

APPENDIX A

NOTICE OF VIOLATION

Portland General Electric Company  
Trojan Nuclear Plant

Docket No. 50-344  
License No. NPF-1

During an NRC inspection conducted from September 2, 1990 through October 6, 1990, one violation of NRC requirements was identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," 10 CFR Part 2, Appendix C (1990), the violation is listed below:

Trojan Technical Specification 4.7.3.1.b states that: "The Component Cooling Water (CCW) system shall be demonstrated OPERABLE by verifying that each CCW pump is OPERABLE when tested pursuant to Specification 4.0.5."

Technical Specification 4.0.5 states, in part, that: "Inservice inspection of ASME Code Class 1, 2 and 3 components and inservice testing of ASME Code Class 1, 2 and 3 pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code..."

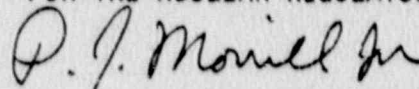
Section IWP 4120 of the ASME Boiler and Pressure Vessel Code states that the "Full scale range of each instrument shall be three times the reference value or less."

Contrary to the above on September 14, 1990, the B train CCW pump is an ASME code class 3 pump and the bearing temperature was taken with an instrument that was 0 to 500 degrees F, which is more than three times the reference value of the bearing temperature

This is a Severity Level V violation (Supplement I).

Pursuant to the provisions of 10 CFR 2.201, Portland General Electric Company is hereby required to submit a written statement of explanation to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555 with a copy to the Regional Administrator, Region V, and a copy to the NRC Senior Resident Inspector, Trojan, within 30 days of the date of the letter transmitting this Notice. This reply should be clearly marked as a "Reply to a Notice of Violation" and should include for each violation: (1) the reason for the violation if admitted, (2) the corrective steps that have been taken and the results achieved, (3) the corrective steps that will be taken to avoid further violations, and (4) the date when full compliance will be achieved. If an adequate reply is not received within the time specified in this Notice, an order may be issued to show cause why the license should not be modified, suspended, or revoked or why such other action as may be proper should not be taken. Consideration may be given to extending the response time for good cause shown.

FOR THE NUCLEAR REGULATORY COMMISSION



S. A. Richards, Chief  
Reactor Projects Branch

Dated at Walnut Creek, California  
this 8<sup>th</sup> day of November 1990