

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

October 1, 1980

Mr. James A. Kay
Senior Engineer-Licensing
Yankee Atomic Electric Company
25 Research Drive
Westborough, Massachusetts 01581

Dear Mr. Kay:

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By letter dated November 16, 1979, we sent you the list of topics which would not be reviewed by your plant in the Systematic Evaluation Program (SEP). These topics were deleted from the SEP review because they were not applicable to your plant or they were being reviewed as part of an ongoing generic issue outside the SEP. The following topics should be added to the November 16 listing. Removal of these additional topics from the SEP review results from 1) related generic NRC activities and 2) non-applicability of topics to certain facilities. The following is a list of such topics and indicates their disposition:

Topic	Categorization	Disposition
II-2.8	Generic	Compliance with Appendix E is being evaluated under the Emergency Prepardness Program Office (EPPO) effort. Compliance with Appendix I is being evaluated as part of the NRC Appendix I review effort.
II-2.D	Generic	Compliance is being evaluated under the EPPO effort.
8.8-111	Deleted	This topic was originally intended for BWR's.
8-1V	Generic	Will be reviewed by NRC as part of TMI Task Action Plan (NUREG-0660) as specified in May 7, 1980 letter to all operating reactor licensees.
XV-11	Deleted	This item applies specifically to BWR's.
XV-18	Deleted	This item applies specifically to BWR's. The radiological consequences of steam line preaks for PWR's is part of Topic XV-2.

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The following two topics will not be reviewed by SEP in their entirety due to overlap with other NRC activities:

Topic	Disposition			
II-1.B	Those portions dealing with plant emergency plans will be evaluated as part of the EPPO effort.			
X	Those portions of the auxiliary systems being reviewed as part of Lessons Learned, the TMI Action Plan and IE Circular 80-04 will not be reviewed by SEP.			

In addition, the original DBE review list indicated that Group VI events, "Uncontrolled Rod Withdrawal at Power" and "Uncontrolled Rod Withdrawal - Low Power Startup" were part of Topic XV-13 which does not apply to PWR's. The actual topic reference should be Topic XV-8, "Control Rod Misoperation" which is applicable to PWR's. The new topic reference list for Topic XV-8 should include Standard Review Plans 15.4.1 and 15.4.2.

Enclosure 1 to this letter includes a revised listing of the DBE Accident and Transient Groups applicable to PWR's. Of note should be the addition of Group XII, "Steam Generator Tube Failure and the inclusion of Fuel Handling Accidents in Group VIII.

Sincerely,

Dennis M. Crutchfield, Cruer Operating Reactors Branch #5 Division of Licensing

Enclosure: As stated

cc w/enclosure: See next page cc w/enclosure: Mr. James E. Tribble, President Yankee Atomic Electric Company 25 Research Drive Westborough, Massachusetts 01581

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U. S. Environmental Protection Agency Region I Office ATTN: EIS COORDINATOR JFK Federal Building Boston, Massachusetts 02203

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PWR OBE TOPIC LIST

Accident and Transient Groups

	TOP1C	DESIGN BASIS EVENT	SRP
Group I (PWR)	XV-1 XV-1 XV-1 XV-1	Decrease in feedwater temperature Increase in feedwater flow Increase in steam flow Inadvertent opening of steam	15.1.1 15.1.2 15.1.3 15.1.4
	XV-9 XV-10	generator relief/safety valve Start-up of inactive loop System malfunction causing decrease in boron concentration	15.4.4 15.4.6
Group II	XV-3 XV-3 XV-3 XV-5 XV-6	Loss of external load Turbine trip Loss of condenser vacuum Steam pressure regulatory failure Loss of feedwater flow Feedwater system pipe break	15.2.1 15.2.2 15.2.3 15.2.5 15.2.7 15.2.8
Group III	XV-2 XV-2	Steam line break inside containment Steam line break outside containment	15.1.5 15.1.5
Group IV	XV-4 XV-24	Loss of AC power to station auxiliaries Loss of all AC power	15.2.6
Group V	XV-7 XV-7 XV-7	Loss of forced coolant flow Primary pump rotor seizure Primary pump shaft break	15.3.1 15.3.3 15.3.4
Group VI	XV-8 XV-8 XV-12	Uncontrolled rod assembly with- drawal at power Uncontrolled rod assembly with- drawal low power start-up Control rod misoperation Spectrum of rod ejection accidents	15.4.2 15.4.1 15.4.3 15.4.8
Group VII	XV-19	Spectrum of loss of coolant accidents	15.6.5
Group VIII	XV-21 XV-20	Orop of cask or heavy equipment Radiological Consequences of Damaging Accidents (inside and outside containment)	15.7.5 15.7.4
Group IX	XV-15	Inadvertent opening of PWR pressurizer relief valve	15.5.1
Group X	XV-14	Inadvertent operation of ECCS or CVCS malfunction that causes an increase in coolant inventory	15.5.1 15.5.2

Enclosure 6

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	RELATED TOPIC	DESIGN BASIS EVENT	SRP
Group XI		Not applicable to PWRs	
Group XII	xV-17	Steam Generator Tube Failure	15.6.3