



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555-0001

GPU NUCLEAR CORPORATION

DOCKET NO. 50-320

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 2

POSSESSION ONLY LICENSE

Amendment No. 47
License No. DPR-73

- I. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by GPU Nuclear Corporation (the licensee) dated August 5, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the regulations of the Commission as set forth in 10 CFR Chapter I;
 - B. The facility will be maintained in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the regulations of the Commission;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the regulations of the Commission and all applicable requirements have been satisfied.

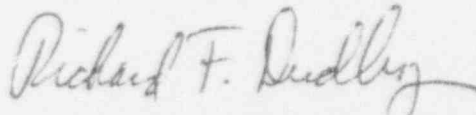
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(1) of Possession Only License No. DPR-73 is hereby amended to read as follows:

- (1) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 47, are hereby incorporated into this license. The licensee shall maintain the facility in accordance with the Technical Specifications and all Commission Orders issued subsequent to the date of the possession only license.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard F. Dudley, Acting Director
Non-Power Reactors and Decommissioning
Project Directorate
Division of Operating Reactor Support
Office of Nuclear Reactor Regulation

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: December 6, 1993

ENCLOSURE TO LICENSE AMENDMENT NO. 47

POSSESSION ONLY LICENSE NO. DPR-73

DOCKET NO 50-320

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove

1-5
1-6
3.9-3
3.9-4
B 3/4 9.1

Insert

1-5
1-6
3.9-3
3.9-4
B 3/4 9-1

1.0 DEFINITIONS

CORE ALTERATION

1.15 CORE ALTERATION shall be the movement or manipulation of any reactor component (including fuel) within the reactor pressure vessel with the vessel head removed and fuel in the vessel. Suspension of CORE ALTERATION shall not preclude completion of movement of a component to a safe conservative position.

LOSS-TO-AMBIENT

1.16 LOSS-TO-AMBIENT is a passive cooling mode by which decay heat, generated by the reactor core, is removed and transferred to the surrounding environment by air and passive components (i.e., Reactor Vessel) inside the Reactor Building.

ACCIDENT GENERATED WATER

1.17 Deleted

1.18 LICENSED OPERATOR (OL) - any individual who possesses an NRC Operator's license pursuant to Title 10, Code of Federal Regulations, Part 55, "Operators Licenses."

1.19 SENIOR LICENSED OPERATOR (SOL) - any individual who possesses an NRC Senior Operator's license pursuant to Title 10, Code of Federal Regulations, Part 55, "Operators Licenses."

1.20 FUEL HANDLING SENIOR LICENSED OPERATOR (SOL-FH) - an individual licensed by the Nuclear Regulatory Commission to supervise fuel handling and core alterations operations.

1.0 DEFINITIONS

1.21 CONTAINMENT ISOLATION shall exist when:

- a. Each penetration is:
 1. Closed by an accessible manual valve, a welded or bolted blind flange, or a deactivated automatic valve secured in the closed position to provide isolation of each penetration, or;
 2. Open per an approved procedure but can be closed pursuant to Specification 1.21.a.1. Controls shall be implemented to minimize the time the penetration is allowed open and to specify the conditions for which the penetration is open. Penetrations shall be expeditiously closed upon completion of the conditions specified in the approved procedures.
- b. The Equipment Hatch is closed and sealed.
- c. Each Containment Airlock is OPERABLE pursuant to Specification 3.6.1.6.

1.22 The OFFSITE DOSE CALCULATION MANUAL (ODCM) shall contain the methodology and parameters used in the calculation of offsite doses resulting from radioactive gases and liquid effluents, in the calculation of gaseous and liquid effluent monitoring Alarm/Trip Setpoints, and in the conduct of the Environmental Radiological Monitoring Program. The ODCM shall also contain (1) the Radioactive Effluent Controls and Radiological Environmental Monitoring Programs required by Section 6.8.4 and (2) descriptions of the information that should be included in the Annual Radiological Environmental Operating Report and the Annual Radioactive Effluent Release Report required by Specifications 6.9.1.1 and 6.9.1.2, respectively.

1.23 MEMBER(S) OF THE PUBLIC shall include all persons who are not occupationally associated with the plant. This category does not include employees of the GPU System, GPU contractors or vendors. Also excluded from this category are persons who enter the site to service equipment or to make deliveries.

1.24 UNRESTRICTED AREA shall be any area at or beyond the SITE BOUNDARY access which is not controlled by the licensee for purposes of protection of individuals from exposure to radiation and radioactive materials, or any area within the SITE BOUNDARY used for residential quarters or for industrial, commercial, institutional, and/or recreational purposes.

1.25 SITE BOUNDARY shall be that line beyond which the land is neither owned, nor leased, nor otherwise controlled by GPU Nuclear.

1.26 Deleted.

LIMITING CONDITIONS FOR OPERATION

AUXILIARY BUILDING AIR CLEANUP EXHAUST SYSTEM

3.9.12.2 The Auxiliary Building Air Cleanup Exhaust System shall be OPERABLE with one of the four system air cleanup exhaust fans OPERABLE.

APPLICABILITY: MODES 1, 2 and 3

ACTION:

With the Auxiliary Building Air Cleanup Exhaust System inoperable, restore the system to operable status within 4 hours or suspend all operations involving movement of liquid and solid radioactive wastes in the Auxiliary Building (other than sampling evolutions required by the Technical Specifications or RECOVERY OPERATIONS PLAN), the release of which could exceed 50% of the Appendix B Technical Specification instantaneous release rate for gaseous effluents, until the system is restored to OPERABLE status.

ACCIDENT GENERATED WATER

3.9.13 Deleted

TABLE 3.9-1

DELETED

3/4.9 RADIOACTIVE MATERIALS STORAGE

BASES

3/4.9.1 SPENT FUEL STORAGE POOL "A" WATER LEVEL MONITORING

Spent Fuel Storage Pool "A" Water Level Monitoring instrumentation has been provided to assure the capability to monitor water level in the Spent Fuel Storage Pool "A".

3/4.9.2 SPENT FUEL STORAGE POOL "A" WATER LEVEL

The water level in the Spent Fuel Storage Pool "A" has been established to limit the dose rate, due to the storage of Canisters, to acceptable levels.

3/4.9.3 FUEL TRANSFER CANAL (DEEP END) WATER LEVEL MONITORING

Fuel Transfer Canal Water Level Monitoring instrumentation has been provided to assure the capability to monitor water level in the deep end of the Fuel Transfer Canal.

3/4.9.4 FUEL TRANSFER CANAL (DEEP END) WATER LEVEL

The water level in the Fuel Transfer Canal (deep end) has been established to limit the dose rate, due to the storage of the plenum assembly and Canisters, to acceptable levels.

3/4.9.12 FUEL HANDLING BUILDING/AUXILIARY BUILDING AIR CLEANUP SYSTEM

The requirement for the Fuel Handling Building/Auxiliary Building Air Cleanup System to be operating or OPERABLE ensures that radioactive material released to these buildings will be filtered through the HEPA filters prior to release to the atmosphere. In the event the systems are not restored to OPERABLE status within 4 hours, the Technical Specifications require the suspension of any liquid or solid radioactive waste handling, the release of which could exceed 50% of the instantaneous release rate limits for gaseous effluents specified in Section 2.1.2 of the Appendix B Technical Specifications. These restrictions correspond to solid radioactive waste with a total activity of greater than 1 curie of particulates with half-lives greater than eight (8) days and liquid radioactive waste with an activity greater than 0.5 curies of particulates with half-lives greater than eight (8) days.

3/4.9.13 ACCIDENT GENERATED WATER

Deleted