Attachment 1

Haddam Neck Plant

Proposed Revision to Technical Specifications Reactor Coolant Pump Flywheel Inspections Marked Up Pages to Technical Specifications

December 1993

REACTOR COOLANT SYSTEM

3/4.4.10 STRUCTURAL INTEGRITY

LIMITING CONDITION FOR OPERATION

3.4.10 The structural integrity of ASME Code Class 1, 2, and 3 components shall be maintained in accordance with Specification 4.4.10.

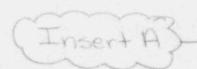
APPLICABILITY: All MODES.

ACTION:

- a. With the structural integrity of any ASME Code Class I component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature more than 50°F above the minimum temperature required by NDTT considerations.
- b. With the structural integrity of any ASME Code Class 2 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature above 200°F.
- With the structural integrity of any ASME Code Class 3 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) from service.

SURVEILLANCE REQUIREMENTS

4.4.10 In addition to the requirements of Specification 4.0.5, leach reactor coolant pump flywheel shall be inspected per the recommendations of Regulatory Position C.4.b of Regulatory Guide 1.14, Revision 1, August 1975.



HADDAM NECK

3/4 4-48

Amendment No. 125

Insert A

"the areas of higher stress concentration at the bore and keyway of each reactor coolant pump flywheel shall be ultrasonically examined at least once during each 10-year inspection interval. Scheduled examinations shall not exceed any 10-year period of operational service."

Attachment 2

Haddam Neck Plant

Proposed Revision to Technical Specifications Reactor Coolant Pump Flywheel Inspections Retyped Pages to Technical Specifications

REACTOR COOLANT SYSTEM

3/4.4.10 STRUCTURAL INTEGRITY

LIMITING CONDITION FOR OPERATION

3.4.10 The structural integrity of ASME Code Class 1, 2, and 3 components shall be maintained in accordance with Specification 4.4.10.

APPLICABILITY: All MODES.

ACTION:

- a. With the structural integrity of any ASME Code Class 1 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature more than 50°F above the minimum temperature required by NDTT considerations.
- b. With the structural integrity of any ASME Code Class 2 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature above 200°F.
- c. With the structural integrity of any ASME Code Class 3 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) from service.

SURVEILLANCE REQUIREMENTS

4.4.10 In addition to the requirements of Specification 4.0.5, the areas of higher stress concentration at the bore and keyway of each reactor coolant pump flywheel shall be ultrasonically examined at least once during each 10-year inspection interval. Scheduled examinations shall not exceed any 10-year period of operational service.

Attachment 3

Haddam Neck Plant

Inspection Data Base Information Matrix For RCP Flywheels

| Year Examined | RCP Flywheel Number | Inspection Method/Area Requirements(1) | Examination Methods Used (2) | Areas of Flaws/Cracks Identified (3) |
|------------------|---------------------------|--|---|---|
| 1970 | 1 2 3 4 | А | PT OF BORE AREA RCP #1 ONLY & UT ALL | NONE AND UT <5% DAC INDICATION RECORDED ON RCP #4 |
| 1971 | 1 2 3 4 | С | PT OF BORE AREA ALL & UT VT ALL | BSW ON RCP #4 |
| 1973 | 1 4 | АВ | PT INCLUDED BORE PAWL AREAS & BORE AREA ON ALL. UT VT ALL | BSW ON RCP #4 |
| 1976* | 2 | АВ | UT VT | NONE |
| 1977 | 3 | A B | UT VT | NONE |
| 1979 | 1 | A B | UT VT | NONE |
| 1980 | 1234 | A B | UT VT | NONE |
| 1981 | 3 4 | A B | UT VT | NONE |
| 1983 | 1 | A D | MT UT | NONE |
| 1984 | 1 2 | С | UT VT | NONE |
| 1986 | 3 4 | C D | MT UT VT | NONE |
| 1987 | 1 2 | ср | MT UT VT | SBF ON RCP #2 |
| 1989 | 3 4 | A | UT | NONE |
| 1991 | 1 2 | A | UT | NONE |
| 1993 | 3 4 | A | UT | NONE |

^{*}No cracks have been identified on any of the RCP flywheels in the critical areas of the bore and keyways since 1973.

NOTES

- (1) Inspection Method/Area Requirements are:
 - A = 100% Volumetric of Bore and Keyway Areas;
 - B = Visual All Accessible Surface Areas;
 - C = 100% Visual and Volumetric All Areas; and
 - D = 100% Surface All Areas.
- (2) Examination Methods Used are:
 - PT = Surface (Liquid Penetrant Examination);
 - MT = Surface (Magnetic Particle Examination);
 - UT = Volumetric (Ultrasonic Examination); and
 - VT = Visual (Visual Examination).
- (3) Areas of Flaws/Cracks Identified are:
 - BSW = Bore Seal Weld Area; and
 - SBF = Seal Baffle Fillet Weld Area.