

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285

UNIT Fort Calhoun #1

DATE December 9, 1981

COMPLETED BY R. W. Short

TELEPHONE (402)536-4543

MONTH November, 1981

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0.0	17	0.0
2	0.0	18	0.0
3	0.0	19	0.0
4	0.0	20	0.0
5	0.0	21	0.0
6	0.0	22	0.0
7	0.0	23	0.0
8	0.0	24	0.0
9	0.0	25	0.0
10	0.0	26	0.0
11	0.0	27	0.0
12	0.0	28	0.0
13	0.0	29	0.0
14	0.0	30	0.0
15	0.0	31	
16	0.0		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

8112180293 811209
PDR ADOCK 05000285
R PDR

OPERATING DATA REPORT

DOCKET NO 50-285
 DATE December 9, 1981
 COMPLETED BY R. W. Short
 TELEPHONE (402) 536-4543

OPERATING STATUS

1. Unit Name: Fort Calhoun Station Unit No. 1
2. Reporting Period: November, 1981
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 501
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 501
7. Maximum Dependable Capacity (Net MWe): 478
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

Notes

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>720.0</u>	<u>8,016.0</u>	<u>71,737.0</u>
12. Number Of Hours Reactor Was Critical	<u>0.0</u>	<u>6,130.6</u>	<u>55,018.9</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>1,309.5</u>
14. Hours Generator On-Line	<u>0.0</u>	<u>6,080.3</u>	<u>54,840.2</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>0.0</u>	<u>6,728,000.2</u>	<u>66,446,240.5</u>
17. Gross Electrical Energy Generated (MWH)	<u>0.0</u>	<u>2,200,498.0</u>	<u>21,988,769.6</u>
18. Net Electrical Energy Generated (MWH)	<u>0.0</u>	<u>2,069,985.6</u>	<u>20,768,168.5</u>
19. Unit Service Factor	<u>0.0</u>	<u>75.9</u>	<u>76.4</u>
20. Unit Availability Factor	<u>0.0</u>	<u>75.9</u>	<u>76.4</u>
21. Unit Capacity Factor (Using MDC Net)	<u>0.0</u>	<u>53.8</u>	<u>63.2</u>
22. Unit Capacity Factor (Using DER Net)	<u>0.0</u>	<u>53.8</u>	<u>62.9</u>
23. Unit Forced Outage Rate	<u>0.0</u>	<u>2.9</u>	<u>3.9</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: December 14, 1981
26. Units In Test Status (Prior to Commercial Operation): N/A

	Forecast	Achieved
INITIAL CRITICALITY	-----	-----
INITIAL ELECTRICITY	-----	-----
COMMERCIAL OPERATION	-----	-----

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH November, 1981

DOCKET NO. 50-285
 UNIT NAME Fort Calhoun #1
 DATE December 9, 1981
 COMPLETED BY R. W. Short
 TELEPHONE (402) 536-4543

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
81-06	810918	S	1754.1	C	1	N/A	N/A	N/A	Reactor shutdown for annual refueling and maintenance outage.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

Exhibit I - Same Source

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending November 1981.

1. Scheduled date for next refueling shutdown. December 1, 1982
2. Scheduled date for restart following refueling. March 1, 1983
3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes
- a. If answer is yes, what, in general, will these be?

A Technical Specification change.

- b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. _____
- c. If no such review has taken place, when is it scheduled? _____
4. Scheduled date(s) for submitting proposed licensing action and support information. September 1, 1982
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
6. The number of fuel assemblies:
- | | | |
|---|------------|------------|
| a) in the core | <u>133</u> | assemblies |
| b) in the spent fuel pool | <u>237</u> | " |
| c) spent fuel pool storage capacity | <u>483</u> | " |
| d) planned spent fuel pool storage capacity | <u>483</u> | " |
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1985

Prepared by

JK Gayer

Date

December 3, 1981

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

November 1981
Monthly Operations Report

I. OPERATIONS SUMMARY

The Operations staff at Fort Calhoun Station participated in the scheduled Refueling Outage tests and Maintenance activities.

The staff completed a verification walkdown of Fort Calhoun piping and instrumentation drawings to support a timely update of drawings to be used for Fort Calhoun startup.

Three new employees joined the operations staff in November. The new Auxiliary Operators were participating in the Districts training program.

Special training was conducted by Combustion Engineering at Fort Calhoun Station on degraded core performance. Cycle 7 training is in progress. No safety valve or PCRV challenges occurred.

A. PERFORMANCE CHARACTERISTICS

<u>LER Number</u>	<u>Deficiency</u>
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NONE

B. CHANGES IN OPERATING METHODS

NONE

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

Surveillance tests as required by the Technical Specifications Section 3.0 and Appendix B, were performed in accordance with the annual surveillance test schedule. The following is a summary of the surveillance tests which results in Operations Incidents and are not reported elsewhere in the report:

<u>Operation Incidents</u>		<u>Deficiency</u>
OI-1414	ST-ESF-7, F.3	D/LC-383-1 and D/LC-383-2 left and right level switches were out of setpoint tolerances.
OI-1415	ST-ISI-CC-3	Missed 96 hours ISI signoff.
OI-1420	ST-RM-4,	Environmental Air Sampler operational checks reading low flow.
OI-1423	ST-FD-10,	Not performed within the allowable time span of $\pm 25\%$.

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS (Continued)

<u>Operations Incidents</u>	<u>Deficiency</u>
OI-1426 ST-ISI-CC-1 Component Cooling Water.	QC and Shift Supervisor not notified prior to start of test.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

<u>Procedure</u>	<u>Description</u>
SP-STRETCH-1	<p>Power Ascension to 1500 mwth.</p> <p>The purpose of Special Procedure SP-STRETCH-1 was to provide a safe and orderly method for initial power ascension to 1500 mwth from the previously licensed power level. After having received the duly approved license amendment from the NRC to raise the Fort Calhoun rated power level from 1420 mwth to 1500 mwth, power was raised in a controlled manner per this Special Procedure. The method used in raising power utilized existing plant procedures, guidance from Combustion Engineering, and guidelines from the Safety Analysis which was prepared to justify the power increase. Power was raised from 1420 mwth to 1500 mwth successfully and without incident.</p> <p>This procedure did not involve an unreviewed safety question per 10CFR50.59. Power ascension was in accordance with Technical Specification requirements which were in turn justified to, and approved by, the Commission.</p>
SP-FAUD-1	<p>An unreviewed safety question as defined in 10CFR50.59 did not exist as this procedure only involves evaluating data from a surveillance test.</p>

E. RESULTS OF LEAK RATE TESTS

Annual Leak Rate Tests are in progress. A consolidated report will be made at the end of testing.

F. CHANGES IN PLANT OPERATING STAFF

The following personnel were promoted to the following positions and made effective December 1, 1981.

Merl R. Core - Supervisor - I&C and Electrical Field Maintenance.
Bruce J. Hickie - Supervisor - Chemistry and Radiation Protection.
Dick Hyde - Supervisor - Field Maintenance

Two new STA employees joined the Technical staff in November.

Three new employees joined the Operations staff in November.

G. TRAINING

Training during the month of November consisted of General Employee Training, Crane Operator Training, Hot License Training for NRC Examinations, Monitor Team and Emergency Duty Officer Training.


H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59.

NONE

II. MAINTENANCE (Significant Safety Related).

M.O. #	Date	Description	Corrective Action
12326	10-20-81	RM-050 is spiking in alert.	Balanced alarm board.
11723	9-29-81	Replaced RPS-D channel B-10 detector.	Removed old detector and replaced per MP-PR-WRD-1.
11810	9-29-81	Replaced RPS-B channel B-10 detector.	Removed old detector and replaced per MP-PR-WRD-1.
9353	10-30-81	Couple Control Element Assemblies MP-RC-10-7.	Completed per procedure.
12366	10-22-81	HCV-403A/C will not operate via the control switch.	Installed new coil in solenoid valve.

Approved By:


S.C. Stevens
Manager-Fort Calhoun Station