

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

In addition to the requirements of Specification 4.0.5, each Reactor Coolant Pump flywheel shall be inspected per the recommendations of Regulatory Position C.4.b of Regulatory Guide 1.14, Revision 1, August 1975.

4.4.10.1.2 Augmented Inservice Inspection Program for Main Steam and Main Feedwater Piping - The unencapsulated welds greater than 4 inches in nominal diameter in the main steam and main feedwater piping runs located outside the containment and traversing safety related areas or located in compartments adjoining safety-related areas shall be inspected per the following augmented inservice inspection program using the applicable rules, acceptable criteria, and repair procedures of the ASME Boiler and Pressure Vessel Code, Section XI, 1974 Edition and Addenda through Summer 1975, for Class 2 components.

- a. System integrity and baseline data shall be established by performing a 100% volumetric examination of each weld prior to exceeding 18 months of operation.
- b. Each weld shall be examined in accordance with the above ASME Code requirements, except that 100% of the welds shall be examined, cumulatively, during each 10 year inspection interval. The welds to be examined during each inspection period shall be selected to provide a representative sample of the conditions of the welds. If these examinations reveal unacceptable structural defects in one or more welds, an additional 1/3 of the welds shall be examined and the inspection schedule for the repaired welds shall revert back to the first 10 year inspection program. If additional unacceptable defects are detected in the second sampling, the remainder of the welds shall also be inspected.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

In addition to the requirements of Specification 4.0.5, each Reactor Coolant Pump flywheel shall be inspected per the recommendations of Regulatory Position C.4.b of Regulatory Guide 1.14, Revision 1, August 1975.

4.4.10.1.2 Augmented Inservice Inspection Program for Main Steam and Main Feedwater Piping - The unencapsulated welds greater than 4 inches in nominal diameter in the main steam and main feedwater piping runs located outside the containment and traversing safety related areas or located in compartments adjoining safety-related areas shall be inspected per the following augmented inservice inspection program using the applicable rules, acceptable criteria, and repair procedures of the ASME Boiler and Pressure Vessel Code, Section XI, 1974 Edition and Addenda through Summer 1975, for Class 2 components.

- a. System integrity and baseline data shall be established by performing a 100% volumetric examination of each weld prior to exceeding 18 months of operation.
- b. Each weld shall be examined in accordance with the above ASME Code requirements, except that 100% of the welds shall be examined, cumulatively, during each 10 year inspection interval. The welds to be examined during each inspection period shall be selected to provide a representative sample of the conditions of the welds. If these examinations reveal unacceptable structural defects in one or more welds, an additional 1/3 of the welds shall be examined and the inspection schedule for the repaired welds shall revert back to the first 10 year inspection program. If additional unacceptable defects are detected in the second sampling, the remainder of the welds shall also be inspected.

EMERGENCY CORE COOLING SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

2. Within 4 hours following completion of maintenance on the valve or its operator by measurement of stem travel when the ECCS subsystems are required to be **OPERABLE**.

HPSI SYSTEM

Valve Number

MOV-616
MOV-626
MOV-636
MOV-646

Valve Number

MOV-617
MOV-627
MOV-637
MOV-647

- h. By performing a flow balance test during shutdown following completion of HPSI system modifications that alter system flow characteristics and verifying the following flow rates:

HPSI SYSTEM

Single Pump

170 \pm 5 gpm to each injection leg.

EMERGENCY CORE COOLING SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

2. Within 4 hours following completion of maintenance on the valve or its operator by measurement of stem travel when the ECCS subsystems are required to be **OPERABLE**.

HPSI SYSTEM

Valve Number

MOV-616
MOV-626
MOV-636
MOV-646

Valve Number

MOV-617
MOV-627
MOV-637
MOV-647

- h. By performing a flow balance test during shutdown following completion of HPSI system modifications that alter system flow characteristics and verifying the following flow rates:

HPSI SYSTEM

Single Pump

170 ± 5 gpm to each injection leg.

ADMINISTRATIVE CONTROLS

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The facility shall be placed in at least **HOT STANDBY** within one hour.
- b. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within one hour. The Manager - Nuclear Power Department and the OSSRC shall be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the POSRC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems, or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the OSSRC, and the Manager - Nuclear Power Department within 14 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, 1978 Revision.
- b. Refueling operations.
- c. Surveillance and test activities of safety-related equipment.
- d. Security Plan Implementation.
- e. Emergency Plan Implementation.
- f. Fire Protection Program Implementation.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the POSRC and approved by the Plant Superintendent prior to implementation and reviewed periodically as set forth in administrative procedures.

PROPOSED CHANGE

TABLE 3.7-4

SAFETY-RELATED HYDRAULIC SNUBBERS*

<u>SNUBBER NUMBER</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A OR I)</u>	<u>HIGH RADIATION ZONE** (YES OR NO)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (YES OR NO)</u>
1-83-40	MS FROM SG #12 TO AFW PUMP 27'	A	No	No
1-83-40A	MS FROM SG #12 TO AFW PUMP 27'	A	No	No
1-83-41	AFW PUMP ISOLATION VALVE BYPASS 27'	A	No	No
1-83-44	STEAM SUPPLY TO AFW PUMP 27'	A	No	No
1-83-47	AFW PUMP ISOLATION VALVE BYPASS 27'	A	No	No
1-83-48	AFW PUMP ISOLATION VALVE BYPASS 27'	A	No	No
1-83-49	MAIN STM LINE ENCAPSULATION 27'	A	No	Yes
1-83-50	MAIN STM LINE ENCAPSULATION 27'	A	No	Yes
1-83-51	MAIN STM LINE ENCAPSULATION 27'	A	No	Yes
1-83-52	MAIN STM LINE ENCAPSULATION 27'	A	No	Yes
1-83-53	MAIN STM LINE ENCAPSULATION 27'	A	No	Yes
1-83-54	MAIN STM LINE ENCAPSULATION 27'	A	No	Yes

CONTAINMENT SYSTEMS

CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

a. An overall integrated leakage rate of:

1. $\leq L_a$ (346,000 SCCM), 0.20 percent by weight of the containment air per 24 hours at P_a , 50 psig, or
2. $\leq L_t$ (61,600 SCCM), 0.058 percent by weight of the containment air per 24 hours at a reduced pressure of P_t , 25 psig.

b. A combined leakage rate of $\leq 0.60 L_a$ (207,600 SCCM), for all penetrations and valves subject to Type B and C tests when pressurized to P_a .

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With either (a) the measured overall integrated containment leakage rate exceeding $0.75 L_a$ (259,500 SCCM) or $0.75 L_t$ (46,200 SCCM), as applicable, or (b) with the measured combined leakage rate for all penetrations and valves subject to Types B and C tests exceeding $0.60 L_a$, restore the leakage rate(s) to within the limit(s) prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of ANSI N45.4 - 1972:

a. Three Type A tests (Overall Integrated Containment Leakage Rate) shall be conducted at 40 ± 10 month intervals during shutdown at either P_a (50 psig) or at P_t (25 psig) during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.

CONTAINMENT SYSTEMS

CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

a. An overall integrated leakage rate of:

1. $\leq L_a$ (346,000 SCCM), 0.20 percent by weight of the containment air per 24 hours at P_a , 50 psig, or
2. $\leq L_t$ (44,600 SCCM), 0.058 percent by weight of the containment air per 24 hours at a reduced pressure of P_t , 25 psig.

b. A combined leakage rate of $\leq 0.60 L_a$ (207,600 SCCM), for all penetrations and valves subject to Type B and C tests when pressurized to P_a .

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With either (a) the measured overall integrated containment leakage rate exceeding $0.75 L_a$ (259,500 SCCM) or $0.75 L_t$ (33,400 SCCM), as applicable, or (b) with the measured combined leakage rate for all penetrations and valves subject to Types B and C tests exceeding $0.60 L_a$, restore the leakage rate(s) to within the limit(s) prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of ANSI N45.4 - 1972:

a. Three Type A tests (Overall Integrated Containment Leakage Rate) shall be conducted at 40 ± 10 month intervals during shutdown at either P_a (50 psig) or at P_t (25 psig) during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- b. If any periodic Type A test fails to meet either $.75 L_a$ or $.75 L_t$, the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet either $.75 L_a$ or $.75 L_t$, a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet either $.75 L_a$ or $.75 L_t$ at which time the above test schedule may be resumed.
- c. The accuracy of each Type A test shall be verified by a supplemental test which:
 - 1. Confirms the accuracy of the Type A test by verifying that the difference between supplemental and Type A test data is within $0.25 L_a$ (86,500 SCCM) or $0.25 L_t$ (15,400 SCCM).
 - 2. Has a duration sufficient to establish accurately the change in leakage between the type A test and the supplemental test.
 - 3. Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 25 percent of the total measured leakage rate at P_a (50 psig) or P_t (25 psig).
- d. Type B and C tests shall be conducted with gas at P_a (50 psig) at intervals no greater than 24 months except for tests involving air locks.
- e. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.
- f. All test leakage rates shall be calculated using observed data converted to absolute values. Error analyses shall be performed to select a balanced integrated leakage measurement system.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- b. If any periodic Type A test fails to meet either $.75 L_a$ or $.75 L_t$, the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet either $.75 L_a$ or $.75 L_t$, a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet either $.75 L_a$ or $.75 L_t$ at which time the above test schedule may be resumed.
- c. The accuracy of each Type A test shall be verified by a supplemental test which:
 - 1. Confirms the accuracy of the Type A test by verifying that the difference between supplemental and Type A test data is within $0.25 L_a$ (86,500 SCCM) or $0.25 L_t$ (11,100 SCCM).
 - 2. Has a duration sufficient to establish accurately the change in leakage between the type A test and the supplemental test.
 - 3. Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 25 percent of the total measured leakage rate at P_a (50 psig) or P_t (25 psig).
- d. Type B and C tests shall be conducted with gas at P_a (50 psig) at intervals no greater than 24 months except for tests involving air locks.
- e. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.
- f. All test leakage rates shall be calculated using observed data converted to absolute values. Error analyses shall be performed to select a balanced integrated leakage measurement system.

CONTAINMENT SYSTEMS

CONTAINMENT AIR LOCKS

LIMITING CONDITION FOR OPERATION

- 3.6.1.3 Each containment air lock shall be OPERABLE with:
- a. Both doors closed except when the air lock is being used for normal transit entry and exit through the containment, then at least one air lock door shall be closed, and
 - b. An overall air lock leakage rate of $\leq 0.05 L_a$ (17,300 SCCM) at P_a , 50 psig.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

- a. With an air lock inoperable, except as a result of an inoperable door gasket, restore the air lock to OPERABLE status within 24 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With an air lock inoperable due to an inoperable door gasket:
 1. Maintain the remaining door of the affected air lock closed and sealed, and
 2. Restore the air lock to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

- 4.6.1.3 Each containment air lock shall be demonstrated OPERABLE:
- a.* After each opening, except when the airlock is being used for multiple entries, then at least once per 72 hours by verifying that the seal leakage is $< 0.0002 L_a$ (69.2 SCCM) as determined by precision flow measurement when the volume between the door seals is pressurized to a constant pressure of 15 psig,

* Exemption to Appendix "J" of 10 CFR 50.

CONTAINMENT SYSTEMS

CONTAINMENT AIR LOCKS

LIMITING CONDITION FOR OPERATION

- 3.6.1.3 Each containment air lock shall be OPERABLE with:
- a. Both doors closed except when the air lock is being used for normal transit entry and exit through the containment, then at least one air lock door shall be closed, and
 - b. An overall air lock leakage rate of $\leq 0.05 L_a$ (17,300 SCCM) at P_a , 50 psig.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

- a. With an air lock inoperable, except as a result of an inoperable door gasket, restore the air lock to OPERABLE status within 24 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With an air lock inoperable due to an inoperable door gasket:
 1. Maintain the remaining door of the affected air lock closed and sealed, and
 2. Restore the air lock to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

- 4.6.1.3 Each containment air lock shall be demonstrated OPERABLE:
- a.* After each opening, except when the airlock is being used for multiple entries, then at least once per 72 hours by verifying that the seal leakage is $< 0.0002 L_a$ (69.2 SCCM) as determined by precision flow measurement when the volume between the door seals is pressurized to a constant pressure of 15 psig,

* Exemption to Appendix "J" of 10 CFR 50.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

4.6.4.1.2 Each isolation valve specified in Table 3.6-1 shall be demonstrated OPERABLE during the COLD SHUTDOWN or REFUELING MODE at least once per 18 months by:

- a. Verifying that on each containment isolation Channel A or Channel B test signal, each required isolation valve actuates to its isolation position.
- b. Verifying that on each Containment Radiation-High Test Channel A or Channel B test signal, both required containment purge valves actuate to their isolation position.
- c. Verifying that on each Safety Injection Actuation Channel A or Channel B test signal, each required isolation valve actuates to its isolation position.

4.6.4.1.3 The isolation time of each power operated or automatic valve of Table 3.6-1 shall be determined to be within its limit when tested pursuant to Technical Specification 4.0.5.

4.6.4.1.4 Containment purge isolation valves shall be demonstrated OPERABLE at least once every 6 months by verifying that when the measured leakage rate is added to the leakage rates determined pursuant to Technical Specification 4.6.1.2.d for all other Type B or C penetrations, the combined leakage rate is less than or equal to $0.60 L_a$ (207,600 SCCM). The leakage rate for the containment purge isolation valves shall also be compared to the previously measured leakage rate to detect excessive valve degradation.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

4.6.4.1.2 Each isolation valve specified in Table 3.6-1 shall be demonstrated OPERABLE during the COLD SHUTDOWN or REFUELING MODE at least once per 18 months by:

- a. Verifying that on each containment isolation Channel A or Channel B test signal, each required isolation valve actuates to its isolation position.
- b. Verifying that on each Containment Radiation-High Test Channel A or Channel B test signal, both required containment purge valves actuate to their isolation position.
- c. Verifying that on each Safety Injection Actuation Channel A or Channel B test signal, each required isolation valve actuates to its isolation position.

4.6.4.1.3 The isolation time of each power operated or automatic valve of Table 3.6-1 shall be determined to be within its limit when tested pursuant to Technical Specification 4.0.5.

4.6.4.1.4 Containment purge isolation valves shall be demonstrated OPERABLE at least once every 6 months by verifying that when the measured leakage rate is added to the leakage rates determined pursuant to Technical Specification 4.6.1.2.d for all other Type B or C penetrations, the combined leakage rate is less than or equal to $0.60 L_a$ (207,600 SCCM). The leakage rate for the containment purge isolation valves shall also be compared to the previously measured leakage rate to detect excessive valve degradation.