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AUG 1 3 1982

Mr. Harold R. Denton, Director Office of Nuclear Reactor Regulation United States Nuclear Regulatory Commission Washington, DC 20555

Subject:

Limerick Generating Station, Units 1 & 2 Final Safety Analysis Report - Applicable

Code Reference

References:

- Letter E. G. Bauer/PECo to A. Giambusso/NRC dated 7/15/75
- b) Letter R. C. DeYoung/NRC to E. G. Bauer/PECo dated 11/18/75

Files:

QUAL 2-4-2 (NRC) GOVT 1-1 (NRC)

Dear Mr. Denton:

On July 15, 1975, ref. (a) was transmitted to the NRC requesting, for certain equipment, permission to use alternative codes to those required in 10CFR50.55a for pressure boundary components. Included in the letter was the request that, for the Main Steam Isolation Valves, permission be granted to use the ASME Standard Code for Pumps and Valves for Nuclear Power-1968 Draft.

The letter from NRC (ref. b) granted the requested permission.

Both letters are referenced in Table 3.2-1 of the Limerick Final Safety Analysis Report.

A recent review of the documentation for the valves has identified that they were, in fact, designed, fabricated and tested in accordance with ASME Standard Code for pumps and valves for Nuclear Power - 1968 Draft, including March 1970 Addenda.

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As a result of the above information, arrangements have been made to revise the FSAR to reflect the Code and Addenda actually used to design, fabricate and test the Main Steam Isolation Valves.

Sincerely,

John 5 Kenten