BALTIMORE GAS AND ELECTRIC COMPANY

P. O. BOX 1475

BALTIMORE MARYLAND 21203 1980 NOV 7 PM 3 31

ARTHUR E. LUNDVALL, JR. VICE PRESIDENT SUPPLY November NBC 1000 UNIVERVICES URANCH

Director Begion I, U. S. Nuclear Regulatory Commission Office of Inspection and Enforcement 631 Park Avenue King of Prussia, PA 19406

> Subject: Calvert Cliffs Nuclear Power Flant Units Nos. 1 & P. Dockets Nos. 50-317 & 50-318 Report of Changes, Tests and Experiments

Gentlemen:

As required by 10 CPP Part 50 Paragraph 50.59, attached is a report of all changes, tests and experiments completed on Calvert Cliffs Units 1 and/or 2 under the provisions of that Part and covering the period from our last such report through December 31, 1979.

Items in the attached are referred to by "Facility Change Request (FCR)" number.

2. Sundwall in

cc: Director of Inspection and Enforcement (39 copies) U. S. Nuclear Regulatory Commission Washington, D. C. 20555

J. A. Biddison, Esquire G. F. Trowbridge, Esquire

8011400

THIS DOCUMENT CONTAINS POOR QUALITY PAGES

045

Page 1 of 3

Changes, Tests and Experiments Made in Accordance With 10 CFE 50.59 for Calvert Cliffs Units 1 and/or 2

- 74-65 This FCB added a scaffold hoist on the polar crane which is Seismic Class I designed but not required to operate through/after an earchquake or LOCA. The hoist is seismically restrained and is not to be raised if failure could impact the NSSS. Therefore, there is no unreviewed safety question.
- 75-1 The only portion of this FCR which was performed was to revise the pipe classification number for various pipe lines. No physical change was required, and thus, there was no unreviewed safety question.
- 75-9 This FCR provides for lockable wire case doors. Anchor bolt drilling was done in accordance with SR procedures and, therefore, not an unreviewed safety question.
- T5-113 This FCR provides for a new ladder for access to previously inaccessible valves, etc. The ladder and anchor bolt installation are designed to previous SR standards and do not represent an unreviewed safety question.
- 75-121 This FCR provides for lockable wire cage doors. Anchor bolt drilling was done in accordance with SR procedures and, therefore, not an unreviewed safety question.
- 75-164 This FCR called for changing the fan motor and fan wheel pulleys to double groove pulleys and adding a second drive belt to each of the switchgear room air handling units. The safety analysis concluded that no unreviewed safety questions exist since reliability of the fan drives will be increased with the addition to the second drive belt.
- 75-264 This FCR modified support to PS 403 to allow removal of de-surger. Modification did not effect the pressure boundary or stress analysis of piping, but only supports.
- 75-1001 This FCE changed the interval for surveillance of the containment nost tensioning system from 1, 2, and 3 years to 1, 3 and 5 years. Since there is no physical change to the facility and surveillance neriod is considered adequate per Req. Guide 1.35, there is no unreviewed safety question.
- 75-1048 This FCR provides personnel safe walking baths over screen wash area at the intake structure, stiles were installed for safety. Concrete drilling and anchor installation was done in accordance with SR procedures and does not constitute an unreviewed safety question.

- 75-1058 This FCR added backseats to the letdown containment isolation valves pursuant to reducing packing leaks. The safety analyses concluded that this FCR did not consititute an unreviewed safety question because the change would not affect the valves' safetyrelated functioning as the original design criteria for the valve. were being maintained.
- 75-1070 This FCR, which applied to Unit 2 only, relocated the reactor vessel vent values to a location adjacent to the refueling pool wall for easier access. As the function of the values and the vent line was not altered, it was declared that no unreviewed safety question exists concerning this modification.
- 75-1133 This FCR installed an orifice in a containment spray drain line between the containment spray header and the last valve to prevent valve leakage from reaching the header. The orifice was sized such that at full containment spray flow, the flow directed through the orifice was not significant, and therefore, there was no unreviewed safety question.
- 76-47 Although the new card reader doors were installed NSP, the concrete anchors are SR and were installed using SP procedures. Therefore, there is no unreviewed safety guestion.
- 76-153 This FCR added a fenced area to provide controlled storage for tools, parts, material, equipment, etc. for use in Unit 1 containment. A similar fenced area already exists in Unit 2. There is no unreviewed safety question.
- 76-165 This FCR revised drawings of valves to reflect changes in dimensions and material by the vendor. This did not constitute an unreviewed safety question since the function of the valves were not altered.
- 76-171 This FCR replaced the 120 V parking bus and collector shoes of the Polar Crane with an extension cord. The polar crane is classified safety related only due to structural considerations. The weight of the 6' to 8' extension cord is insignificant. In addition, if it were to fall, it would cause no damage.
- 76-210 "This FCR changed sketches and interchanged snubbers to bring installation and as-built drawings into agreement with Technical Specification tables. Changes were administrative only, no design change was involved, and therefore no unreviewed safety duestions were involved.
- 76-1040 This FCR modified data in FSAR high energy analysis to reflect higher flow through the excess flow check valves than had previously been analyzed. The analysis showed that potential accident consequences are not increased over those previously licensed.

- This FCR is to add flanges to the piping on both sides of RV-417 to facilitate removal of the valve. RV-417 is the relief valve on the auxiliary MPSI header. The safety analysis concluded that since the function of the valve and the system did not change, that the stresses in the system were not affected and that no unreviewed safety question existed.
- 78-40 This FCR provides for the deletion of the auxiliary feedwater pump suction pressure transmitters which had been repeatedly damaged by pressure cransients on pump trips. Transmitters are safety-related for pressure retention only. Their non-safety-related function is not input to any safety-systems or regulating loops, but is mentioned in the FSAP. The low suction pressure alarm function, also non-safety-related, is served by a separate pressure switch.
- 76-115 This FCR is a drawing change only. Equipment was already reviewed when vendor drawing was approved. These turbo blowers mounted on the dresels were not previously shown in the schematics.
- 78-1020 This FCR removed the requirement that the female face of a tongue and proove flange be installed on the valves and similar equipment for the reactor coolant and other systems seeing full primary pressure. The safety analysis concluded that there was no engineering justification for this requirement and there was no change in the structural integrity of the flanged joint by this removal, hence no unreviewed safety question was involved.
- 79-126

77-146

This FCR replaced damaged containment spray pump motor with one of a different type. Direct replacement not available. This change was not an unreviewed safety question because the new motor met the design criteria of the original installation.