



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DAIRYLAND POWER COOPERATIVE

DOCKET NO. 50-409

LACROSSE BOILING WATER REACTOR

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 29
License No. DPR-45

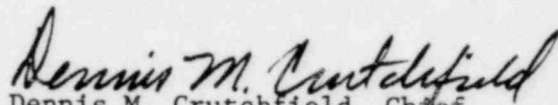
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Dairyland Power Cooperative (the licensee) dated May 19, 1981 and September 24, 1981, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public, and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Provisional Operating License No. DPR-45 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A issued October 31, 1969, with Authorization No. DPRA-6, as revised through Amendment No. 29, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 30, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 29

LACROSSE BOILING WATER REACTOR (LACBWR)

PROVISIONAL OPERATING LICENSE NO. DPR-45

Revise Appendix A by replacing the following pages with the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the areas of change.

Pages

6-18

6-19

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ADMINISTRATIVE CONTROLS

RECORD RETENTION (Continued)

- e. Records of transient or operational cycles for those facility components identified in Table 5.7.1-1.
- f. Records of reactor tests and experiments.
- g. Records of training and qualification for current members of the unit staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities required by the Operational Quality Assurance Manual.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the-ORC and the SRC.
- l. Records of environmental qualification which are covered under the Provisions of Section 6.13.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203 (c)(2) of 10 CFR 20, each high radiation area in which the intensity of radiation is greater than 100 mRem/hr but less than 1000 mRem/hr shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Special Work Permit (SWP).^{*} Any individual or group of individuals permitted to enter such areas shall be provided with one or more of the following:

^{*}Health Physics personnel or personnel escorted by Health Physics personnel shall be exempt from the SWP issuance requirement during the performance of their assigned radiation protection duties, provided they are following plant radiation protection procedures for entry into high radiation areas.

ADMINISTRATIVE CONTROLS

HIGH RADIATION AREA (Continued)

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.
 - b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and personnel have been made knowledgeable of them.
 - c. A health physics qualified individual (i.e., qualified in radiation protection procedures) with a radiation dose rate monitoring device and who is responsible for providing positive exposure control over the activities within the area and who will perform periodic radiation surveillance at the frequency which will be established by the Health and Safety Supervisor.
- 6.12.2 For each area with radiation levels greater than 1000 mrem/hr the controls of 6.12.1 shall be implemented and
- (1) Each entrance or access point to the area shall be maintained locked except during periods when access to the area is required, with positive control over each individual entry, or
 - (2) Each entrance or access point to the area shall be equipped with a control device which shall energize a conspicuous visible or audible alarm signal in such a manner that the individual entering the high radiation area and the licensee or a supervisor of the activity are made aware of the entry.
- 6.13 ENVIRONMENTAL QUALIFICATION*
- A. By no later than June 30, 1982, all safety-related electrical equipment in the facility shall be qualified in accordance with the provisions of: Division of Operating Reactors "Guidelines for Evaluating Environmental Qualification of Class IE Electrical Equipment in Operating Reactors" (DOR) Guidelines"; or, NUREG-0588, "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment", December 1979. Copies of these documents are attached to Order for Modification of License DPR-45 dated October 24, 1980.
 - B. By no later than December 1, 1980, complete and auditable records must be available and maintained at a central location which describe the environmental qualification method used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with the DOR guidelines or NUREG-0588. Thereafter, such records should be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified.

4.2.2.8 Operation of the forced circulation pumps shall be as described in Section 2.3.4.3.

4.2.2.9 The reactor shall not be operated above a power level of 82.5 megawatts thermal when only one forced circulation loop is in operation.

4.2.2.10 Automatic initiation of shutdown condenser operation shall cause the shutdown condenser tube side vent control valve to open and then to close automatically after a maximum of 2 min of condenser operation. This action shall be subject to remote manual override.

4.4.2.11 The return isolation valve on the decay heat system shall be locked closed except as required during reactor shutdown.

4.2.2.12 The decay heat pump shall not be placed in service unless the reactor is subcritical by at least 0.5% delta k/k by the criteria of Section 4.2.4.6, or unless boron solution has been injected into the reactor.

4.2.2.13 The purification system shall not be operated whenever the presence of boron solution is required in the reactor.

4.2.2.14 The high pressure core spray system shall provide water to the fuel elements whenever a reactor low water level scram is generated, except at times during which the reactor vessel head is removed, during injection of boron solution, or during performance of tests specified in Sec. 5.2.7 and 5.2.9.

4.2.2.15 One core spray pump may be removed from service for maintenance provided that all control rods are fully inserted, the reactor pressure is less than 85 psig, and the "Control Power" key switch is in the "OFF" position.

Both core spray pumps may be removed from service for maintenance provided that all control rods are fully inserted, the reactor pressure is at atmospheric, the "Control Power" key switch is in the "OFF" position and the low pressure core spray subsystem is operable.

Core spray pump 1B may be removed from service for up to one hour for maintenance of pressure switch 37-35-702. This provision shall be effective at 4:00 p.m. CDT May 19, 1981 and shall expire at 12:01 a.m. CDT on May 22, 1981.

4.2.2.16 The containment overhead storage tank shall always contain a minimum of 15,000 gal. of demineralized water supply for the high pressure core spray system whenever that system must be available as defined in Sec. 4.2.2.14.

4.2.2.17 The boron injection system shall be available for remote manual operation whenever the "Control Power" key switch is in the "ON" position or whenever all control rods are not fully inserted, except during tests specified in Sec. 5.2.7 and 5.2.9.