UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DEC 12 1978

MEMORANDUM FOR: L. P. Crocker, Technical Assistant to the Director, DPM

FROM: T. M. Novak, Chief, Reactor Systems Branch, DSS

SUBJECT: INFORMATION REQUEST FROM DR. CARBON, ACRS

Following my discussion with Dr. Carbon during the November ACRS meeting, I agreed to provide him with some additional information regarding natural circulation capabilities of PWRs following a loss of flow transient. The enclosed information represents only a partial survey of PWR performance during natural circulation tests. It supports the staff position that natural circulation in a PWR is well behaved and the likelihood of fuel rod overheating resulting from a critical heat flux condition is very unlikely.

Please transmit the enclosure to Dr. Carbon for his information.

Thomas M. Novak, Chief
Reactor Systems Branch
Division of Systems Safety

Enclosure:
Memo, J. Olshinski to S. Israel,
dated Dec. 7, 1978

cc: R. Mattson
R. Tedesco

Contact: Tom Novak, NRR
49-27460

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