ORGANIZATION: GENERAL ELECTRIC COMPANY
NUCLEAR ENERGY BUSINESS OPERATIONS
SAN JOSE, CALIFORNIA

REPORT NO: 99900403/82-02	INSPECTION DATE(S) 6/7-11/82	INSPECTION ON-SITE HOURS: 35

CORRESPONDENCE ADDRESS: General Electric Company
Nuclear Energy Business Operations
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175 Curtner Avenue
San Jose, California 95125

ORGANIZATION CONTACT: Mr. A. Breed, Manager, Quality Assurance
TELEPHONE NUMBER: (408) 925-2726

PRINCIPAL PRODUCT: Nuclear Steam System Supplier

NUCLEAR INDUSTRY ACTIVITY: General Electric Company, Nuclear Energy Business Operations (NEBO), has a work force of approximately 7650 people with approximately 98% of that work force devoted to domestic nuclear activity. NEBO currently has 26 reactor units under construction and 4 reactor units under contract. NEBO also has approximately 115 service contracts with various clients.

ASSIGNED INSPECTOR: Dwight D. Chamberlain 7-23-82
D. D. Chamberlain, Reactor Systems Section (RSS) Date

OTHER INSPECTOR(S): P. H. Harrell, RSS

APPROVED BY: C. J. Hale 7/23/82
C. J. Hale, Chief, RSS Date

INSPECTION BASES AND SCOPE:

A. BASES: General Electric Topical Report No. NEDO-11209-04A and 10 CFR Part 50, Appendix B.

B. SCOPE: Status of previous inspection findings and follow up on the following items: (Cont. next page.)

PLANT SITE APPLICABILITY: Docket Numbers 50-522/523, 50-278, 50-333, 50-373/374 50-410, 50-416/417, 50-440/441, 50-458/459, 50-461, 50-466/467, 50-518 and 50-556/557.

DESIGNATED ORIGINAL
Certified By Eleanor Foutz

ORGANIZATION: GENERAL ELECTRIC COMPANY
NUCLEAR ENERGY BUSINESS OPERATIONS
SAN JOSE, CALIFORNIA

REPORT NO: 99900403/82-02	INSPECTION RESULTS:	PAGE 2 of 8

B. SCOPE: (Cont.)

1. 10 CFR Part 50.55(e) report (River Bend project) stating that the heat loads provided by GE to Stone & Webster (S&W) for sizing the heating and ventilating (HVAC) system in the high pressure coolant injection (HPCI) system diesel generator room were about 1/3 of the expected value.

2. Mississippi Power & Light (Grand Gulf project) reported that the fuel pool cooling and cleanup heat exchanger relief valve settings were 10% above design pressure.

3. 10 CFR Part 21 report (Grand Gulf project) stating that reactor pressure vessel water level transmitters and trip units could not be adjusted to the specified setting.

4. 10 CFR Part 21 report (Hartsville project) stating that the orifice diameter for the Residual Heat Removal discharge line was incorrectly specified as 0.63" rather than the required 6.31".

5. GE notification to operating BWR plants that the HPCI or RCIC high steam flow trip differential pressure setting must be determined by actual test results.

6. Report of GE error in computing four bundle power allocation factors (Peach Bottom project).

ORGANIZATION: GENERAL ELECTRIC COMPANY
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SAN JOSE, CALIFORNIA

REPORT NO: 99900403/82-02	INSPECTION RESULTS:	PAGE 3 of 8

A. VIOLATIONS:

None

B. NONCONFORMANCES:

1. Contrary to Section 5 of Topical Report No. NEDO-11209-04A and Sections 1.1 and 4.3.b.5 of EOP 45-4.00, a supplier submitted drawing for the Fuel Pool Heat Exchangers (Grand Gulf project) was reviewed and approved by GE even though the setpoints for the heat exchanger relief valves were specified to be set at a pressure 10% above the design pressure. The purchase order that included these valves invokes ASME code, Section III, 1974 Edition, Winter Addenda requirements which state that the set pressure of pressure relief devices shall not be greater than the design pressure of the protected system.

2. Contrary to Section 5 of Topical Report No. NEDO-11209-04A and Appendix B of NEBGP 70-42, a GE 10 CFR Part 21 report relating to a defect in selection of instrument range for reactor vessel water level transmitters and trip units submitted on August 19, 1981, did not include all required information.

C. UNRESOLVED ITEMS:

None

D. STATUS OF PREVIOUS INSPECTION FINDINGS:

1. (Closed) Nonconformance (82-01): Requested additional information for a potentially reportable condition was not provided within 5 working days.

   GE stated that the additional information was transmitted orally and the required action is now complete. A memo was issued to emphasize the need for the Safety Evaluation Programs unit to continue to expedite the responses within the specified time frame and to document the communications in the 10 CFR Part 21 record files.

2. (Closed) Nonconformance (82-01): No Engineering Review Memo (ERM) existed to document the engineering review of operating and maintenance manual GEK 75662A for the High Pressure Core Spray System (Clinton Project).

The missing ERM (preliminary manual issue) was located after the inspection and to preclude misplacing ERM's in the future, the ERM will be processed through the document release and control function at the time of manual issue and a permanent microfilm record will be made and retained. A thorough review was made by GE of all operating and maintenance manual-related ERM's since implementation of the ERM practice. All required ERM's have been located.

E. OTHER FINDINGS OR COMMENTS:

1. HVAC Heat Load Sizing for the High Pressure Coolant System (River Bend) - This area of inspection resulted from a 10 CFR Part 50.55(e) report from the River Bend project stating that the heat loads provided to S&W by GE for the sizing of HVAC in the HPCS diesel generator room were about 1/3 the actual values. An S&W engineer apparently questioned the values of heat radiation provided. One supplier (Morrison Knudsen) of HPCS diesel generators to GE has performed tests on one unit for verification of actual heat radiation values. GE is presently evaluating the results of these tests and it appears that the test values are higher (2-3 times) than the originally stated values. This item will remain open at GE for completion of the following:

   a. GE to obtain revised heat load values.

   b. GE to notify affected projects.

   c. GE to revise affected documents.

   GE stated that the following sites could be affected: La Salle, Nine Mile Point 2; Perry; Clinton; River Bend; Hanford; Grand Gulf; Allens Creek; TVA (all); Skagit; and Black Fox.

2. Incorrect Relief Valve Settings (Grand Gulf project) - The Grand Gulf licensee (Mississippi Power and Light) reported that incorrect relief valve pressure settings for the Fuel Pool Heat Exchangers had been provided by GE on a drawing. The relief valves should be set at system design pressure (250 psig for the shell side and 150 psig for the tube side) as required by the ASME Code, Section III, 1974 Edition. The values supplied on the drawing stated the setpoints to be 275 psig for the shell side and 165 psig for the tube side (10% above the system design pressure). The purpose of this inspection at GE was to determine if a design verification was performed and who provided the incorrect set point values.

After a review of GE documents and discussions with GE personnel, the NRC inspector found the following:

a. The relief valves were supplied by a supplier to GE (Atlas Industrial Manufacturing Co.) and the setpoints were specified on the supplier drawing. This drawing was part of the supplier information provided with the heat exchanger.

b. The responsible GE engineer reviewed and approved the vendor drawing that specified the incorrect relief valves settings.

Based on the above, a nonconformance (B.1 above) was identified.

Prior to completion of this inspection, GE reviewed all other heat exchanger orders and reported by memo (dated June 10, 1982) that all other heat exchanger orders had been reviewed and no other cases of this type were identified. Also, discussions had been held with engineering personnel to assure the requirements of the ASME code for relief valve setpoir clearly understood. GE stated that the licensee is co ..g the problem at Grand Gulf. Since corrective and preventive actions have been taken by GE, no further written response to the nonconformance is necessary. The NRC inspector will make a spot check of other heat exchanger orders during a future inspection.

3. Incorrect Range for Water Level Transmitters (Grand Gulf project) - This area of inspection resulted from a 10 CFR Part 21 report regarding incorrect range of reactor pressure vessel (RPV) water level transmitters and trip units. This item was initially identified by GE on the Grand Gulf project. The problem was evaluated by GE and determined to be reportable under 10 CFR Part 21 since the shipped transmitters and trip units could not fulfill their intended safety function to provide RPV Level 3 and Level 8 scram signals. The affected plants include: Clinton; Perry; TVA (Hartsville); River Bend; and Grand Gulf. GE has issued Field Disposition Instructions to the affected projects for replacement of the transmitters and/or trip units. GE had initiated a program in late 1980 for a general upgrade of design specifications to include more complete information for instrument ranges, set points, accuracy, etc., and GE stated that the completion of this program should prevent recurrence of this type of problem. It appears that the initial problem was due to the design imput document (design specifications) being incomplete for proper instrument selection.

The review of the general upgrade program in process by GE indicates that the upgraded design specifications will contain the required information for proper instrument selection and for design verification. Therefore, this item is considered closed at GE.

During the review of the 10 CFR Part 21 report issued by GE on this problem, it was noted that the report did not contain all required information. A nonconformance (B.2 above) was issued relative to 10 CFR Part 21 report contents. After a telephone conference with the Region IV inspector, GE and the Events Analysis Branch of the NRC's Office of Inspection and Enforcement, it was determined that the following two commitments from GE were required (GE memo CAC 56-82, dated June 11, 1982):

a. All future GE reports will use the 10 CFR Part 21 list of requirements as the report format in order to clearly outline the information required. The information submitted will be complete, or a commitment will be made in the report to supply missing information in a followup report.

b. GE will continue to supply to the NRC offices any information they request for clarification of GE 10 CFR Part 21 reports.

Based on the above commitments, no additional written response to the nonconformance will be necessary.

4. Incorrect RHR Orifice Size (Hartsville) - A 10 CFR Part 21 report, dated March 2, 1982, filed by the Nuclear Power Systems division of GE stated that an error was found in a fabrication document released by GE to the utility (TVA). The orifice diameter for the RHR discharge line was stated as 0.63", instead of the required 0.31" diameter. The report also stated the error was discovered and corrected before the hardware was fabricated and installed in the plant (Hartsville).

A review of the specification confirmed that this particular error has been corrected. However, the supporting design documentation and calculations were not reviewed because this work was performed by a GE subcontractor (C. F. Braun) and the records were not available at GE (San Jose). A future inspection will be made at C. F. Braun or at GE (when applicable documents become available at GE) to determine the reason for the error. This item will remain open pending the results of future inspections.

5. Nonconservative Calculation of Differential Pressure Settings (Fitzpatrick) - The licensee (Power Authority of the State of New York) reported finding incorrect high steam flow differential pressure settings on the high steam flow isolation for the high pressure coolant injection (HPCI) system. It was also noted that a similar feature exists in the reactor core isolation cooling (RCIC) system. The purpose of this inspection was to determine the method used to establish the high steam flow trip settings and to verify that GE has supplied adequate information regarding the high steam flow differential set point requirements.

   A review of the design specification for Fitzpatrick indicated that a calculated value for the setting had been supplied by GE. However, it was noted in GE-provided start up instructions that the actual number to be used in specifying the setting was to be determined by test measurements performed during plant start up. This determination is required because the value will vary from plant to plant depending on the actual physical configuration of the flow instrument connections. The method for obtaining test measurements is provided by GE in the test start up instructions. GE stated the reason the calculation is made is to determine the required range for purchase of the flow instrument.

   The inspector determined that all affected plants had been notified of the problem by GE. In addition, the NRC has issued an Information Notice (No. 82-16) to all facilities holding an operating or construction license. The inspector also performed a spot check of design specifications for other plants. During the review, the inspector found some specifications did not contain a note stating the value provided was a calculated setting. To provide additional clarity, GE agreed to review all design specifications and include the applicable note (if missing).

   During a future inspection, a spot check of design specifications will be made to ensure the note has been included.

6. Errors in Four Bundle Power Allocation Factors (Peach Bottom) - This area of inspection resulted from a Region I request for follow up on reported errors in GE-provided computer data for licensee core power calculations. GE performs computer calculations for operating plants during each refueling in order to provide coefficients for the licensee to perform process computer calculations for core power allocations. The review of this area at GE revealed that the problem was caused by a computer input format error. The GE design review/check did not identify the error.

GE has evaluated this condition for all operating plants and although other errors were identified, an evaluation revealed that no operational problems exist. Also, GE has evaluated the calculation process and a decision has been made to develop an automated input program to preclude future problems of this nature. This item will remain open pending an indepth evaluation of the GE calculation process during a future NRC inspection.

Inspector D. CHAMBERLAIN

Scope/Module PREVIOUS INSPECTION FINDINGS

Docket No. 7100490
Report No. 82-02
Page 1 of 1

DOCUMENTS EXAMINED

1	2	TITLE/SUBJECT	3	4
1	6	MEMO - ERMs AND MANUAL ISSUE	3-1-82	
2	8	ERM NO. DMM-0542	3-31-82	
3	3	TECHNICAL PUBLICATIONS OPERATIONS GUIDE (OG) XIII ERM PROCEDURES	6/82	REV. 0

Document Types:
1. Drawing
2. Specification
3. Procedure
4. QA Manual
5. Purchas Order
6. Internal Memo
7. Letter
8. Other (Specify-if necessary)

Columns:
1. Sequential Item Number
2. Type of Document
3. Date of Document
4. Revision (If applicable)

Inspector D. CHAMBERLAIN

Scope/Module PREVIOUS INSPECTION FINDINGS

Docket No. 99900403
Report No. 82.02
Page 1 of 1

DOCUMENTS EXAMINED

1	2	TITLE/SUBJECT	3	4
1	6	MEMO - ERMs AND MANUAL ISSUE	3-1-82	
2	8	ERM NO. DMM-0542	3-31-82	
3	3	TECHNICAL PUBLICATIONS OPERATIONS GUIDE (OG) XIII ERM PROCEDURES	6/82	REV. 0

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Inspector D. CHAMBERLAIN

Scope/Module ACTION ITEM #1

Docket No. 99900403
Report No. 82-02
Page 1 of 1

DOCUMENTS EXAMINED

1	2	TITLE/SUBJECT	3	4
1	8	PRC 81-43 HPCS DIESEL GENERATOR HEAT LOAD	BEGAN 11/81	
2	8	406C-0-0150 LETTER FROM MORRISON-KNUDSEN POWER SYSTEMS DIVISION TO GENERAL ELECTRIC ENGINE ROOM VENTILATION	6/7/82	
3	8	HEAT RUN TEST # HL182	5/5/82	
4	8	W.O. 74497 2600 KW GENERATOR SET CUSTOMER P.O. AG897 (RIVER BEND) COMMERCIAL INSTRUCTION AND PARTS MANUAL (STEWART & STEVENSON)	4/27/81	
5	2	21A9236 ENGINE-GENERATOR FOR HPCS	7/7/75	REV. 4
6	5	RIVERBEND 1&2 DIESEL GENERATOR HPCS 205-AG897	10/29/76	

Document Types:
1. Drawing
2. Specification
3. Procedure
4. QA Manual
5. Purchas Order
6. Internal Memo
7. Letter
8. Other (Specify-if necessary)

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3. Date of Document
4. Revision (If applicable)

Inspector D. CHAMBERLAIN

Scope/Module ACTION ITEM #2

Docket No. 99900403
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DOCUMENTS EXAMINED

1	2	TITLE/SUBJECT	3	4
1	2	21A9520 PURCHASE SPECIFICATION HEAT EXCHANGER, FUEL POOL	6-12-78	REV. 5
2	1	26-252 FUEL POOL HEAT EXCHANGERS D-4532-6	10-15-76	REV. 6
3	2	DC21A9520AB DESIGN CERTIFICATION FUEL POOL COOLING	6-13-78	REV. 4
4	3	EOP 45-4.00 ENGINEERED EQUIPMENT SUPPLIER DOCUMENT REVIEW	10-2-81	
5	1	P&ID 795E805 FUEL POOL COOLING & CLEANUP SYS	9-28-81	REV. 1

Document Types:
1. Drawing
2. Specification
3. Procedure
4. QA Manual
5. Purchas Order
6. Internal Memo
7. Letter
8. Other (Specify-if necessary)

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Inspector D. CHAMBERLAIN

Scope/Module ACTION ITEM #3

Docket No. 99900403
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DOCUMENTS EXAMINED

1	2	TITLE/SUBJECT	3 REPORTED	4
1	8	PRC 81-23 XMITTERS AND TRIP UNITS FOR REACTOR VESSEL WATER LEVEL SCRAMS	8-19-81	
2	2	22A3771AE DESIGN SPEC DATA SHEET (GRAND GULF 1&2) C71-4010	1-25-82	REV. 0
3	8	ERM NO. AMD-2270 REVIEW OF DEVICE LIST (DL) 828E531BA	7-25-79	
4	1	169C8392 TRANSMITTER DIFFERENTIAL PRESSURE	12-1-81	REV, 8
5	8	DL807E166TY SYSTEM DEVICE LIST (NMP 2) C72-1050	12-13-77	REV. 0
6	8	ERM NO. AMD-1667 FOR DL807E166TY	11-21-77	
7	8	DL828E531CA SYSTEM DEVICE LIST (PERRY)	5-2-80	REV. 5
8	2	22A2887AL NUCLEAR BOILER SYSTEM DESIGN SPEC (PERRY)	8-4-80	REV. 6
9	1	761E445 P&ID DATA NUCLEAR BOILER	6-18-81	REV. 11
10	1	732E103AF P&ID NUCLEAR BOILER SYSTEM (NMP 2)	10-5-78	REV. 3
11	1	761E445AF P&ID DATA NUCLEAR BOILER (NMP 2)	10-3-78	REV, 2
12	8	ERM NO. AMD-872 FOR DL828E531CA (PERRY	5-13-76	
13	8	DL828E226CA AUTOMATIC DEPRESSURIZATION DEVICE LIST (PERRY)	4-28-81	REV. 6

Document Types:
1. Drawing
2. Specification
3. Procedure
4. QA Manual
5. Purchas Order
6. Internal Memo
7. Letter
8. Other (Specify-if necessary)

Columns:
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3. Date of Document
4. Revision (If applicable)

Inspector D. CHAMBERLAIN

Scope/Module ACTION ITEM 3

Docket No. 99900403
Report No. 82-02
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DOCUMENTS EXAMINED

1	2	TITLE/SUBJECT	3	4
14	8	DL828E534CA RHR SYSTEM DEVICE LIST (PERRY)	4-28-81	REV. 5
15	8	DL828E536CA HIGH PRESSURE CORE SPRAY (PERRY)	4-1-81	REV. 5
16	1	769E305CA P&ID NUCLEAR BOILER SYSTEM (PERRY)	10-12-81	
17	8	DL828E445CA NUCLEAR STEAM SUPPLY SYSTEM (PERRY)	6-30-81	REV. 6
18	8	DL828E535CA LOW PRESSURE CORE SPRAY (PERRY)	1-28-82	REV. 9

Document Types:
1. Drawing
2. Specification
3. Procedure
4. QA Manual
5. Purchas Order
6. Internal Memo
7. Letter
8. Other (Specify-if necessary)

Columns:
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Inspector D. CHAMBERLAIN

Scope/Module ACTION ITEM #4

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Report No. 82-02
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DOCUMENTS EXAMINED

1	2	TITLE/SUBJECT	3	4
			CLOSE	
1	8	PRC 81-12 DISCREPANCIES IN STRIDE DOCUMENTATION	4/13/82	
		VERSUS G.E. SPECIFICATION - RHR DISCHARGE ORIFICE		
2	4	NEDO-11209 GE TOPICAL REPORT	10-1-80	

Document Types:
1. Drawing
2. Specification
3. Procedure
4. QA Manual
5. Purchas Order
6. Internal Memo
7. Letter
8. Other (Specify-if necessary)

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3. Date of Document
4. Revision (If applicable)

Inspector D. CHAMBERLAIN

Scope/Module ACTION ITEM #5

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Report No. 82-02
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DOCUMENTS EXAMINED

1	2	TITLE/SUBJECT	3	4
1	8 2	22A1354AY REACTOR CORE ISOLATION COOLING SYSTEM DESIGN SPECIFICATION DATA SHEET (SUSQUEHANNA)	2-26-82	REV. 7
2	2	22A1354AW REACTOR CORE ISOLATION COOLING SYSTEM DESIGN SPECIFICATION DATA SHEET (SHOREHAM)	2-26-82	REV. 6
3	8	JAFNPP STARTUP TEST RESULTS STR. NO: 14-2 RCIC VESSEL INJECTION	3-3-75	
4	8	JAFNPP STARTUP TEST RESULTS STR NO: 15-2 HPCI INJECTION TO VESSEL	5-22-75	
5	2	22A6082 START UP TEST SPECIFICATION	2-4-82	REV. 0
6	8	STARTUP TEST INSTRUCTION - RCIC SYSTEM STI-14	1-8-80	
7	8	22A2521AV (FITZPATRICK) - STARTUP TEST INSTRUCTIONS	4-30-73	
8	8	TELEPHONE COMMUNICATION- DOMESTIC CUSTOMER SERVICE	2-26-82	
9	2	22A3735AA DESIGN SPECIFICATION DATA SHEET LEAK DETECTION SYSTEM (GRAND GULF)	2-4-82	REV. 0

Document Types:
1. Drawing
2. Specification
3. Procedure
4. QA Manual
5. Purchas Order
6. Internal Memo
7. Letter
8. Other (Specify-if necessary)

Columns:
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3. Date of Document
4. Revision (If applicable)

Inspector D. CHAMBERLAIN

Scope/Module ACTION ITEM #6

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Report No. 82-02
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DOCUMENTS EXAMINED

1	2	TITLE/SUBJECT	3	4
1	6	INTERNAL MEMO FOR PROCESS COMPUTER NUCLEAR CORRELATION DATA PROCEDURES	1-11-82	
2	6	LETTER TO OCD&M-2 ENGINEERS LB26282001018	2-18-82	
3	6	LETTER WRAPUP TAPES AND ID's FOR PROCESS COMPUTER CORRELATIONS VERIFICATION LB262820015	1-14-82	
4	6	LB 264-82-01-013 VERIFICATION OF INPUT TO GPCUS FOR PEACHBOTTOM 2/3 AND MONTICELLO	1-20-82	
5	6	LB 264-82-02-030 RESULTS OF PROCESS COMPUTER QUALIFICATION CHECK ON OPERATING PLANTS	2-16-82	
6	8	DESIGN VERIFICATION FOR DOCUMENT NO. 459HA825 PEACHBOTTOM-3 DRF NO. L11-00195	4-23-81	
7	8	VERIFICATION OF NEW RUN - PEACHBOTTOM-3 CYCLE 5	12-22-81	

Document Types:
1. Drawing
2. Specification
3. Procedure
4. QA Manual
5. Purchas Order
6. Internal Memo
7. Letter
8. Other (Specify-if necessary)

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4. Revision (If applicable)

PERSONS CONTACTED

Company GENERAL ELECTRIC

Dates 6/7-11/82

Docket/Report No. 99900403/82-02

Inspector D. CHAMBERLAIN

NAME(Please Print)	TITLE(Please Print)	ORGANIZATION(Please Print)
CW DILLMANN	MGR - FLOW CONTROL VALVE, HEAT EX., & PUMP DES	GE-NPSED
BJ BEACH	Sr ENGR - QC	NESO-GE
ED GIAMBALVO	Sr ENG. - Licensing - SLO	S&LO. GE 53525
J.S. MOKRI	MGR - EQUIP EVALUATION	NPSED
M.G. MUNSON	PRINCIPAL ENGR - PIT MECH. SYS	NPS ED. x51924
I KLEPPER	C&I - ENG - SEN ENG.	
R.K. WALDMAN	C&I ENG. AUDIT COORD	PDE - NC&ID
N E BARCLAY	AUDIT COORD.	GE NC&ID
L J MACY	LICENSING ENGR	S&LO
R.A. CICCARELLI	RIVER BEND PROJECT ENGR	GE Dom. Proj's.
W.H. Hendrix	C&I Electrical System Eng	NC&ID
R.A. SIEMER	LEAD SYSTEM ENG, NUCL REACTOR PROTECTION SYSTEM	NPSED
C.L. Buckner	Spec. Qual. Sys.	NEPO/QAEE&I
J M MURRAY	MGR Proc Support	NEPO
RF PARIANI	SENIOR BUYER	NEPO
D.W. REIGEL	MGR - SYSTEMS ENGR	NC&ID
R.J. VALENCIA	AUDIT COORDINATOR	NEO
J.L. Murray	Mgr - QA	NRED/NED
L.S. BOHL	Asst Mgr - Nuc. Rel. Engr. Op.	NRED/NED
M A Ross	MANAGER - DATA ACQUISITION & OPERATOR SYST.	NPSED
C.A. Cameron	Manager - Safety Evaluation Programs	Safety & Licensing
J. M. CASE	ACTING MGR, PRODUCT ASSURANCE	NE P&QAO
T T Fox	Principal Engr Nuclear Chemistry	NE P&QAO
M.W. SHERWOOD	LEAD SYSTEM ENGINEER	GE ENGINEERING
W. H. BROWN	Senior Program Manager	Nuclear Services Div (GE)
C. CHRISTENSON	LSE	G.E.

PERSONS CONTACTED

Company GENERAL ELECTRIC

Dates 6/7-11/82

Docket/Report No. 99900403/82-02

Inspector D. CHAMBERLAIN

NAME(Please Print)	TITLE(Please Print)	ORGANIZATION(Please Print)
G.R. PARKOS	MGR, P.COMP. METHODS DEV	OREACTOR P.C. ENG.
T P Shannon	Engineer, OCD&M-3	NUCLEAR ENGINEERING
E.T. NORTIN	SR. ENGINEER	OPERATING REACTOR PROCESS COMPUTER ENGINEERING
J. P. Rea	Sr. Engineer / Tech Leader	Nuclear Engineering
R B Linford	Mgr, Operating Reactor PC Engg	Core Nuclear Design

B-1
Rev. 3-12-75

ATTENDANCE LIST

COMPANY: GENERAL ELECTRIC DOCKET NO. 99900403

Date: 6-11-82 ☐ Pre-Inspection Conference ☒ Post Inspection Conference

NAME (Please Print)	TITLE (Please Print)	ORGANIZATION (Please Print)
D.D. CHAMBERLAIN	REACTOR ENGINEER	US NRC REG. IV
P.H. HARRELL	REACTOR ENGINEER	USNRC R.IV
D.E. LEE	MGR QC	NED/NESO
C.A. CAMERON	MGR-SAFETY EVALUATION PROGS.	GE - S&LO
E W GIAMBALVO	Sr Engr S.E.P.	GE- S&LO.
N E BARCLAY	AUDIT COORD.	GE NC & ID
R.J. VALENCIA	Audit COORDINATOR	NED/NESO
R C BOESSER	MGR TECHNOLOGY PROG	GE NPSD
J K Powledge	Mgr-QAEE&I	GE-NEPO
A. Breed	Mgr QA	GE- P&QAO
C.L. Buckner	Spec. Equal. Sys.	GE-NEPO
C.W DILLMANN	MGR-FCV, ~~REACTOR~~ DBS	GE-NPSED
G.G. Sherwood	Mgr Safety & Lic	GE-NPSD