



**Commonwealth Edison**  
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January 14, 1981

Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

Subject: Zion Station Units 1 and 2  
Containment Purge Valves  
NRC Docket Nos. 50-295 and 50-304

References (a): October 23, 1979 letter from A. Schwencer  
to D. L. Peoples

(b): January 14, 1981 letter from I. R. Tramm  
to H. R. Denton

Dear Mr. Denton:

This is to supplement the information contained in Reference (b) regarding our plans to use Zion's containment purge valves in accordance with the NRC guidance transmitted in Reference (a). Additional information is provided regarding the conditions which justify the purging discussed in Reference (b).

Containment purging is to be conducted in the interest of personnel safety and maintaining occupational exposure as low as reasonably achievable. The containment atmosphere is relatively warm and contains radioactive gases such as iodine and xenon. The atmosphere will be purged so that equipment will be accessible for surveillance and maintenance associated with Technical Specification requirements.

One activity which establishes the need to purge prior to reaching cold shutdown is a visual examination of the reactor coolant system. The purpose of this examination is to identify sources of minor leakage; it is best performed at elevated temperature and pressure. Postponement to a time when the reactor is in the cold shutdown or refueling mode would significantly reduce the usefulness of the examination.

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This visual examination is considered good operating practice and is also necessitated by requirements of Zion's Technical Specifications. Section 3.3.3 of the Tech Specs requires identification of leakage from the reactor coolant system. Specific limits are placed on unidentified leakage and the bases emphasize the importance of minimizing unidentified leakage. Section 4.3.4 requires inspection of the reactor coolant system following refueling in accordance with Section XI of the ASME Boiler and Pressure Vessel Code. Satisfactory completion of that requirement is partially dependent upon repair of leaks identified in the inspection at the beginning of the refueling outage.

Please address further questions regarding this matter to my office.

Very truly yours,

T. R. Tramm  
Nuclear Licensing Administrator  
Pressurized Water Reactors

cc: Zion - Resident Office

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