



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

GULF STATES UTILITIES COMPANY

DOCKET NO. 50-458

RIVER BEND STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 52  
License No. NPF-47

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Gulf States Utilities Company (the licensee) dated August 22, 1990, and supplemented by letter dated October 17, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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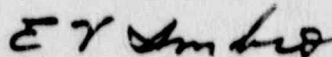
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-47 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 12 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. GSU shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Eugene V. Imbro, Acting Director  
Project Directorate IV-2  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 23, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 52

FACILITY OPERATING LICENSE NO. NPF-47

DOCKET NO. 50-458

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change. The overleaf pages are provided to maintain document completeness.

REMOVE

3/4 3-22

3/4 3-23

3/4 3-79

INSERT

3/4 3-22

3/4 3-23

3/4 3-79

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
4. <u>REACTOR WATER CLEANUP SYSTEM ISOLATION (Cont'd)</u>		
d. Equipment Area $\Delta$ Temperature - High		
1. Heat Exchanger Room	$< 39^{\circ}\text{F}$	$< 42.5^{\circ}\text{F}$
2. Pump Rooms A and B	$< 78^{\circ}\text{F}$	$< 82^{\circ}\text{F}$
3. Valve Nest Room	$< 46^{\circ}\text{F}$	$< 49.5^{\circ}\text{F}$
4. Demineralizer Rooms 1 and 2	$< 46^{\circ}\text{F}$	$< 49.5^{\circ}\text{F}$
5. Receiving Tank Room	$< 46^{\circ}\text{F}$	$< 49.5^{\circ}\text{F}$
e. Reactor Vessel Water Level - Low Low Level 2	$\geq - 43$ inches*	$\geq - 47$ inches
f. Main Steam Line Tunnel Ambient Temperature - High	$< 141^{\circ}\text{F}$	$< 148.5^{\circ}\text{F}$
g. Main Steam Line Tunnel $\Delta$ Temperature - High	$< 57^{\circ}\text{F}$	$< 61^{\circ}\text{F}$
h. SLCS Initiation	NA	NA
5. <u>REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</u>		
a. RCIC Steam Line Flow - High	$\leq 127'' \text{ H}_2\text{O}$	$\leq 135.5'' \text{ H}_2\text{O}$
b. RCIC Steam Line Flow - High Timer	$\geq 3$ seconds	$\leq 13$ seconds
c. RCIC Steam Supply Pressure - Low	$\geq 60$ psig	$\geq 55$ psig
d. RCIC Turbine Exhaust Diaphragm Pressure - High	$\leq 10$ psig	$\leq 20$ psig
e. RCIC Equipment Room Ambient Temperature - High	$\leq 182^{\circ}\text{F}$	$\leq 186.4^{\circ}\text{F}$
f. RCIC Equipment Room $\Delta$ Temperature - High	$\leq 96^{\circ}\text{F}$	$\leq 99^{\circ}\text{F}$

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
5. <u>REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</u> (Cont'd)		
g. Main Steam Line Tunnel Ambient Temperature - High	$\leq 141^{\circ}\text{F}$	$\leq 148.5^{\circ}\text{F}$
h. Main Steam Line Tunnel $\Delta$ Temperature - High	$\leq 57^{\circ}\text{F}$	$\leq 61^{\circ}\text{F}$
i. Main Steam Line Tunnel Temperature Timer	0 seconds	NA
j. RHR Equipment Room Ambient Temperature - High	$\leq 117^{\circ}\text{F}$	$\leq 121.1^{\circ}\text{F}$
k. RHR Equipment Room $\Delta$ Temperature - High	$\leq 29^{\circ}\text{F}$	$\leq 33.6^{\circ}\text{F}$
l. RHR/RCIC Steam Line Flow - High	$\leq 60.7'' \text{H}_2\text{O}$	$\leq 64.2'' \text{H}_2\text{O}$
m. Drywell Pressure - High	$\leq 1.68 \text{ psig}$	$\leq 1.88 \text{ psig}$
n. Manual Initiation	NA	NA
6. <u>RHR SYSTEM ISOLATION</u>		
a. RHR Equipment Area Ambient Temperature - High	$\leq 117^{\circ}\text{F}$	$\leq 121.1^{\circ}\text{F}$
b. RHR Equipment Area $\Delta$ Temperature - High	$\leq 29^{\circ}\text{F}$	$\leq 33.6^{\circ}\text{F}$
c. Reactor Vessel Water Level - Low Level 3	$\geq 9.7 \text{ inches}^*$	$\geq 8.7 \text{ inches}$
d. Reactor Vessel Water Level - Low Low Low Level 1	$\geq - 143 \text{ inches}^*$	$\geq - 147 \text{ inches}$

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
6. <u>RHR SYSTEM ISOLATION</u> (Cont'd)		
e. Reactor Vessel (RHR Cut-in Permissive) Pressure - High	≤ 135 psig	≤ 150 psig
f. Drywell Pressure - High	≤ 1.68 psig	≤ 1.88 psig
7. <u>MANUAL INITIATION</u>	NA	NA

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\* See Bases Figure B 3/4 3-1.

TABLE 3.3.2-3  
ISOLATION SYSTEM INSTRUMENTATION RESPONSE TIME

<u>TRIP FUNCTION</u>	<u>RESPONSE TIME (Seconds)#</u>
<b>1. <u>PRIMARY CONTAINMENT ISOLATION</u></b>	
a. Reactor Vessel Water Level - Low Low Level 2	< 10 <sup>(a)</sup>
b. Drywell Pressure - High	< 10 <sup>(a)</sup>
c. Containment Purge Isolation Radiation - High <sup>(b)</sup>	NA
<b>2. <u>MAIN STEAM LINE ISOLATION</u></b>	
a. Reactor Vessel Water Level - Low Low Low Level 1	< 1.0 * / < 10 <sup>(a)**</sup>
b. Main Steam Line Radiation - High <sup>(b)</sup>	< 1.0 * / < 10 <sup>(a)**</sup>
c. Main Steam Line Pressure - Low	< 1.0 * / < 10 <sup>(a)**</sup>
d. Main Steam Line Flow - High	< 0.5 * / < 10 <sup>(a)**</sup>
e. Condenser Vacuum - Low	NA
f. Main Steam Line Tunnel Temperature - High	NA
g. Main Steam Line Tunnel Δ Temperature - High	NA
h. Main Steam Line Area Temperature - High (Turbine Bldg)	NA
<b>3. <u>SECONDARY CONTAINMENT ISOLATION</u></b>	
a. Reactor Vessel Water Level - Low Low Level 2	< 10 <sup>(a)</sup>
b. Drywell Pressure - High	< 10 <sup>(a)</sup>
c. Fuel Building Ventilation Exhaust Radiation - High <sup>(b)</sup>	NA
d. Reactor Building Annulus Ventilation Exhaust Radiation - High <sup>(b)</sup>	NA
<b>4. <u>REACTOR WATER CLEANUP SYSTEM ISOLATION</u></b>	
a. Δ Flow - High	< 10 <sup>(a)##</sup>
b. Δ Flow Timer	NA
c. Equipment Area Temperature - High	NA
d. Equipment Area Δ Temperature - High	NA
e. Reactor Vessel Water Level - Low Low Level 2	≤ 10 <sup>(a)</sup>
f. Main Steam Line Tunnel Ambient Temperature - High	NA
g. Main Steam Line Tunnel Δ Temperature - High	NA
h. SLCS Initiation	NA
<b>5. <u>REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</u></b>	
a. RCIC Steam Line Flow - High	< 10 <sup>(a)###</sup>
b. RCIC Steam Line Flow-High Timer	NA
c. RCIC Steam Supply Pressure - Low	< 10 <sup>(a)</sup>
d. RCIC Turbine Exhaust Diaphragm Pressure - High	NA
e. RCIC Equipment Room Ambient Temperature - High	NA
f. RCIC Equipment Room Δ Temperature - High	NA
g. Main Steam Line Tunnel Ambient Temperature - High	NA
h. Main Steam Line Tunnel Δ Temperature - High	NA

TABLE 3.3.7.4-2 (Continued)

## REMOTE SHUTDOWN SYSTEM CONTROLS

	MINIMUM CHANNELS OPERABLE	
	RSP1	RSP2
22. RHR Shutdown Cooling MOV (1E12*MOV F006A, 6B)	2 <sup>(a)</sup>	NA
23. RHR Outboard Shutdown Isolation MOV (1E12*MOV F008)	1	NA
24. RHR Inboard Shutdown Isolation MOV (1E12*MOV F009)	1	NA
25. RHR Hx Flow to Suppression Pool MOV (1E12*MOV F011A, B)	1	1
26. RHR Reactor Head Spray MOV (1E12*MOV F023)	1	NA
27. RHR Test Line MOV (1E12*MOV F024A, B)	1	1
28. Deleted		
29. RHR Injection Shutoff MOV (1E12*MOV F027A, B)	1	1
30. RHR Upper Pool Cooling Shutoff MOV (1E12*MOV F037A, B)	1	1
31. RHR Injection MOV (1E12*MOV F042A, B, C)	1	2 <sup>(a)</sup>
32. RHR Hx Shell Side Inlet MOV (1E12*MOV F047A, B)	1	1
33. RHR Hx Shell Side Bypass MOV (1E12*MOV F048A, B)	1	1
34. RHR Discharge to Radwaste MOV (1E12*MOV F040)	1	NA
35. Deleted		
36. RHR Injection MOV (1E12*MOV F053A, B)	1	1
37. RHR Pump Minimum Flow MOV (1E12*MOV F064A, B, C)	1	2 <sup>(a)</sup>
38. RHR Hx Water Discharge MOV (1E12*MOV F068A, B)	1	1
39. Safety Relief Valves (1E21*RV F051C, G, D)	3 <sup>(a)</sup>	3 <sup>(a)</sup>
40. SSW Pump (1SWP*P2A, 2C, <sup>(b)</sup> 2B, 2D)	1	2 <sup>(a)</sup>
41. Normal Service Water Isolation MOV (1SWP*MOV 50A, B)	1	1
42. SSW Cooling Tower Inlet MOV (1SWP*MOV 55A, B)	1	1
43. SSW Component Cooling Water Inlet MOV (1SWP*MOV 510A, B)	1	1
44. SSW Component Cooling Water Outlet MOV (1SWP*MOV 504A, B)	1	1

(a) One per control equipment.

(b) SSW pump 1SWP\*P2C is provided on panel 1EGS\*PNL4C.



TABLE 4.3.7.4-1

REMOTE SHUTDOWN MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Reactor Vessel Pressure	M	R
2. Reactor Vessel Water Level	M	R
3. Safety/Relief Valve Demand Position	M	NA
4. Suppression Pool Water Level	M	R
5. Suppression Pool Water Temperature	M	R
6. Drywell Pressure	M	R
7. Drywell Temperature	M	R
8. RHR System Flow: Loop A	M	R
Loop B	M	R
Loop C	M	R
9. RHR Hx Cooling Water System Flow: Loop A	M	R
Loop B	M	R
10. RCIC System Flow	M	R
11. RCIC Turbine Speed	M	R