



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

MAY 12 1980

Power Authority of the State of New York  
ATTN: Mr. J. P. Bayne, Resident Manager  
Indian Point 3 Nuclear Power Plant  
P. O. Box 215  
Buchanan, New York 10511

Gentlemen:

The U. S. Nuclear Regulatory Commission (NRC) presently has underway a program to reassess its requirements regarding selection, training and licensing of all categories of personnel involved in the operation and maintenance of nuclear power plants. As a part of this program, the NRC has contracted an outside organization, Analysis & Technology, Inc., to provide an independent perspective regarding the requirements for operator licensing. In support of this effort, a survey team from Analysis & Technology will visit selected reactor sites and simulator facilities during the period of May 14 to June 27, 1980. Indian Point, Unit No. 3 has been selected as one of the sites. Analysis & Technology would like to visit the site May 21 - 23, 1980. This visit will coincide with one to Indian Point, Unit No. 2.

The objective of these visits will be to collect information on NRC/industry requirements and practices regarding the selection, prescreening, training, examination, requalification and performance of control room operators, senior operators and other operational and maintenance personnel. Selection of these sites was based on a desire that they be representative of all existing facilities and provide an adequate sampling of different training programs. It is in no way intended for these surveys to constitute an audit of an individual facility's practices, but rather, that they provide a sampling of industry-wide practices for subsequent analysis.

The results of this analysis will lead to recommendations for changes to NRC/industry practices (where appropriate) which will provide increased assurance of the operational safety of nuclear power plants. The major parts of these surveys will consist of document research and personnel interviews. Survey teams will consist of from two to five people.

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Enclosure (1) provides a list of specific documentation desired to be made available, a list of personnel desired for interviews and additional support required for both reactor site and simulator facility visits. In addition, a proposed general schedule for the conduct of these visits is included.

A vital part of this research will consist of the determination of predictive indices of operator and trainee performance based on a collection of data from review of personnel training, performance and background records. These predictive indices will be used in the evaluation of selection and prescreening criteria and determination of prerequisites for advancement. Analysis & Technology personnel are authorized by the NRC to review these records. It is fully realized that these records are considered to be of a confidential nature. To maintain this confidentiality, prior to departure from the site, information collected from these records will be correlated to an individual by use of a letter-number code rather than the use of an individual's name, social security number, etc. Similarly, any information collected that is considered by the facility to be of a proprietary nature will be treated as such. Facilities are requested to follow their normal administrative procedures in granting release of these types of information (confidential and/or proprietary) to Analysis & Technology survey personnel.

Your cooperation in this effort is appreciated. If any questions arise, or the scheduled dates for visiting the sites are not satisfactory, please contact myself or Mr. Joseph Buzy at (301) 492-7486. For your information, we have requested a final report from Analysis & Technology by mid-December 1980. Therefore, it is necessary to complete the site visits by the end of June.

Sincerely,

Paul F. Collins, Chief  
Operator Licensing Branch  
Division of Human Factors Safety

Enclosure:  
Information Sources and Support  
Required for Reactor Site and  
Simulator Facility Visits

cc w/o enclosures:  
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