LICENSEE EVENT REPORT CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) 2 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 57 CAT C|B|E|P 0 1 CON'T L 6 0 5 0 - 0 3 2 4 7 1 1 1 1 7 8 8 1 2 1 3 7 8 9 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80 REPORT 0 1 SOURCE EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) On November 11, 1978 the reactor was in hot shutdown and RHR was in the 0 2 steam condensing mode of operation. At 0605 the B loop of RHR was shut down. 0 3 At 0800 the torus level increased approximately .2" above the Technical 0 4 Specification limit of -27". 0 5 0 6 (Technical Specification 3.6.2.1) 0 7 OR SYSTEM CAUSE CAUSE COMP VALVE CODE CODE SUBCODE COMPONENT CODE SUBCODE SUBCODE C F (1) B (12) 0 9 L V E X C (15 A (13) V (14 A (16) 13 SEQUENTIAL OCCURRENCE REVISION REPORT EVENT YEAR REPORT NO. CODE LER/RO TYPE NO. 7 18 REPORT 0 18 12 0131 01 L NUMBER EFFECT ON PLANT SUBMITTED NPRD-4 COMPONENT CTION FUTURE METHOD PRIME COMP HOURS (22) AKEN ACTION SUPPLIER N 24 18 F (19 Z 21 ¥ 23 Z (20) 0 0 0 10 25 A 13 19 1A 43 15 (26 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) Torus level was returned to normal in 3.2 hours (1112). RHR check valves 1 0 Ell-F031 will not fully seat on low differential pressures. Demineralized make-up 1 1 water leakage through the valves from the RHR keep-fill system tends to raise 1 7 torus level. Plant modifications 77-255 (cont'd) 1 3 1 4 FACILITY METHOD OF OTHER STATUS (30) % POWER DISCOVERY DESCRIPTION (32) DISCOVERY G (28) 0 0 0 0 29 NA B (31) Operator Observation 1 5 80 CONTENT AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36) OF RELEASE RELEASED 1 6 Z (33) Z (34) NA NA 80 PERSONNEL EXPOSURES DESCRIPTION (39) NUMBER 7 0 Z (38) 0 (37) NA PERSONNEL INJURIES DESCRIPTION (41) NUMBER 8 0 0 (40 NA 0 LOSS OF OR DAMAGE TO FACILITY (43) TYPE DESCRIPTION Z (42) 1 9 NA 10 7812200199 PUBLICITY NRC USE ONLY DESCRIPTION (45) SSUED Z (44 NA 2 0 10 68 69 919-457-6701 A. C. Tollison NAME OF PREPARER -PHONE:.

LER CONTINUATION - RO# 2-78-82

Facility: BSEP Unit #2

Event Date November 11, 1978

and 77-256 are designed to eliminate the torus level increases due to valve leakages. These modifications use water from the torus to keep safety system piping (RHR) pressurized. This will prevent any changes in torus level due to RHR check valve leakage. The modifications are almost complete and should be operable by the end of the upcoming refueling outage.