



UNITED STATES
NUCLEAR REGULATORY COMMISSION

REGION II
101 MARIETTA STREET, N.W.
ATLANTA, GEORGIA 30303

Report Nos. 50-388/82-23, 50-339/82-23

Licensee: Virginia Electric and Power Company
P. O. Box 26666
Richmond, VA 23262

Facility Name: North Anna Units 1 and 2

Docket Nos. 50-338, 50-339

License Nos. NPF-4 and NPF-7

Inspection at North Anna site near Mineral, Virginia

Inspector: E. H. Brooks
E. H. Brooks

7-19-82
Date Signed

Approved by: F. Jape
F. Jape, Section Chief
Engineering Inspection Branch
Division of Engineering and Technical Programs

7/24/82
Date Signed

SUMMARY

Inspection on June 28 - July 2, 1982

Areas Inspected

This routine, unannounced inspection involved 31 inspector-hours on site in the areas of pipe supports and directional restraints, and types B and C leakage rate testing.

Results

Within the scope of this inspection no violations or deviations were identified.

REPORT DETAILS

1. Persons Contacted

Licensee Employees

W. R. Cartwright, Station Manager
*E. W. Harrell, Assistant Station Manager
*D. L. Benson, Superintendent Operations
*R. T. Johnson, Quality Assurance
*J. A. Hanson, Jr., Superintendent Technical Services
*J. C. Smith, Engineer Supervisor Performance & Test
J. Roth, Engineer, Performance & Test
J. Masticone, Operations Coordinator
F. Timpano, Engineer Performance & Test
*M. E. Fellows, Staff Assistant

NRC Resident Inspector

*D. Johnson
M. Shymlock

*Attended exit interview

2. Exit Interview

The inspection scope and findings were summarized on July 2, 1982, with those persons indicated in paragraph 1 above. Licensee acknowledged the findings without significant comment.

3. Licensee Action on Previous Inspection Findings

Not inspected.

4. Unresolved Items

Unresolved items were not identified during this inspection.

5. Inspection of Pipe Support and Restraint Systems

The inspector and licensee representative performed an inspection of snubbers installed in both Units 1 and 2. Approximately 75 snubbers in the main steam valve house, service building, safeguards building and auxiliary feedwater pump house were inspected. All snubbers appeared to be well maintained, properly installed, and free from "lockup." One snubber in the Unit 2 mechanical equipment room was exposed to high temperature feedwater leaking from feedwater valve 2-FW-196. The water was impinging on the snubber piston rod and seal. Licensee management was advised of the potential for snubber damage. The licensee was in the process of performing

scheduled snubber inspection and functional testing on Unit 1. The previous inspection and functional testing of Unit 1 snubbers was performed during the 1981 spring outage.

The most recent inspection and functional testing of Unit 2 snubbers was performed during the 1982 spring outage.

Based on technical specifications requirements for augmented inservice inspection, both Units 1 and 2 are presently on a 12 months $\pm$ 25% subsequent visual inspection period. Also, the licensee has properly selected for functional testing a representative sample in accordance with the technical specifications requirements for both Units 1 and 2.

The licensee has received an exemption from the requirement to functionally test large snubbers greater than 50 KIP around the steam generators and reactor coolant pumps, during the current outage only. These snubbers will be included in representative samples selected for testing during future outages. Open Items (79-04-02 and 79-04-06) concerning this subject are considered closed.

Bulletin 81-01 concerns surveillance of mechanical snubbers. The licensee has responded to the bulletin, stating that no mechanical snubbers are installed in North Anna Unit 1. Four mechanical snubbers installed in Unit 2 will be sent to Wyle Laboratories for testing and the NRC will be advised of the test results. This item remains open pending receipt of test results.

In the area of pipe support and restraint systems no violations or deviations were identified.

6. Type B and C Local Leakage Rate Testing

The inspector reviewed the documentation of local leakage rate testing for containment penetrations (Type B) and containment isolation valves (Type C) for both Units 1 and 2. The allowable leakage rate for Type B and C tests as stated in Appendix J to 10 CFR 50 is 0.6 times the maximum allowable leakage rate as specified for periodic tests in the operating license. For North Anna Units 1 and 2 the allowable leakage rate is 171 SCFH. The as found leakage rates for both units exceeded 171 SCFH by a factor of at least 3, due primarily to isolation valve leakage. Subsequent to repair, the leakage was reduced in all cases to less than 60 SCFH. The licensee is required by Appendix J to provide to the NRC a summary analysis of periodic Type B and Type C tests that were performed since the last containment integrated leakage rate test (Type A).

In the area of Type B and C local leakage rate testing no violations or deviations were identified.