



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

BOSTON EDISON COMPANY

DOCKET NO. 50-293

PILGRIM NUCLEAR POWER STATION  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 62  
License No. DPR-35

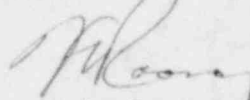
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Boston Edison Company (the licensee) dated March 19, 1982 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;  
and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-35 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 62, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3 This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Vernon L. Rooney, Acting Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 5, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 62

FACILITY OPERATING LICENSE NO. DPR-35

DOCKET NO. 50-293

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised page is identified by Amendment Number and contains a vertical line indicating the area of change.

Remove

44a  
59a  
103  
104  
137g  
137h  
137i  
194

Replace

44a  
59a  
103  
104  
137g  
137h  
137i  
194

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

G. Recirculation Pump Trip/Alternate Rod Insertion Initiation

This system is only required when the reactor mode switch is in the RUN mode.

The recirculation pump trip system causes a pump trip and the alternate rod insertion system provides for initiating control rod insertion on a signal of high reactor pressure or low-low reactor water level when the mode select switch is in the RUN mode.

The limiting conditions for operation for the instrumentation are listed in Table 3.2-G.

G. Recirculation Pump Trip/Alternate Rod-Insertion

Surveillance for instrumentation which initiates Recirculation Pump Trip and Alternate Rod Insertion shall be specified in Table 4.2-G.

PNPS

TABLE 3.2-G

INSTRUMENTATION THAT INITIATES RECIRCULATION PUMP TRIP  
AND  
ALTERNATE ROD INSERTION

Minimum Number of Operable or Tripped Instrument Channels Per Trip System (1)	Trip Function	Trip Level Setting
2	High Reactor Dome Pressure	1175 ± 15 PSIG
2	Low-Low Reactor Water Level	≥ 78.5" above the top of the active fuel

- Actions (1) There shall be two (2) operable trip systems for each function.
- (a) If the minimum number of operable or tripped instrument channels for one (1) trip system cannot be met, restore the trip system to operable status within 14 days or be in at least hot shutdown within 24 hours.
  - (b) If the minimum operability conditions (1.a) cannot be met for both (2) trip systems, be in at least hot shutdown within 24 hours.

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENT

3.5 CORE AND CONTAINMENT COOLING SYSTEMS

4.5 CORE AND CONTAINMENT COOLING SYSTEMS

Applicability

Applicability

Applies to the operational status of the core and suppression pool cooling subsystems.

Applies to the Surveillance Requirements of the core and suppression pool cooling subsystems which are required when the corresponding Limiting Condition for operation is in effect.

Objective

Objective

To assure the operability of the core and suppression pool cooling subsystems under all conditions for which this cooling capability is an essential response to station abnormalities.

To verify the operability of the core and suppression pool cooling subsystems under all conditions for which this cooling capability is an essential response to station abnormalities.

Specification

Specification

A. Core Spray and LPCI Subsystems

A. Core Spray and LPCI Subsystem

1. Both core spray subsystems shall be operable whenever irradiated fuel is in the vessel and prior to reactor startup from a Cold Condition, except as specified in 3.5.A.2 below.

1. Core Spray Subsystem Testing.

<u>Item</u>	<u>Frequency</u>
a. Simulated Automatic Actuation test.	Once/Operating Cycle
b. Pump Operability	Once/month
c. Motor Operated Valve Operability	Once/month and Once/cycle from the Alternate Shutdown Panel
d. Pump flow rate Each pump shall deliver at least 3600 gpm against a system head corresponding to a reactor vessel pressure of 104 psig.	
e. Core Spray Header $\Delta$ p Instrumentation	

LIMITING CONDITIONS FOR OPERATION

3.5.A Core Spray and LPCI Subsystems  
(cont'd)

2. From and after the date that one of the core spray subsystems is made or found to be inoperable for any reason, continued reactor operation is permissible during the succeeding seven days, provided that during such seven days all active components of the other core spray subsystem and active components of the LPCI subsystem and the diesel generators are operable.
3. The LPCI Subsystems shall be operable whenever irradiated fuel is in the reactor vessel, and prior to reactor startup from a Cold Condition, except as specified in 3.5.A.4, 3.5.A.5 and 3.5.F.5.

SURVEILLANCE EQUIPMENT

4.5.A Core Spray and LPCI Subsystems  
(cont'd)

- |           |               |
|-----------|---------------|
| Check     | Once/day      |
| Calibrate | Once/3 months |
| Test      | Once/3 months |
2. When it is determined that one core spray subsystem is inoperable, the operable core spray subsystem, the LPCI subsystem and the diesel generators shall be demonstrated to be operable immediately. The operable core spray subsystem shall be demonstrated to be operable daily thereafter.
  3. LPCI Subsystem Testing shall be as follows:
    - a. Simulated Automatic Actuation Test      Once/Operating Cycle
    - b. Pump Operability      Once/month
    - c. Motor Operated valve operability      Once/Month and Once/cycle from the Alternate Shutdown Panel
    - d. Pump Flow      Once/3 months

Three LPCI pumps shall deliver 14,400 gpm against a system head corresponding to a vessel pressure of 20 psig

Table 3.6.I(a)

## SAFETY RELATED HYDRAULIC SHOCK SUPPRESSORS (SNUBBERS)

Snubber No.	Location	Elevation	Snubber in High Radiation Area During Shutdown	Snubbers Especially Difficult to Remove	Snubbers Inaccessible During Normal Operation	Snubbers Accessible During Normal Operation
SS-6-10-1	Feedwater System	42'			X (Drywell)	
SS-6-10-2	Feedwater System	42'			X (Drywell)	
SS-6-10-3	Feedwater System	42'			X (Drywell)	
SS-6-10-4	Feedwater System	42'			X (Drywell)	
SS-6-10-5	Feedwater System	42'			X (Drywell)	
SS-13-3-1	RCIC	38'			X (Drywell)	
SS-13-3-2	RCIC	38'			X (Drywell)	
SS-14-3-1	Core Spray	65'			X (Drywell)	
SS-14-3-2	Core Spray	65'			X (Drywell)	
SS-14-3-3	Core Spray	65'			X (Drywell)	
SS-14-3-4	Core Spray	65'			X (Drywell)	
SS-23-10-1	H.P.C.I.	42'			X (Drywell)	
SS-23-10-2	H.P.C.I.	42'			X (Drywell)	
SS-23-3-30	H.P.C.I.	-3' 09"				X H.P.C.I. Quadrant
SS-23-20-31	H.P.C.I.	-3' 09"				X H.P.C.I. Quadrant
SS-23-10-32	H.P.C.I.	-3' 09"				X H.P.C.I. Quadrant
SS-23-10-34	H.P.C.I.	-6'				X H.P.C.I. Quadrant
SS-23-10-35	H.P.C.I.	-6'				X H.P.C.I. Quadrant
SS-23-20-36	H.P.C.I.	-3' 09"				X H.P.C.I. Quadrant
SS-23-20-37	H.P.C.I.	-3' 09"				X H.P.C.I. Quadrant
SS-10-3-43	RHR	-3' 06"				X RHR Pump Room
SS-10-20-44	RHR	-3' 06"				X RHR Pump Room
SS-30-3-45	RBCCW	83' 5"				X Reactor Building
SS-10-10-46	RHR	6"				X Torus Compartment
SS-23-10-38	HPCI	-3' 09"				X HPCI Quadrant
SS-23-10-39	HPCI	-3' 09"				X HPCI Quadrant

Modifications to this Table due to changes in high radiation areas should be submitted to the NRC as part of the next license amendment.



Table 3.0.1(b)

## SAFETY RELATED MECHANICAL SHOCK SUPPRESSORS (SNUBBERS)

Snubber No.	Location	Elevation	Snubber in High Radiation Area During Shutdown	Snubbers Especially Difficult to Remove	Snubbers Inaccessible During Normal Operation	Snubbers Accessible During Normal Operation
SS-1-10-13	M.S. Relief Line A				X (Drywell)	
SS-1-10-14	M.S. Relief Line A				X (Drywell)	
SS-1-10-15	M.S. Relief Line A				X (Drywell)	
SS-1-10-16	M.S. Relief Line A				X (Drywell)	
SS-1-10-17	M.S. Relief Line A				X (Drywell)	
SS-1-10-18	M.S. Relief Line A				X (Drywell)	
SS-1-10-19	M.S. Relief Line A				X (Drywell)	
SS-1-10-20	M.S. Relief Line A				X (Drywell)	
SS-1-10-21	M.S. Relief Line B				X (Drywell)	
SS-1-10-22	M.S. Relief Line B				X (Drywell)	
SS-1-10-23	M.S. Relief Line B				X (Drywell)	
SS-1-10-24	M.S. Relief Line B				X (Drywell)	
SS-1-10-25	M.S. Relief Line B				X (Drywell)	
SS-1-10-26	M.S. Relief Line B				X (Drywell)	
SS-1-10-27	M.S. Relief Line B				X (Drywell)	
SS-1-10-28	M.S. Relief Line B				X (Drywell)	
SS-1-10-29	M.S. Relief Line C				X (Drywell)	
SS-1-10-30	M.S. Relief Line C				X (Drywell)	
SS-1-10-31	M.S. Relief Line C				X (Drywell)	
SS-1-10-32	M.S. Relief Line C				X (Drywell)	
SS-1-10-33	M.S. Relief Line C				X (Drywell)	
SS-1-10-34	M.S. Relief Line C				X (Drywell)	
SS-1-10-35	M.S. Relief Line C				X (Drywell)	
SS-1-10-36	M.S. Relief Line C				X (Drywell)	
SS-1-10-37	M.S. Relief Line C				X (Drywell)	

Table 3.0.1(b)

SAFETY RELATED MECHANICAL SHOCK SUPPRESSORS (SNUBBERS)

Snubber No.	Location	Elevation	Snubber in High Radiation Area During Shutdown	Snubbers Especially Difficult to Remove	Snubbers Inaccessible During Normal Operation	Snubbers Accessible During Normal Operation
SS-1-10-38	M.S. Relief Line D				X (Drywell)	
SS-1-10-39	M.S. Relief Line D				X (Drywell)	
SS-1-10-40	M.S. Relief Line D				X (Drywell)	
SS-1-10-41	M.S. Relief Line D				X (Drywell)	
SS-1-10-42	M.S. Relief Line D				X (Drywell)	
SS-1-10-43	M.S. Relief Line D				X (Drywell)	
SS-1-10-44	M.S. Relief Line D				X (Drywell)	
SS-1-10-45	M.S. Relief Line D				X (Drywell)	
SS-1-10-46	M.S. Relief Line D				X (Drywell)	

3.9 AUXILIARY ELECTRICAL SYSTEMApplicability:

Applies to the auxiliary electrical power system.

Objective:

To assure an adequate supply of electrical power for operation of those systems required for safety.

Specification:A. Auxiliary Electrical Equipment

The reactor shall not be made critical unless all of the following conditions are satisfied:

1. At least one offsite transmission line and the startup transformer are available and capable of automatically supplying auxiliary power to the emergency buses.
2. An additional source of offsite power consisting of one of the following:
  - a. A transmission line and shutdown transformer capable of supplying power to the emergency 4160 volt buses.
  - b. The main transformer and unit auxiliary transformer available and capable of supplying power to the emergency 4160 volt buses.
3. Both diesel generators shall be operable. Each diesel generator shall have a minimum of 19,800 gallons of diesel fuel on site.

Amendment No. 42, 62

4.9 AUXILIARY ELECTRICAL SYSTEMApplicability:

Applies to the periodic testing requirements of the auxiliary electrical systems.

Objective:

Verify the operability of the auxiliary electrical system.

Specification:A. Auxiliary Electrical Equipment Surveillance

1. Diesel Generators
  - a. Each diesel generator shall be manually started and loaded once each month to demonstrate operational readiness. The test shall continue for at least a one hour period at rated load.

During the monthly generator test the diesel generator starting air compressor shall be checked for operation and its ability to recharge air receivers. The operation of the diesel fuel oil transfer pumps shall be demonstrated, and the diesel starting time to reach rated voltage and frequency shall be logged.

Once per operating cycle the diesel generator control circuits shall be isolated from the cable spreading room and the diesel generator started and loaded.

- b. Once per operating cycle the condition under which the diesel generator is required will be simulated and a test conducted to demonstrate that it will start and accept the emergency load within the specified time sequence. The results shall be logged.