

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
 UNIT Fort Calhoun Station
 DATE August 10, 1982
 COMPLETED BY R. W. Short
 TELEPHONE (402)536-4543

MONTH July, 1982

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	469.5	17	342.4
2	470.2	18	459.3
3	467.1	19	462.7
4	464.5	20	461.4
5	459.2	21	461.1
6	458.3	22	459.7
7	461.1	23	459.4
8	460.8	24	459.6
9	460.7	25	460.9
10	461.7	26	460.7
11	461.5	27	459.8
12	460.3	28	460.9
13	458.7	29	462.0
14	457.0	30	461.4
15	456.6	31	461.2
16	454.6		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

OPERATING DATA REPORT

DOCKET NO. 50-285
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OPERATING STATUS

1. Unit Name: Fort Calhoun Station
2. Reporting Period: July, 1982
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 501
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 501
7. Maximum Dependable Capacity (Net MWe): 478
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
None

Notes

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: None

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744.0	5,087.0	77,568.0
12. Number Of Hours Reactor Was Critical	744.0	5,020.9	61,259.9
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14. Hours Generator On-Line	744.0	5,011.8	60,101.8
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1,094,321.9	7,378,242.5	74,080,072.6
17. Gross Electrical Energy Generated (MWH)	357,514.0	2,481,297.9	24,555,243.5
18. Net Electrical Energy Generated (MWH)	340,183.1	2,366,498.3	23,214,366.8
19. Unit Service Factor	100.0	98.5	77.5
20. Unit Availability Factor	100.0	98.5	77.5
21. Unit Capacity Factor (Using MDC Net)	95.7	97.3	65.1
22. Unit Capacity Factor (Using DER Net)	95.7	97.3	64.8
23. Unit Forced Outage Rate	0.0	1.5	3.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
1983 Refueling Outage Scheduled to Commence January 3, 1983 for
Three Months.

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

26. Units In Test Status (Prior to Commercial Operation): None

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
 UNIT NAME Fort Calhoun Station
 DATE August 10, 1982
 COMPLETED BY R. W. Short
 TELEPHONE (402) 536-4543

REPORT MONTH July, 1982

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
									There were no unit shutdowns during the month of July, 1982.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit I - Same Source

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending July 1982.

1. Scheduled date for next refueling shutdown. January 3, 1983
2. Scheduled date for restart following refueling. April 1, 1983
3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes
 - a. If answer is yes, what, in general, will these be?

A Technical Specification Change

- b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. _____
 - c. If no such review has taken place, when is it scheduled? _____
4. Scheduled date(s) for submitting proposed licensing action and support information. November 1, 1982
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
6. The number of fuel assemblies:

a) in the core	<u>133</u>	assemblies
b) in the spent fuel pool	<u>237</u>	"
c) spent fuel pool storage capacity	<u>483</u>	"
d) planned spent fuel pool storage capacity	<u>728</u>	"
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1985

Prepared by

JR Gayer

Date

August 2, 1982

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

July, 1982
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station operated at a nominal 100% power for the month of July, 1982. One 24 hour power reduction to about 65% occurred on July 17, 1982 for the purpose of cleaning the condensate cooler and the bearing water coolers. The power reduction and cooler cleaning went very well and power was returned to 100% on the evening of July 17, 1982.

During July, the Auxiliary Boiler Fuel Oil Tank and the Diesel Fire Pump Fuel Oil Tank were drained, cleaned and refilled.

Also during July, the QA vault was upgraded to comply with appropriate records storage standards. In addition, the Sigma II computer was removed in preparation for the installation of the QSPDS/ERF computer.

No safety valve or PORV challenges occurred.

A. PERFORMANCE CHARACTERISTICS

LER Number

Deficiency

LER-014

During normal power operation, in the process of switching component cooling water heat exchangers, the associated inlet valves HCV-489A, HCV-490A and HCV-492A did not open. The consequence of this event was that only one component cooling water heat exchanger was operational. Tech. Spec. 2.3(1)g. requires three of four CCW heat exchangers to be operable which may be modified to two of four CCW heat exchangers operable for not more than 24 hours under Tech. Spec. 2.3(2)c.

LER-015

During the performance of surveillance test ST-ISI-WD-1, F.1, HCV-507A (the gas vent header containment isolation valve) failed to close via the control room switch. The valve was immediately closed by failing the air to the operator. Emergency Procedure EP-25 "Loss of Containment Integrity" was implemented and a Maintenance Order was written to investigate/correct the problem. Throughout the incident, the redundant gas vent header containment isolation valve remained operable and was in the closed position.

B. CHANGES IN OPERATING METHODS

NONE

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

Surveillance tests as required by the Technical Specifications Section 3.0 and Appendix B, were performed in accordance with the annual surveillance test schedule. The following is a summary of the surveillance tests which results in Operations Incidents and are not reported elsewhere in the report:

<u>Operations Incident</u>	<u>Deficiency</u>
OI-1557	ST-FP-1 FP-1B Failed to run properly

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

<u>Procedure</u>	<u>Description</u>
SP-BTUGEN-1	Control Room BTU Generation Study for air conditioning system requirements. This procedure did not constitute an unreviewed safety question as defined in 10CFR50.59. This procedure was used to measure and calculate the heat generated from equipment, personnel, and structures in the control room. Approved procedures were prepared prior to the start of work.
SP-SS-5	This procedure did not constitute an unreviewed safety question as defined in 10CFR50.59. The procedure involved inspection and disassembly of certain spent fuel assemblies for the purpose of evaluating fuel performance. Potential accidents were enveloped by FSAR Section 14.18, "Fuel Handling Incidents". Approved procedures were prepared prior to the start of work.
SP-SS-4	This procedure did not constitute an unreviewed safety question as defined in 10CFR50.59. The procedure involved inspection and disassembly of certain spent fuel assemblies for the purpose of evaluating fuel performance. Potential accidents were enveloped by FSAR Section 14.18, "Fuel Handling Incidents". Approved procedures were prepared prior to the start of work.

D. (CONTINUED)

<u>Procedure</u>	<u>Description</u>
SP-SS-3	This procedure did not constitute an unreviewed safety question as defined in 10CFR50.59. The procedure involved inspection and disassembly of certain spent fuel assemblies for the purpose of evaluating fuel performance. Potential accidents were enveloped by FSAR Section 14.18, "Fuel Handling Incidents". Approved procedures were prepared prior to the start of work.
SP-SS-2	This procedure did not constitute an unreviewed safety question as defined in 10CFR50.59. The procedure involved inspection and disassembly of certain spent fuel assemblies for the purpose of evaluating fuel performance. Potential accidents were enveloped by FSAR Section 14.18, "Fuel Handling Incidents". Approved procedures were prepared prior to the start of work.
SP-SS-1	This procedure did not constitute an unreviewed safety question as defined in 10CFR50.59. The procedure involved inspection and disassembly of certain spent fuel assemblies for the purpose of evaluating fuel performance. Potential accidents were enveloped by FSAR Section 14.18, "Fuel Handling Incidents". Approved procedures were prepared prior to the start of work.
SP-FE-8	This procedure did not constitute an unreviewed safety question as defined in 10CFR50.59. The procedure involved inspection and disassembly of certain spent fuel assemblies for the purpose of evaluating fuel performance. Potential accidents were enveloped by FSAR Section 14.18, "Fuel Handling Incidents". Approved procedures were prepared prior to the start of work.
SF-FE-9	This procedure did not constitute an unreviewed safety question as defined in 10CFR50.59. The procedure involved inspection and disassembly of certain spent fuel assemblies for the purpose of evaluating fuel performance. Potential accidents were enveloped by FSAR Section 14.18, "Fuel Handling Incidents". Approved procedures were prepared prior to the start of work.

D. (CONTINUED)

<u>Procedure</u>	<u>Description</u>
SP-FAUD-1	Fuel Assembly Uplift Condition Detection This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved evaluating data from a surveillance test.
DCR 73-19	Insulate Process Radiation Monitor Sampling Lines This modification did not involve an unreviewed safety question as defined by 10CFR50.59 as it only involved adding heat tracing to radiation monitor sample lines to improve the quality of the samples taken.
DCR 74A-92	Gai-tronics in Storeroom This modification did not involve an unreviewed safety question as defined by 10CFR50.59 as it only involved the installation of a gai-tronics communications station to the Storeroom.
DCR 74A-106	PCV-1912 This modification did not involve an unreviewed safety question as defined by 10CFR50.59 as it did not involve a safety related system, replaced pressure control valve in Circulating Water System.
DCR 74A-118	NSSS Parameters Permanent Test Rig This modification did not involve an unreviewed safety question as defined by 10CFR50.59 as it only provided a means to monitor NSSS parameters.
DCR 75A-19	Waste Filters WD-17A/B This modification did not involve an unreviewed safety question as defined by 10CFR50.59 as it only involved changing the mesh sizing of the filters in the waste disposal system.

D. (CONTINUED)

<u>Procedure</u>	<u>Description</u>
DCR 75A-25	<p>Variable Overpower Trip</p> <p>This modification did not involve an unreviewed safety question as defined by 10CFR50.59 as it added the variable overpower trip feature to the Reactor Protective System to limit the extent of any reactor power excursions.</p>
DCR 75A-44	<p>Instrument Air Isolation Valves</p> <p>This modification did not involve an unreviewed safety question as defined by 10CFR50.59 as it only provided a means to isolate portions of the instrument air system for maintenance.</p>
DCR 75A-45	<p>Redundant Pressure Indication System for HCV-348</p> <p>This modification did not involve an unreviewed safety question as defined by 10CFR50.59 as it only involved added redundant pressure indication to the shutdown cooling isolation valves.</p>
DCR 75A-48	<p>Shutdown Cooling Sampling System</p> <p>This modification did not involve an unreviewed safety question as defined by 10CFR50.59 as it only provided for a sample connection in the shutdown cooling system.</p>
DCR 76-20	<p>Remodel Health Physics Station & Locker Room</p> <p>This modification did not involve an unreviewed safety question as defined by 10CFR50.59 as it did not involve a safety related system.</p>
DCR 76-27	<p>RC Level Indication During Cold Shutdown</p> <p>This modification did not involve an unreviewed safety question as defined by 10CFR50.59 as it provided an alternate method to monitor water level in the reactor during refueling outages.</p>

D. (CONTINUED)

<u>Procedure</u>	<u>Description</u>
DCR 76-104	<p>Steam Dump & Bypass Valve Position Indication</p> <p>This modification did not involve an unreviewed safety question as defined by 10CFR50.59 as it only added position indication lights in the control room for the steam dump and bypass valves.</p>
DCR 77-91	<p>Backflush Connections on Raw Water Pumps</p> <p>This modification did not involve an unreviewed safety question as defined by 10CFR50.59 as it only provided a means to backflush raw water pumps.</p>
EE/AR 78-6	<p>Heating Coils for Auxiliary Building</p> <p>This modification did not involve an unreviewed safety question as defined by 10CFR50.59 as it only added preheater coils for the auxiliary building supply fans.</p>
EEAR 79-18	<p>Auxiliary Building Ventilation System</p> <p>This modification did not involve an unreviewed safety question as defined by 10CFR50.59 as it only provided air conditioning to the Radio Chemistry Counting Room.</p>
EEAR 79-41	<p>Condensate to Chemical Feed Isolation</p> <p>This modification did not involve an unreviewed safety question as defined by 10CFR50.59 as it did not involve a safety related system.</p>
EEAR 79-147	<p>Trip Circuit Bypass for Auxiliary Building</p> <p>This modification did not involve an unreviewed safety question as defined by 10CFR50.59 as it provided a means to bypass the trip circuit on the auxiliary building ventilation fans actuated by smoke detectors, the freezestat, and duct pressure transmitters.</p> <p>The modification will help ensure the fans can be operated in a post-accident situation.</p>

D. (CONTINUED)

Procedure

Description

EEAR 80-68

Dual Setpoints for RM-061

This modification did not involve an unreviewed safety question as defined by 10CFR50.59 as it provided a means to account for temperature inversion effects on RM-061, Radiation Monitor on the Auxiliary Building Stack.

EEAR 81-29

Thermal Stress in Piping Between Boric Acid Pumps CH-4A/B

This modification did not involve an unreviewed safety question as defined by 10CFR50.59 as it was made to reduce stress due to temperature changes in the boric acid piping.

E. RESULTS OF LEAK RATE TESTS

Leakage for P.A.L. Door was 4200 sccm as measured per ST-CONT-2, F.2.

F. CHANGES IN PLANT OPERATING STAFF

Effective August 1, 1982 two positions were approved.

A. W. Richard became Technical Supervisor and

J. J. Tesarek became the Plant Engineer.

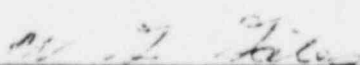
G. TRAINING

Training was continued as scheduled for general employee maintenance, licensed and non-operators, C/RP and Technical personnel.

In addition, several special lectures were given to monitor team members and emergency response personnel.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

NONE



W. G. Gates
Plant Manager

II. MAINTENANCE (Significant Safety Related)

M. O. #	Date	Description	Corrective Action
15433	6-28-82	HCV-318 relay in breaker is chattering.	Realigned contacts, cleaned and adjusted.
15273	6-25-82	Raw Water Pump AC-10A improper readings of amps.	Repaired per MP-AC-10.
15173	7-16-82	CH-1A Charging Pump. Packing Coolant Tank is filling up when standing in idle.	Repaired per MP-CH-1-1.
15497	7-8-82	FP-1 not running properly.	Cleaned out clogged line.
15526	7-14-82	RM-050/51 stopped and won't restart.	Replaced pump.
15503	7-14-82	CH-1B charging pump is leaking into Packing Coolant Tank.	Repacked CH-1B and installed new discharge valve.