodei	PC-	3A		nzu (w.	A. GI		177
	Background of detector bef Background of detector aft	ore co	ounting unting:	a_	1.4			_			
	Efficiency of detector: a_	42%	_ρ_			Cou	inted b	y:F	US 6-	-4-74	
			Al	pha				Beta-	Gamm	la	ld/m
	APLE IDENTIFICATION	Ctg.	c/2m Gross	c/m Net	d/m	d/m/ 100cm ²	Ctg. Time	c/m Gross	Net Net	d/m	cm2
NO. Time	Location Shipping Pulverizing Floor Cerium Oxide Storage Floor	2 min	29	7		± 3					
119	Shipping Pulverizing Windo Cerium Oxide Storage - Sill	w "	47	15	34	± 4.2					
120	Shipping Pulverizing Cerium Oxide Storage-Wall	"	23	4	9	± 2.3					
121	Cerium Oxide Storage-Wall										
											-
0		His									-
										_	
										P	
		-									
					-						
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Health Physics Laboratory Report: Analyses of Removable Contamination

7-,		Client: W.R. GRACE	w.o.	#			Date:			_ Pag	e #	4 of
		Description of survey:								N:	atura	1
		Surveyed by: M. McClosky/F	R. Slay	tonDat	e: 6-3	-74	Sus	pected	Activi	ty: Th	noriu	<u>m</u>
		Counted with: GM detect					ctor [2	Gas. F	Proport	ional	detec	ctor
		Counter Mfg: NMC					C-3A	,	620 (V	V.R. G	race)	
		Background of detector bei	ore co	unting	: a	10	β_β_		_			
		Efficiency of detector: a_					_ Cou	inted b	y:RS,M	1 6-5	-74	
				A1	pha				Beta-		a	
SER.		PLE IDENTIFICATION	Ctg.	c/2m Gross	c/m Net	d/m	d/m/ 100cm ²	Ctg. Time	c/m Gross	c/m Net	d/m	d/m
NO.	7	Location Shipping, Pulverizing Cerium Oxide Storage-Floor		28	5	12	+2.6	Tall				
B-122		Shipping, Pulverizing Cerium Oxide Storage-Wall	"	24	3	7	±2.1					
B-123			"	39	11	26	+3.7					
124		Shipping, Pulverizing Cerium Oxide Storage- Hole Shipping, Pulverizing Wall Cerium Oxide Storage-Beam	in "	26	4	9	±2.3					
125		Cerium Oxide Storage-Beam	11	23	3	7	+2.1					
1)		Shipping, Pulverizing Cerium Oxide Storage-Scale Shipping, Pulverizing	es "		10	24	±3.6					
127	E	Shipping, Pulverizing Cerium Oxide Storage-Floo:	"	37		5	±1.9				P	
128	4.01	Shipping, Pulverizing Cerium Oxide Storage-Wall	"	22	2	9	±1.9					
129		Shipping, Pulverizing Cerium Oxide Storage-Lock	er "	30	6	14	±2.8					
130		Shipping Pulverizing Wind Cerium Oxide Storage Sill	-	33	8	19	±3.2					
131		Shipping, Pulverizing Cerium Oxide Storage-Cross Shipping, Pulverizing	S			30	+3.9					
132		CELIUM OXIGE COLUMN	-	44	13	-	+-					
133		Shipping, Pulverizing Cerium Oxide Storage-Door	1	26	4	9	+2.3			-		
134		First Level 7-K Bastasite Room-Back of Room	"	24	3	7	±2.1			-		-
135		First Level 7-K Bastasite Room-Wall	"	34	8	19	±3.2			-		-
136		First Level 7-K Bastasite Room-Floor	"	15	0	0	±0.97			-		
137		First Level 7-K Bastasite	"	24	3	.7	±2.1		-	-	-	
138		First Level 7-K Bastasite	"	13	0	0	±0.97	4		-	-	-
139		First Level 7-K Bastasite Room-Inside Louver	"	16	0	0	+0.97					
140		First Level 7-K Bastasite		25	4	9	+2.3				-	-
		First Level 7-K Bastasite	- 11	19	1	2	±1.4			-	-	-
141		Room-Wall First Level 7-K Bastasite	"	14	0	0	±0.9	1				
142		Room-Girder Pirst Level 7-K Bastasite	"	20	1	2	±1.4					
143		Room-Floor	•									

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Health Physics Laboratory Report: Analyses of Removable Contamination

	Client: W.R. GRACE	W.O.	#			Date:			_ Pag	e #_	5 of 8
	Description of survey:										
	Surveyed by: M. McClosky/R.	Slayt	on Dat	e:6-:	3-74	Sus	pected	l Activi	ty: Th	atura	n
	Counted with: GM detect	or DS									
	Counter Mfg: NMC		ì	Model	PC-	3A	_ s/n_	620 (W	. K. G	race)	
	Background of detector bei	ter cou	inting:	۵	9	β	untad h	_			
	Efficiency of detector: a_	42%				- 000	inted t				4
	AMPLE IDENTIFICATION	Ctg.	c/2m	c/m	1 / 2 / 2	d/m/	Ctg.	c/m Gross	c/m		d/m/
10. Ti	me Location First Level 7-K Bastasite Room-Wall		Gross 18	0		±0.97		1	1100		
-		1227									
-											
			1: 1:								
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Health Physics Laboratory Report: Analyses of Removable Contamination

	Client: W.R. GRACE	w, o.	#			Date-			_ Pag	e #_£	5 ot 8
	Description of survey:										
	Surveyed by: M.McClosky/R.	Slayto	n Dat	e: <u>6-3-</u>	-74	Sus	pected	Activi	ty: N	atura	il im
	Counted with: ☐GM detect										
	Counter Mfg: NMC		1	Model	PC-3	JA	_ s/n_	620 (1	W.R. G	race)	
	Background of detector be Background of detector af	ter cou	inting:	a		β		_			
	Efficiency of detector: a_	42%	P			Col	inted b	y: <u>MM</u>			
ED S	AMPLE IDENTIFICATION	-		pha		1 1/m/	Ctg.	Beta-		ia	d/m/
	me Location	Ctg.	c/2m Gross	c/m Net	d/m	100 cm ²	Time	c/m Gross	c/m Net	d/m	cm2
-145 10	0:00 First Level 7-K Bastasite	2 mir		2	5	±1.8					
.46	Locker & Boiler Room- Radiator		55	21	46	+4.9					
47	Locker & Boiler Room- Doorway		37	12	29	+3.9					
48	Locker & Boiler Room- Metal Floor		43	15	34	+4.2					
49	Locker & Boiler Room-		46	16	35	+4.3					
7	Locker & Boiler Room-		27	7	17	+3.0					
.51	Locker & Boiler Room- Wall	-	48	17	36	±4.3	4				
.52	Locker & Boiler Room- Incinerator		29	8	19	±3.2					
.53	Locker & Boiler Room- Floor		26	6	14	±2.8					
.54	1 & 2 Sulfonation Tank Room-Wall		25	6	14	±2.8					
.55	1 & 2 Sulfonation Tank Room-Wall		60	23	50	+5.1					
.56	1 & 2 Sulfonation Tank Room-Chimney		49	18	39	±4.5					
.57	1 & 2 Sulfonation Tank Room-Wall		29	8	19	+3.2					
.58	1 & 2 Sulfonation Tank Room-Deor Frame		39	13	30	±3,9					
.59	1 Room-Walfonation Tank		36	11	26	+3.7					
.60	Furnace & Press Room- Wall		15	1	2	±1.3					
.61	Furnace & Press Room- Doorway		7	0	0	±0.8					
.62	Furnace & Press Room- Window Sill		13	0	0	+0.8	int:				
.63	Furnace & Press Room- Drain Grating		13	0	0	±0.8					
.64	Furnace & Press Room- Floor		24	5	12	+2.6					
é	Furnace & Press Room- Wall		10	0	0	+0.8					
66	Furnace & Press Room-		12	0	0	±0.8	0111				
67	Sharples Collector Room- Floor		15	1	2	±1.3					

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Health Physics Laboratory Report: Analyses of Removable Contamination

	Client: W.R. GRACE	w,o	#			Date			_ Pag	ge #_	7 of 8
	Description of survey:										
	Surveyed by: M.McClosky/	R. Slay	tonDat	e: <u>6-3</u>	-74	Sus	spected	Activi	ty: Na	tural	
	Counted with: GM detect	or D	Scintill	lation	dete	ctor [Gas I	Proport	ional	orium dete	ctor
	Counter Mfg: NMC		1	Model	PC-	-3A	_ s/n_	620 (W.	R. Gr	ace)	
	Background of detector be Background of detector af Efficiency of detector: a_	ter co		a		β	unted b		6-5-	74	
	SAMPLE IDENTIFICATION			pha	,			Beta-			12//
SER. NO.		Ctg.	C/2m Gross	c/m Net	d/m	d/m/	Ctg.	c/m Gross	c/m Net	d/n	d/m/
0-168	9:00 Sharples Collector Room-	2 min		20	48	+4.9					
169	Sharples Collector Room-		26	14	34	±4.2					
170	Sharples Collector Room- Machine		13	1	2	±1.2					
171	Sharples Collector Room-		12	0	0	+0.7					
172	Sharples Collector Room-		26	14	34	+4.2					44.1
1(Drying Rooms-Ledge		17	3	7	±2.0					THE REAL PROPERTY.
174	" " Floor		15	2	5	±1.7					
175	" " Hole in wall		10	0	0	±0.7				•	
176	" " Wall ~		14	1	2	±1.2					
177	" " Floor		21	5	12	±2.6					
178	Test Lab #2 Floor		37	12	28	±3.8					
179	" " Storage Close	E	31	19	46	±4.8					
180	" " " Floor		15	2	5	±1.7					
181	" " " Lab Sink		14	1	2	±1.2					
182	" " " Shelf		19	3	7	+2.0					
183	Conference Room #1-		19:	. 3	7	±2.0					
184	Conference Room #1- Top of A/C		14	1	2	±1.2					
185	Conference Room #1- Floor		23	6	14	±2.7					
186	Conference Room #1-		17	3	7	±2.0					
187	Conference Room #2- Window Sill		11	0	0	±0.7		FUEL			
1	Conference Room #2-		12	0	0	±0.7					
189	Conference Room #2-		30	9	21	±3.3					
190	Conference Room #2-		30	9	21	±3.3					

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Health Physics Laboratory Report: Analyses of Removable Contamination

~		Client:_	W.R. GRACE		w,o,	#			Date			_ Pag	ge # <u>8</u>	of 8
		Descript	ion of survey:									Na	tural	
		Surveyed	by: M.McClosk	y/R.	Slayt	on Dat	e:		Sus	pected	Activi	ty: The	orium	
		Counted	with: □GM de	tect	or D									ctor
		Counter	Mfg: NMC			,	Model	PC	-3A	_ s/n_	620(W.F	R. Gra	ce)	
		Backgrou	and of detector	bef af	ore co	unting unting:	: a_	8.8c			_			
		Efficienc	y of detector:	a_	42%	μ			Co	unted b	y: RJS	6-4-	74	
ER.	SAM	DIFIDE	NTIFICATION		C.		pha	1	d/m/	Ctg.	Beta-	Gamn c/m		d/m/
- m	Time	Loc	ation		Ctg. Time	c/2m Gross	c/m Net	d/m	100mc	Time	Gross	Net	d/m	cm2
-191		Storage	1 7-K Blending		2 min	25	3	7	±2.0					
92		2nd Lev Storag	rel 7-K Blendin ge- Floor	g &	4.0	28	5	12	±1.3					
93		2nd Leve	1 7-K Blending	&		31	6	4	±1.7					
94		2nd Leve	1 7-K Blending	6		22	3	7	+2.1					
95		2nd Leve Storage	I Z-K Blending	å		27	5	12	±2.6					
		2nd Leve	1 7-K Blending	&		16	0	0	±0.9					
97		Press &	Sulfonation Roller	om-		55	14	34	±4.2					
98		Press &	Sulfonation Ro	om-		21	2	5	±1.8					
99			Sulfonation Ro			60	21	51	+5.1					
00			g. Rear 2nd Fl	oor		29	10	24	±3.5		th.			
01		Main, Blo	Rear 2nd Fl	oor		35	9	22	±3.4					
02		Main Blo	Rear 2nd Fl	oor		34	9	22	+3.4		T			
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Health Physics Laboratory Report: Analyses of Removable Contamination

0		Client:_	W.R	. GR	ACE	_w.o.	#			Date.	6-4-	74	_ Pag	ge #	1 of 1
		Descrip	tion	of su	rvey:										
		Surveye	d by:	M.M	c #€ losky/R	. Slay	tonDat	e: _ 6/	3/74	Sus	pected	Activi	ty: Th	tural orium	
		Counted	with:	: 00	GM detecto	or OS	Scintill	lation	detec	tor [Gas I	roport	ional	detec	ctor
		Counter	Mfg:		NMC		1	Model	PC	-3A	s/n_	620 (W	R. G	race)	
		Backgro	und c	of de	tector beforector aft	er cou	unting:	α	8.8	β	inted b	y: RJS	6-	4-74	
No.	T			516	ATION		Al	pha				Beta-			
	SAM	PLE IDI	ENII.	FIGI	ATION	Ctg.	c/2m Gross	c/m Not	d/m	d/m/	Ctg.	c/m Gross	c/m Net	d/m	d/m/
576	Time	Loc Outside	Work	Sho	p-Bench	2 min		45	107	±7.4					
577	P.M.	"	"	"	Floor	T.H	77	30	77	±6.3	1				
578		"	"	***	5' 5" Wall		33	8	19	±3.2					
579			-11	11	Wall Heate	r	106	45	106	±7.3					BEE ST
580		11	11	"	Floor		51	17	40	±4.6	,				
4		Compres	sor R	loom-	Ledge		24	4	8	±2.2	Time.				The second
582		**		"	Wall		25	4	10	±2.4					
583		"		"	Wall		50	17	39	±4.5				•	
584		11		"	Girder ~		80	32	75	±6.2					
585		"		"	Storage		42	13	30	±3.9					
586		Ball Mi Floor	11 Lo	wer	Operation-		85	34	81	±6.4					
587		Ball Mi	ll Lo	wer	Operation-	1	104	44	104	±7.3					
588		Ball Mi Wall	ll Lo	wer	Operation-		57	20	48	+4.9					
589		Ball Mi Power	Il 2n Supp	dLe	vel Storag	e-	91	37	88	±6.7					
590		Ball, Mi	11 2n	d Le	vel Storag	e-	48	16	37	±4.4					
591		Ball	11 2n	d Le	vel Storag	e-	30	7	15	±2.9					
							LTL								
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(Territoria								
-															

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TABLE I

RESULTS OF DECONTAMINATION RADIATION FINAL SURVEY

Location	Reference	Beta-G Meter St mR/h	urvey	Alpha Rem Contamin DPM/100	ation
		average	high	average	high
W. R. Grace Office, 2nd level, Main Bldg.	1-A	0.1	0.1	6±1.8	17±3
Lab #1, 2nd level, Main Bldg.	1-B	0.1	0.2	16±2.7	38±4.5
Floor Drain, 2nd level, Main Bldg.	1-B	0.4	0.6		19 [±] 3.2
Sample Preparation Room, 2nd level, Main Bldg.	1-C	0.1	0.1	11 [±] 2.3	19 [±] 3.2
Small Hallway, 2nd level, Main Bldg.	1-D	0.1	0.1	7 [±] 2.4	17±3
Men's Room, 2nd level, Main Bldg.	1-D	0.1	0.1		5±1.8
Ladies' Room, 2nd level, Main Bldg.	1-D	0.1	0.1	24 [±] 3.6	29±3.9
Empty Office, 2nd level, Main Bldg.	1-D	0.1	0.1	18±3	29±3.9
Grinding & Polishing Test Lab, 2nd level, Main Bldg.	1-E	0.05	0.4	9 [±] 2.1	19 [±] 3.2
Storage level, 2nd level, Main Bldg. against nailed door	1-F	0.2	0.4	15 [±] 2.7	29±3.9
Press Room & Sulfonation Room, 2nd level, Main Bldg.	1-G *	0.3	2.0 back corn	30 [±] 3.7	51 [±] 5.1
Test Lab #2, 2nd level, Main Bldg.	1-Н	0.15	0.2	18 [±] 2.7	46 ⁺ 4.8

TABLE I

RESULTS OF DECONTAMINATION RADIATION FINAL SURVEY

Location	Reference	Beta-Ga Meter St mR/hr	urvey	Alpha Rem Contamin DPM/100	ation
		average	high	average	high
Conference Room #1, 2nd level, Main Bldg.	1-1	0.15	0.2	8 [±] 2.0	14-2.7
7-K Blending & Storage, 2nd level, Main Bldg.	1 - J	0.15	0.2	7 [±] 1.8	12 [±] 2.6
Conference Room #2, 2nd level, Main Bldg.	1-K	0.15	0.25	11 [±] 2.0	21 [±] 3.3
Hallway, 2nd level, Main Bldg. stairs to 1st level	1-L	0.15 0.4	0.2	23 [±] 3.4	24+3.5
Shipping, Pulverizing, Cerium Oxide Storage, 1st level, Main Bldg.	1-M	0.1	0.4	15 [±] 2.8	34-4.2
Drying Rooms, 1st level, Main Bldg.	1-N	0.15	0.3	5.2 [±] 1.6	12-2.6
Furnace & Press Room, 1st level, Main Bldg. four drains	1-0	0.3	0.7	2 [±] 1.3	12-2.6
Sharples Collector Room, 1st level, Main Bldg. two corners	1-P	0.3	0.6	20±2.8	48-4.9
1 & 2 Sulfonation Tank Rooms, 1st level, Main Bldg. brick wall	1-Q	0.25	0.7	30±3.9	50±5.8
7-K Bastacite Room, 1st level, Main Bldg.	1-R	0.2	0.7	4±1.6	19 [±] 3.2

Continued on next page

TABLE I

RESULTS OF DECONTAMINATION RADIATION FINAL SURVEY

Location	Reference	Beta-Gamma Meter Survey mR/hr		Alpha Removable Contamination DPM/100 cm ²	
		average	high	average	high
Locker Room, 1st level, Main Bldg.	1-S	0.1	0.3	36±4.3	46 [±] 4.9
Boiler Room, 1st level, Main Bldg.	1-S	0.2	0.4	22 [±] 3.3	36 [±] 4.3
Machine Shop, 1st level, Main Bldg.	1-S	0.05	0.1		
Workshop Bldg.	2-A	0.3	0.7	70 [±] 5.8	107 [±] 7.4
Ball Mill, 2nd level, Storage	2-В	0.3	(1.0-2.0) (drains)	47±4.7	88 [±] 6.7
Ball Mill, Lower Operation	2-C		1.0 trench & concrete)	77 [±] 6.2	104 [±] 7.3
Compressor Bldg., 1st and 2nd levels small concrete slab	2-D	0.2	0.6	32 [±] 3.8	75 [±] 6.2
Outside of Electronucleonic Storage	2-E	1.0			

*

