



Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

August 4, 1982

Mr. R. C. Haynes  
Regional Administrator  
USNRC  
Region 1  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

Dear Mr. Haynes:

LICENSE NO. DPR-75  
DOCKET NO. 50-311  
REPORTABLE OCCURRENCE 82-061/03L

Pursuant to the requirements of Salem Generating Station Unit No. 2, Technical Specifications, Section 6.9.1.9.b, we are submitting Licensee Event Report for Reportable Occurrence 82-061/03L. This report is required within thirty (30) days of the occurrence.

Sincerely yours,

A handwritten signature in cursive script, appearing to read "H. J. Midura".

H. J. Midura  
General Manager -  
Salem Operations

RF:ks *JGJ*

CC: Distribution

Report Number: 82-061/03L  
Report Date: 08-04-82  
Occurrence Date: 07-16-82  
Facility: Salem Generating Station, Unit 2  
Public Service Electric & Gas Company  
Hancocks Bridge, New Jersey 08038

IDENTIFICATION OF OCCURRENCE:

No. 21 Steam Generator Level Channel II Instrument - Inoperable.

This report was initiated by Incident Report 82-182.

CONDITIONS PRIOR TO OCCURRENCE:

Mode I - Rx Power 83% - Unit Load 900 MWe.

DESCRIPTION OF OCCURRENCE:

At 1600 hours, July 16, 1982, during routine operation, the Control Room Operator noticed that No. 21 Steam Generator Level Channel II indication was erratic. The channel was declared inoperable, and Limiting Conditions for Operation 3.3.1 Action 7 and 3.3.2 Action 14 were entered. All bistables associated with the channel were immediately placed in the tripped condition. The redundant channels were operable throughout the occurrence.

DESIGNATION OF APPARENT CAUSE OF OCCURRENCE:

The erratic operation of the instrument was due to leakage past the packing on Root Valve 21BF28. The resulting variations in sensing line pressure caused the level indication to drift.

ANALYSIS OF OCCURRENCE:

The steam generator level channel II instruments are utilized in the Reactor Trip and Engineered Safety Feature Actuation Systems. A reactor trip resulting from a low steam generator level insures a minimum DNBR of 1.30 during normal operation and anticipated transients. Automatic initiation of auxiliary feedwater insures that the Reactor Coolant System can be cooled down to less than 350°F in the event of a total loss of offsite power. Due to the operability of the redundant channels, no risk to the health or safety of the public was involved. The occurrence constituted operation in a degraded mode permitted by a limiting condition for operation and is reportable in accordance with Technical Specification 6.9.1.9.b.

ANALYSIS OF OCCURRENCE: (continued)

Limiting Conditions for Operation 3.3.1 Action 7 and 3.3.2 Action 14 requires:

With the number of operable channels one less than the total number of channels, power operation may proceed until performance of the next required channel functional test, provided the inoperable channel is placed in the tripped condition within 1 hour.

CORRECTIVE ACTION:

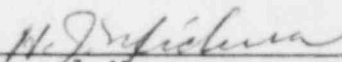
As noted, the bistables associated with the instrument were placed in the tripped condition. The root valve was repacked and the leakage was stopped. The channel was recalibrated and was returned to service. No. 21 S/G Level Channel II was declared operable, and Limiting Conditions for Operation 3.3.1 Action 7 and 3.3.2 Action 14 were terminated at 0807 hours, July 18, 1982.

FAILURE DATA:

No. 21 S/G Level Channel II instrument had failed previously February 9, 1982. The problem involved leakage from a vent on the block and vent manifold, and was corrected.

Rockwell-International  
3/4" Globe Valve  
Type 3624, Mark FA-99

Prepared By R. Frahm

  
\_\_\_\_\_  
General Manager -  
Salem Operations

SORC Meeting No. 82-73