



CONNECTICUT YANKEE ATOMIC POWER COMPANY

HADDAM NECK PLANT

362 INJUN HOLLOW ROAD • EAST HAMPTON, CT 06424-3099

January 15, 1994
Re: Technical Specification 6.9.1.8
Docket No. 50-213

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

In accordance with reporting requirements of Technical Specification 6.9.1.8, the Connecticut Yankee Haddam Neck Plant Monthly Operating Report 93-13 covering operations for the period December 1, 1993 to December 31, 1993 is hereby forwarded.

Very truly yours,

John P. Stetz
Vice President
Haddam Neck Station

JPS/va

- cc: (1) Regional Administrator, Region 1
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406
- (2) William J. Raymond
Sr. Resident Inspector
Connecticut Yankee

2100

IE24

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Connecticut Yankee Atomic Power Company

Haddam Neck Plant

Haddam, Connecticut

Monthly Operating Report No. 93-13

For The Month of

December 1993

Plant Operations Summary - December 1993

The following is a Summary of Plant Operations for December 1993.

The plant continued operating at 52% power due to repair of the 1B main feed pump impeller casing.

On December 9 at 0403 hours, a further power reduction was initiated due to a feedwater MOV design concern. Load was stabilized at three percent power at 0625 hours and the concern was dispositioned. At 1337 hours, the power was increased to 10% for a chemistry hold. The plant reached 10% power at 1356 hours. At 1719 hours, the load increase recommenced. At 1907 hours, the plant reached 30% power for another chemistry hold. At 2102 hours, power ascension resumed.

The load increase was stopped at 2257 hours at 50% power due to overheating of the "B" main feed pump seal. The problem was resolved by Maintenance and a power increase started at 0028 hours on December 10. The plant reached 100% power on December 11 at 2208 hours.

On December 15 at 0105 hours, CONVEX requested that the plant decrease load to 86% due to an overload problem on the grid. The plant was at 86 % power at 0205 hours and started a return to full power at 0455 hours. The plant reached 100% power at 0615 hours and remained there through the end of December.

NRC OPERATING STATUS REPORT

Haddam Neck

1. Docket: 50-213
2. Reporting Period: 12/93 Outage + On-line Hours: 0.0 + 744.0 = 744.0
3. Utility Contact: R. E. Borg (203) 267-3649
4. Licensed Thermal Power (MWT): 1825
5. Nameplate Rating (Gross MWe): 667 x 0.9 = 600.3
6. Design Electrical Rating (Net MWe): 582
7. Maximum Dependable Capacity (Gross MWe): 586.9
8. Maximum Dependable Capacity (Net MWe): 560.1
9. If changes occur above since last report, reasons are: NONE
10. Power level to which restricted, if any (Net MWe): N/A
11. Reasons for restriction, if any: N/A

	MONTH	YEAR-TO-DATE	CUMULATIVE
12. Report period hours:	744.0	8,760.0	227,928.0
13. Hours reactor critical:	744.0	7,145.9	181,952.3
14. Reactor reserve shutdown hours:	0.0	0.0	1,285.0
15. Hours generator on-line:	744.0	6,915.7	175,202.9
16. Unit reserve shutdown hours:	0.0	0.0	398.0
17. Gross thermal energy generated (MWh):	1,130,427.0	11,907,496.0	302,820,268.0 *
18. Gross electrical energy generated (MWeH):	377,729.0	3,940,337.0	99,262,919.0 *
19. Net electrical energy generated (MWeH):	358,405.9	3,740,072.5	94,301,274.1 *
20. Unit service factor:	100.0	78.9	76.9
21. Unit availability factor:	100.0	78.9	77.0
22. Unit capacity factor using MDC net:	86.0	76.2	75.1
23. Unit capacity factor using DER net:	82.8	73.4	71.1
24. Unit forced outage rate:	0.0	2.2	5.5
25. Forced outage hours:	0.0	156.7	10,235.2
26. Shutdowns scheduled over next 6 months (type,date, duration):	NONE		
27. If currently shutdown, estimated startup date:	N/A		

* Cumulative values from the first criticality (07/24/67). (The remaining cumulative values are from the first date of commercial operation, 01/01/68).

AVERAGE DAILY UNIT POWER LEVEL

Docket No: 50-213

Unit: Connecticut Yankee
Haddam Neck

Date: January 15, 1994

Completed By: K. nmons/M. Bigalbal

Month: December 1994

Telephone: (203) 267-3654

<u>DAY</u>	<u>AVERAGE POWER LEVEL</u> (MWe-Net)	<u>DAY</u>	<u>AVERAGE POWER LEVEL</u> (MWe-Net)
1	<u>289</u>	17	<u>585</u>
2	<u>290</u>	18	<u>585</u>
3	<u>287</u>	19	<u>585</u>
4	<u>286</u>	20	<u>585</u>
5	<u>288</u>	21	<u>585</u>
6	<u>287</u>	22	<u>585</u>
7	<u>288</u>	23	<u>585</u>
8	<u>287</u>	24	<u>585</u>
9	<u>108</u>	25	<u>585</u>
10	<u>342</u>	26	<u>585</u>
11	<u>492</u>	27	<u>584</u>
12	<u>584</u>	28	<u>585</u>
13	<u>586</u>	29	<u>585</u>
14	<u>586</u>	30	<u>585</u>
15	<u>572</u>	31	<u>585</u>
16	<u>586</u>		

UNIT SHUTDOWNS AND POWER REDUCTION

Docket No: 50-213

Unit Name: Connecticut Yankee

Date: January 15, 1994

Completed By: Kathy Emmons

Telephone: (203) 267-3654

Report Month: December 1993

No.	Date	Type	Duration (Hours)	Reason	Method of Shutting down Reactor	LER Report #	System Code	Component Code	Cause and Corrective Action to Prevent Recurrence
93-09 (con't)	12/01	F	196.05	A	5	n/a	SK	P	High vibration on the 'B' main feed pump. Repair 'P' feed pump impeller - continued from November
94-01	12/09	F	9.57	A	5	n/a			Reduced load to resolve feedwater MOV design concern

TYPE

F Forced
S Scheduled

REASON

A Equipment Failure
B Maintenance or Test
C Refueling
D Regulatory Restriction
E Operator Training
F Administrative
G Operator Error
H Other (Explain)

METHOD

1 Manual
2 Manual Scram
3 Automatic Scram
4 Continued
5 Reduced Load
9 Other

SYSTEM

IEEE Standard
805-1984 and/or
NUREG-0161 Exhibit F

COMPONENT

IEEE Standard
803A-1983 and/or
NUREG-0161 Exhibit H

MAINTENANCE DEPARTMENT

Report Month - December 1993

System or Component	MALFUNCTION		Effect on Safe Operation	Corrective Action Taken to Prevent Repetition	Special Precautions Taken To Provide For Reactor Safety During Repair
	Cause	Result			
There were no reportable items for the Maintenance Department in December 1993					

I&C DEPARTMENT

Report Month -December 1993

System or Component	MALFUNCTION		Effect on Safe Operation	Corrective Action Taken to Prevent Repetition	Special Precautions Taken To Provide For Reactor Safety During Repair
	Cause	Result			
There were no reportable items for the I&C Department in December 1993					

CONNECTICUT YANKEE
REACTOR COOLANT DATA
DECEMBER 1993

Reactor Coolant Analysis

	minimum	average	maximum
pH @ 25 degrees C	6.1	6.3	6.4
Conductivity (uS/cm)	17.5	20.5	24.5
Chlorides(ppb)	10.1	12.8	17.9
Dissolved Oxygen (ppb)	<5.0	<5.0	<5.0
Boron (ppm)	980	1157	1392
Lithium (ppb)	1748	1949	2204
Total Act. (uCi/ml)	5.65E-02	5.65E-02	9.14E-01
I-131/I33 ratio	5.20E-01	9.67E+00	6.55E+01
I-131 (uCi/ml)	3.50E-04	3.66E-03	7.19E-02
Crud (mg/l)	<1.0	<1.0	<1.0
Tritium (uCi/ml)	1.35E+00	2.06E+00	2.47E+00
Hydrogen (cc/kg)	28.1	34.5	46.9

Aerated liquid waste processed (gallons):

1.09E+05

Waste liquid processed through boron recovery (gallons):

4.40E+04

Average primary leak rate (gallons per minute):

1.99E-01

Primary to secondary leak rate (gallons per day):

2.07E+01

Performed By: C. M. Hange

Date: 1-11-94

Reviewed By: Scott Hand

Date: 1-11-94

Approved By: _____

Date: 12 Jan 94

Refueling Information Request

1. Name of facility
Haddam Neck
2. Scheduled date for next refueling shutdown.
November 12, 1994
3. Scheduled date for restart following refueling.
January 5, 1995
4. (a) Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?
Unknown at this time
(b) If answer is yes, what, in general, will these be?
(c) If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload?
n/a
(d) If no such review has taken place, when is it scheduled?
n/a
5. Scheduled date(s) for submitting proposed licensing action and supporting information.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
Change in fuel vendor from B&W Fuel Co. to Westinghouse Electric Corp., and change in fuel assembly design.
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.
(a) 157 (b) 809
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.
1168
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.
1998