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Director  
Office of Nuclear Reactor Regulation  
U S Nuclear Regulatory Commission  
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT  
Docket Nos. 282 License No. DPR-42  
306 DPR-60

Item II.K.3.30, NUREG-0737

References: 1. Letter G Owsley (Exxon) to Tedesco (NRC) dated July 8, 1982 (Subject: Transmittal of XN-NF-82-49 (P) )

Mr Eisenhut's October 31, 1980 letter, containing NUREG-0737, requested a response to item II.K.3.30, Revised Small-Break Loss-of-Coolant-Accident Methods To Show Compliance with 10 CFR Part 50, Appendix K by January 1, 1982. In our December 29, 1981 letter a schedule change was requested for item II.K.3.30 from January 1, 1982 until July 30, 1982.

At the time the December 29, 1981 letter was written both Westinghouse and Exxon fuel was being used in Prairie Island cores. As currently planned at most only one Westinghouse assembly will be in the Prairie Island cores until 1986 at which time we plan to purchase Westinghouse fuel again. All Westinghouse small break model changes are expected to be approved prior to use of Westinghouse fuel. Therefore, this letter only refers to the Exxon response to NUREG-0737 item II.K.3.30. Exxon has completed the review of their Small-Break Loss-of-Coolant-Accident methods (Reference 1). Please consider reference 1 as NSP's response to item II.K.3.30 for Docket Nos. 50-282 and 50-306.

Exxon indicates method/model changes were necessary and have been made. Following NRC approval of these method/model changes, NSP will provide a schedule for the response to item II.K.3.31, Plant-specific Calculations to show compliance with 10 CFR Part 50.46.

This completes our response to NUREG-0737 item II.K.3.30 for Prairie Island Nuclear Generating Plant.

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DMM/TMP/js

cc: Regional Administrator III, NRC  
NRR Project Manager, NRC  
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G Charnoff

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