



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA STREET, N.W.
ATLANTA, GEORGIA 30303

Report No. 50-261/82-25

Licensee: Carolina Power and Light Company
411 Fayetteville Street
Raleigh, NC 27602

Facility Name: H. B. Robinson

Docket No. 50-261

License No. DPR-23

Inspection at H. B. Robinson site near Hartsville, South Carolina

Inspector: R. H. Albright 7/20/82
R. H. Albright Date Signed

Approved by: K. P. Barr 7/20/82
K. P. Barr, Section Chief Date Signed
Technical Inspection Branch
Division of Engineering and Technical Programs

SUMMARY

Inspection on July 6-9, 1982

Areas Inspected

This routine, unannounced inspection involved 24 inspector-hours on site in the areas of radiation protection and transportation of radioactive materials.

Results

Of the two areas inspected, no violations or deviations were identified.

REPORT DETAILS

1. Persons Contacted

Licensee Employees

R. B. Starkey, Jr., Plant General Manager
*C. W. Crawford, Manager, Operations and Maintenance
*J. Young, Director, QA/QC
*W. L. MacCready, Radiation Control Supervisor
*J. A. Eaddy, Supervisor, Chemistry and Environment
F. Gilman, Senior Specialist, Regulatory Compliance
*C. L. Wright, Specialist, Regulatory Compliance
R. Denny, RC Foreman
W. T. Ritchie, RC Foreman
J. Petitigout, ALARA Specialist
M. Layton, Project Specialist Environmental and Chemistry

NRC Resident Inspector

S. Weise

*Attended exit interview

2. Exit Interview

The inspection scope and findings were summarized on June 9, 1982, with those persons indicated in paragraph 1 above.

3. Licensee Action on Previous Inspection Findings

Closed (82-14-02) (URI) This item concerned the potential overexposure of a contract worker. The licensee became aware of a potential personnel overexposure when an individual's TLD was read as a prerequisite to issuing multiple whole body dosimetry. The TLD reading indicated an exposure of 6.63 rem.

The investigating committee concluded that while the TLD reading could not be discredited, there was also no evidence to indicate the individual had received the 6.63 rem exposure. The committee recommended assigning the individual 265 mrem from his pocket dosimeter totals since this exposure compared well with the exposure estimated from his work at the plant.

The inspector followed the investigation closely and reviewed the final report. The inspector concluded that the investigation appeared to be thorough and the final committee recommendation agreed with available data. The inspector had no further questions.

4. Unresolved Items

Unresolved items were not identified during this inspection.

5. Licensee Action on Previous Inspector Followup Items

(Closed) (81-07-13) (IFI) Whole body count trend analysis. The inspector reviewed a whole body count trend analysis report for the years 1977-1980. The type of analysis performed appears adequate to evaluate the respiratory protection program effectiveness.

6. Health Physics Procedures

The inspector reviewed revision 15 to the Radiation Control Protection Manual. This review did not identify any violations or items of non-compliance. The inspector noted that the criteria used by the licensee to release material for unrestricted use is less restrictive than the guidelines suggested in IE Circular 81-07. The release criteria in question is that materials may be released after passing a smear test as long as direct radiation readings are less than 0.25 mr/hr. The IE Circular does not indicate a direct radiation dose rate limit but it does suggest a fixed contamination limit of 5000 dpm/100 cm². The inspector stated that the direct radiation limit used by the licensee appeared to be high and should be evaluated (82-25-01).

7. Radioactive Waste Shipment

The inspector observed radwaste shipment number D60-82. Radioactive placards were visible on each side of the transport vehicle. The inspector performed an independent radiation survey of the exterior of the shipping vehicle and in the cab and found radiation levels to be within DOT requirements.

Radioactive material shipping records for shipment numbers D59-82, D60-82, and C6-82 were reviewed and found to be in compliance with DOT requirements. The inspector had no further questions.

8. Respiratory Protection

The inspector selectively reviewed RWPs that required personnel to wear respiratory protection. The dosimetry records of selected individuals who were issued respirators to perform the work described on the RWPs were reviewed. These records contained the required medical evaluations for personnel who wear respirators. The inspector had no further questions.

9. Housekeeping Inside Containment

The inspector reviewed startup procedure GP-2 which contained inspection requirements for startup. This procedure requires the reactor vessel sump, RHR sump and the reactor cavity to be inspected and free of debris. In addition, the inspection ensures that loose insulation has been removed from

containment and all unprotected insulation has been secured. A licensee representative stated that substantial effort has been put into cleaning up work areas as work is completed so that the final cleanup is less of a problem. The inspector had no further questions.

10. Health Physics Instruments

The inspector checked calibration stickers, performed response checks and battery checks on several instruments that were available for use. Instruments which needed repair were marked as out of service to ensure that they would not be used. Several friskers located in the plant were checked for proper alarm set point and calibration stickers. No violations or deviations were identified.

11. Tour of Radiation Control Zone

The inspector toured the radiation control zone with a licensee representative to observe posting practices for radiological barriers. During the tour, the inspector found a door to the cask decon area open. A step off pad for the contaminated area was located outside the door; however, the posting for the area was on the outside of the door and could not be seen at the entrance to the area. The licensee representative on tour with the inspector closed the door. The licensee later initiated a request to have a closure device installed on the door.

Also during the tour, the inspector found one side of a radiation area barrier that had fallen down. The highest radiation reading inside the areas and at eighteen inches from the warehouse exterior wall was less than 5 mR/hr. When a licensee representative tried to repost the area to its previous configuration, he found that someone had apparently removed approximately fifteen feet of barrier rope. The inspector stated that licensee control of barriers will be re-examined during future inspections (82-25-02).