AUGUST : Docket No. 50-289	2 1 992	L PDR ORB#4 Rdg DEisenhut OELD AEOD	RJacobs RIngram Gray File+4 ASLAB EBlackwood HOrnstein MPadovan	DMB016
Mr. Henry D. Hukill Vice President GPU Nuclear Corporation P. O. Box 480 Middletown, Pennsylvania	17057	IE-2 ACRS-10 TBarnhart-4 LSchneider OGC OPA DBrinkman	ERossi BSnyder	

Dear Mr. Hukill:

. .

The staff has completed its review of your response to NUREG-0737, Item II.K.2.9 (Long Term Order Item 1) "Failure Mode Effects Analysis on the Integrated Control System" for Three Mile Island Nuclear Station, Unit No. 1 (TMI-1). We find that the Unit 1 design meets all current regulatory requirements. The staff has not identified any control system failures or actions that would lead to unacceptable consequences at TMI-1. Our supplemental Safety Evaluation is enclosed.

RDiggs

The issue of ICS failures will no longer be pursued on a plant by plant basis. All vendor designs and all control systems that affect plant safety will now be reviewed under Unresolved Safety Issue (USI) A-47. The purpose of this USI is to perform in depth evaluations of control systems and to evaluate the adequacy of current licensing requirements.

Consequently, no additional licensing action is to be taken at this time, and NUREG-0737 Item II.K.2.9 is considered closed for TMI-1. Resolution of USI A-47 will determine if it will be necessary to impose additional and more stringent requirements on control systems in the future.

Sincerely,

"ORIGINAL SIGNED BY JOHN F. STOLZ"

John F. Stolz, Chief Operating Reactors Branch #4 Division of Licensing

Enclosure: Safety Evaluation

cc w/enclosure: See next page

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SUPPLEMENTAL SAFETY EVALUATION REPORT

THREE MILE ISLAND NUCLEAR STATION, UNIT 1 (TMI-1)

D. LONG TERM ACTIONS

ORDER ITEM 1

(NUREG-0737, ITEM II.K.2.9)

ICS FAILURE MODE AND EFFECTS ANALYSIS

It was ordered that the licensee demonstrate reasonable progress toward satisfactory completion of the following action:

"1. Submit a failure mode and effects analysis of the ICS to the NRC staff as soon as practicable."

In NUREG-0680 (P. D1-1) and Supplement No. 3 to NUREG-0680 (P. 49) the staff reported that Metropolitan Edison had referenced BAW-1564, "Integrated Control System Reliability Analysis," for the failure mode and effects analysis applicable to TMI-1 and responded to the B&W recommended actions contained in BAW-1564. The staff further indicated in Supplement No. 3 to NUREG-0680 (P. 49) that the staff had not identified any ICS related malfunctions that could not be adequately mitigated by plant safety systems and that the licensee had:

- committed to modify the electrical distribution systems to the ICS to ensure that the operator can take prompt action to minimize the consequences of a failure in the normal power supply
- committed to investigating other modifications which would increase the reliability of the ICS power supply
- 3. described actions taken at TMI to minimize feedwater oscillations
- 4. described procedures and operator training for operation of the ICS.

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In letters L1L 310 dated November 9, 1981, and 5211-82-149 dated June 25, 1982, the licensee described a design review, verification tests, changes in controls, additional instrumentation, controls and alarms, and new operating procedures for the purpose of:

- 1) reducing the probability of loss of NNI/ICS power
- 2) enabling operators to recognize loss of power

. . . .

3) providing guidance to operators for dealing with loss of power.

The staff has reviewed the licensee's responses submitted in L1L 310 and 5211-82-149 and concluded that these responses satisfactorily complete required licensee actions related to the ICS Failure Mode and Effects Analysis and that the Three Mile Island Nuclear Station, Unit 1 ICS design meets all current regulatory requirements. However, for the longer term Unresolved Safety Issue USI A-47, "Safety Implications of Control Systems," which was begun in December, 1980, 'has as its principle task, the assessment of the adequacy of current regulatory requirements for control systems. Resolution of A-47 will determine whether it will be necessary to impose additional and more stringent requirements on control systems in the future.

- 2 -