

Docket No. 50-213
B14507

Attachment 1

Haddam Neck Plant

Proposed Revision to Technical Specifications
Reactor Coolant Pump Flywheel Inspections
Marked Up Pages to Technical Specifications

December 1993

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REACTOR COOLANT SYSTEM3/4.4.10 STRUCTURAL INTEGRITYLIMITING CONDITION FOR OPERATION

3.4.10 The structural integrity of ASME Code Class 1, 2, and 3 components shall be maintained in accordance with Specification 4.4.10.

APPLICABILITY: ALL MODES.

ACTION:

- a. With the structural integrity of any ASME Code Class 1 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature more than 50°F above the minimum temperature required by NDTT considerations.
- b. With the structural integrity of any ASME Code Class 2 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature above 200°F.
- c. With the structural integrity of any ASME Code Class 3 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) from service.

SURVEILLANCE REQUIREMENTS

4.4.10 In addition to the requirements of Specification 4.0.5, each reactor coolant pump flywheel shall be inspected per the recommendations of Regulatory Position C.4.b of Regulatory Guide 1.14, Revision 1, August 1975.

Insert A

Insert A

"the areas of higher stress concentration at the bore and keyway of each reactor coolant pump flywheel shall be ultrasonically examined at least once during each 10-year inspection interval. Scheduled examinations shall not exceed any 10-year period of operational service."

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Attachment 2

Haddam Neck Plant

Proposed Revision to Technical Specifications
Reactor Coolant Pump Flywheel Inspections
Retyped Pages to Technical Specifications

December 1993

REACTOR COOLANT SYSTEM

3/4.4.10 STRUCTURAL INTEGRITY

LIMITING CONDITION FOR OPERATION

3.4.10 The structural integrity of ASME Code Class 1, 2, and 3 components shall be maintained in accordance with Specification 4.4.10.

APPLICABILITY: ALL MODES.

ACTION:

- a. With the structural integrity of any ASME Code Class 1 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature more than 50°F above the minimum temperature required by NDTT considerations.
- b. With the structural integrity of any ASME Code Class 2 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature above 200°F.
- c. With the structural integrity of any ASME Code Class 3 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) from service.

SURVEILLANCE REQUIREMENTS

4.4.10 In addition to the requirements of Specification 4.0.5, the areas of higher stress concentration at the bore and keyway of each reactor coolant pump flywheel shall be ultrasonically examined at least once during each 10-year inspection interval. Scheduled examinations shall not exceed any 10-year period of operational service.

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Attachment 3

Haddam Neck Plant

Inspection Data Base Information Matrix For RCP Flywheels

December 1993

INSPECTION DATA BASE INFORMATION MATRIX FOR RCP FLYWHEELS				
Year Examined	RCP Flywheel Number	Inspection Method/Area Requirements(1)	Examination Methods Used (2)	Areas of Flaws/Cracks Identified (3)
1970	1 2 3 4	A	PT OF BORE AREA RCP #1 ONLY & UT ALL	NONE AND UT <5% DAC INDICATION RECORDED ON RCP #4
1971	1 2 3 4	C	PT OF BORE AREA ALL & UT VT ALL	BSW ON RCP #4
1973	1 4	A B	PT INCLUDED BORE PAWL AREAS & BORE AREA ON ALL. UT VT ALL	BSW ON RCP #4
1976*	2	A B	UT VT	NONE
1977	3	A B	UT VT	NONE
1979	1	A B	UT VT	NONE
1980	1 2 3 4	A B	UT VT	NONE
1981	3 4	A B	UT VT	NONE
1983	1	A D	MT UT	NONE
1984	1 2	C	UT VT	NONE
1986	3 4	C D	MT UT VT	NONE
1987	1 2	C D	MT UT VT	SBF ON RCP #2
1989	3 4	A	UT	NONE
1991	1 2	A	UT	NONE
1993	3 4	A	UT	NONE

*No cracks have been identified on any of the RCP flywheels in the critical areas of the bore and keyways since 1973.

NOTES

(1) Inspection Method/Area Requirements are:

A = 100% Volumetric of Bore and Keyway Areas;
B = Visual All Accessible Surface Areas;
C = 100% Visual and Volumetric All Areas; and
D = 100% Surface All Areas.

(2) Examination Methods Used are:

PT = Surface (Liquid Penetrant Examination);
MT = Surface (Magnetic Particle Examination);
UT = Volumetric (Ultrasonic Examination); and
VT = Visual (Visual Examination).

(3) Areas of Flaws/Cracks Identified are:

BSW = Bore Seal Weld Area; and
SBF = Seal Baffle Fillet Weld Area.