GPU Nuclear

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October 12, 1990
C311-90-2134

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Dear Sir:

Three Mile Island Nuclear Station, Unit I (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
Monthly Operating Report
September 1990

Enclosed are two copies of the September, 1990 Monthly Operating Report for Three Mile Island Nuclear Station, Unit 1.

Sincerely,

H. D. Hukill
Vice President & Director, TMI-1

HDH/WGH:

cc: T. T. Martin, USNRC
F. Young, USNRC
Attachments

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PDR ADOCK 05000289
R PNU

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1/1

OPERATIONS SUMMARY
SEPTEMBER 1990

The unit entered the month operating at ~96% reactor power (limited by high OTSG level) producing ~820 MWe gross electrical generation. A ΔTc of -4.5°F was input to compensate for the high OTSG level on the "B" side. The unit continued at 96% reactor power until September 28th when a reduction to 50% power was executed to support identification and repair of main condenser tube leaks. The unit closed the month operating at 50% reactor power producing 360 MW gross electrical generation. The "A" side of the main condenser was out of service with a ΔTc of -2.5°F input to offset fouling in the "B" OTSG.

MAJOR SAFETY RELATED MAINTENANCE

During September, the following major safety related maintenance activities were performed:

Hydrogen Recombiner HR-R-1B

Efforts to replace the Hydrogen Recombiner HR-R-1B expansion joints were begun in September. Holes were located in the existing bellows in June and since replacement bellows could not be procured, a modification to the existing configuration was planned and approved to replace the failed bellows with hard piping. Welding of flanges and pipe comprising the replacement sections was completed and x-rays of the joints were satisfactory. The bellows have been removed and the inlet and outlet piping prepped to accommodate welding in the new sections during October.

Primary Sample Valves CA-V30 A/B and CA-V31 A/B

The four Primary Sample Valves (CA-V30 A/B and CA-V31 A/B) were replaced during September. The valve packing gland bushing was found damaged on CA-V30A and since no repair part was available in the stores system, Plant Engineering authorized replacing the existing Autoclave valves with Parker-Hannifin compression end valves. The results of post-installation testing were satisfactory.

OPERATING DATA REPORT

DOCKET NO. 50-289
DATE 9-30-90
COMPLETED BY W.G. Heysek
TELEPHONE (717) 948-8191

OPERATING STATUS

		NOTES
1. UNIT NAME:	THREE MILE ISLAND UNIT 1	
2. REPORTING PERIOD:	SEPT ,1990.	
3. LICENSED THERMAL POWER (MWT):	2568.	
4. NAMEPLATE RATING (GROSS MWE):	871.	
5. DESIGN ELECTRICAL RATING(NET MWE):	819.	
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE):	856.	
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE):	808.	

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS:

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE)
10. REASONS FOR RESTRICTIONS, IF ANY:

	THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	720.	6551.	140952.
12. NUMBER OF HOURS REACTOR WAS CRITICAL	720.0	4956.6	66955.2
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	242.8	2245.6
14. HOURS GENERATOR ON-LINE	720.0	4914.6	65910.7
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	1695496.	11694056.	161265950.
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	560456.	3905546.	54212215.
18. NET ELECTRICAL ENERGY GENERATED (MWH)	527403.	3657160.	50843649.
19. UNIT SERVICE FACTOR	100.0	75.0	46.8
20. UNIT AVAILABILITY FACTOR	100.0	75.0	46.8
21. UNIT CAPACITY FACTOR (USING MDC NET)	90.7	69.1	46.0
22. UNIT CAPACITY FACTOR (USING DER NET)	89.4	68.2	44.0
23. UNIT FORCED OUTAGE RATE	0.0	4.6	47.9

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
UNIT TMI-1
DATE 9-30-90
COMPLETED BY W.G. Heysek
TELEPHONE (717) 948-3191

MONTH: SEPT

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	767.	17	775.
2	758.	18	776.
3	764.	19	773.
4	768.	20	768.
5	762.	21	772.
6	758.	22	768.
7	755.	23	772.
8	767.	24	774.
9	771.	25	771.
10	761.	26	771.
11	761.	27	767.
12	757.	28	648.
13	757.	29	319.
14	760.	30	322.
15	762.	31	N/A
16	773.		

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH September 1990

DOCKET NO. 50-289
UNIT NAME TMI-1
DATE 10/15/90
COMPLETED BY W. G. Heysek
TELEPHONE (717) 948-8191

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report#	System Code 4 & 6	Component Code 5 & 6	Cause & Corrective Action to Prevent Recurrence
90-04	9/28 - 9/30/90	S	55	B	4	NA	SG	HTEXCH	The unit was reduced to 50% reactor power to locate and repair main condenser tube leaks.

1
F Forced
S Scheduled

2
Reason
A-Eqipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensing Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5 Exhibit 1 Same Source

6 Actually used Exhibits F & H NUREG 0161

REFUELING INFORMATION REQUEST

1. Name of Facility: **Three Mile Island Nuclear Station, Unit 1**

2. Scheduled date for next refueling shutdown: **October 4, 1991 (9R)**

3. Scheduled date for restart following current refueling: **NA**

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? **No.**

   If answer is yes, in general, what will these be?

   If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)? **No.**

   If no such review has taken place, when is it scheduled? **6/1/91.**

5. Scheduled date(s) for submitting proposed licensing action and supporting information: **None planned.**

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures: **None.**

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool: (a) **177** (b) **441**

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

   The present licensed capacity is 752. Planning to increase licensed capacity through fuel pool reracking is in progress.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

   1991 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).